

South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

February 26, 2007 NOC-AE-07002104 10CFR50.90

1001

U. S. Nuclear Regulatory Commission Attention: Document Control Desk One White Flint North 11555 Rockville Pike Rockville, MD 20852

South Texas Project Units 1 and 2 Docket Nos. STN 50-498, STN 50-499 Response to NRC Request for Additional Information on Proposed Change to Technical Specification 3.3.3.6, "Accident Monitoring Instrumentation"

- References: 1. Letter from T. J. Jordan, STPNOC, to NRC Document Control Desk dated October 2, 2006, "Revision to Proposed Change to Technical Specification 3.3.3.6, 'Accident Monitoring Instrumentation'" (TAC Nos. MD0934 and MD0935, ML062830032, NOC-AE-06002060)
 - 2. Informal NRC Request for Additional Information, E-mail from Mohan Thadani, NRC, to A. Harrison and S. Head, STPNOC, dated October 23, 2006 (ML062980024)

In Reference 1, STP Nuclear Operating Company (STPNOC) submitted a proposed amendment to South Texas Project Operating Licenses NPF-76 and NPF-80 to revise Technical Specification 3.3.3.6, "Accident Monitoring Instrumentation" with respect to the required action for inoperable Wide Range Reactor Coolant Temperature, Wide Range Steam Generator Level, and Auxiliary Feedwater Flow.

In Reference 2, the NRC staff requested additional information on with regard to the STPNOC application. The attachment to this letter responds to the NRC questions.

There are no commitments in this letter.

STI: 32105401



If there are any questions regarding the proposed amendment or the responses, please contact Mr. A. W. Harrison at (361) 972-7298 or me at (361) 972-7454.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on $\frac{2/26/07}{Date}$

Ma

Charles T. Bowman General Manager, Oversight

Attachments:

- 1. Response to NRC Request for Additional Information
- 2. Reactor Trip or Safety Injection (0POP05-EO-EO00)
- 3. Reactor Trip Response (0POP05-EO-ES01)

cc: (paper copy)

Regional Administrator, Region IV U. S. Nuclear Regulatory Commission 611 Ryan Plaza Drive, Suite 400 Arlington, Texas 76011-8064

Senior Resident Inspector U. S. Nuclear Regulatory Commission P. O. Box 289, Mail Code: MN116 Wadsworth, TX 77483

C. M. Canady City of Austin Electric Utility Department 721 Barton Springs Road Austin, TX 78704

Richard A. Ratliff Bureau of Radiation Control Texas Department of State Health Services 1100 West 49th Street Austin, TX 78756-3189 (electronic copy)

A. H. Gutterman, Esquire Morgan, Lewis & Bockius LLP

Mohan C. Thadani U. S. Nuclear Regulatory Commission

Steve Winn Christine Jacobs Eddy Daniels Marty Ryan NRG South Texas LP

Ed Alarcon J. J. Nesrsta R. K. Temple Kevin Pollo City Public Service

Jon C. Wood Cox Smith Matthews

C. Kirksey City of Austin

•

Attachment 1 Response to NRC Request for Additional Information

SOUTH TEXAS PROJECT REQUEST FOR AMENDMENTS PROPOSED CHANGES TO TECHNICAL SPECIFICATIONS FOR ACCIDENT MONITORING INSTRUMENTATION TAC NOS. MD0934 AND MD0935

AFW FLOW INSTRUMENT CONCERNS

Question #1: How does an INOPERABLE AFW flow instrument channel affect the OPERABILITY and function of its associated AFW train? Per the South Texas license amendment request, it appears that an INOPERABLE AFW flow instrument channel may prevent the associated AFW train from automatically delivering a pre-set range of feed flow to its associated S/G, since the AFW regulator valve is controlled by QPDS using AFW flow. Is this correct?

Response: That is correct. If the inoperable AFW flow affects the control function, then application of the definition of OPERABILITY would require an operability evaluation for the associated train of AFW, which is governed by TS 3.7.1.2. Note that the accident analyses do not credit automatic AFW flow control; however, the associated AFW train would be inoperable if the condition prevented AFW actuation or flow. In any case, the proposed Post-Accident Monitoring System (PAMS) actions apply only for inoperable indication of AFW flow.

For all questions, consider a range of malfunctions which would cause an AFW flow instrument to be considered INOPERABLE, to include: instrument failed high, instrument failed low, instrument failed at an intermediate AFW flow, and no signal from the AFW flow instrument to QPDS.

Question #1.a:	Does an INOPERABLE AFW flow instrument channel cause the associated AFW train to be INOPERABLE?
Response:	If the condition prevents AFW actuation or flow or the ability to regulate the flow between 550 gpm and 675 gpm, the associated train of AFW would be inoperable per TS 3.7.1.2.
Question #1.b:	Isn't it a part of the current design basis for an AFW train to AUTOMATICALLY deliver a pre-set range of feed flow given a valid AFW actuation signal? (e.g., SI signal, LOOP with S/G low-low water level). How does an INOPERABLE AFW flow instrument channel affect this design basis?
Response:	Yes. See response to 1.a above.
Question #1.c:	For accidents where AFW actuation occurs (i.e., many events from the South Texas FSAR chapter 15 accident analysis), a value of AFW flow had to be assumed. Per the proposed new TS action statements, one AFW flow channel instrument could be INOPERABLE indefinitely. How does

in the chapter 15 accident analysis?

an INOPERABLE AFW flow instrument affect the AFW flow(s) assumed

Example: Consider a steam generator tube rupture accident on say S/G A, as described in STP's FSAR. Add in a failed low AFW flow instrument for S/G A, which would be allowed indefinitely per the proposed TS change. Without operator action, will the AFW flow regulator valve to S/G A be open too far (due to the failed low AFW flow instrument) and perhaps lead to overfill on S/G A?

Response:

An inoperable AFW flow indication channel does not affect the Chapter 15 analyses. Only the indication function is affected by the proposed TS change. If only indication is affected, the AFW train will function as designed. If the condition affects the ability of the AFW train to start or for the regulating valve to control flow between 550 gpm and 675 gpm, then TS 3.7.1.2 applies. With AFW flow indication available in the other loops or alternate indication of SG level, then adequate level can be maintained in the SG for decay heat removal and cool down.

A failed low AFW flow transmitter will cause the regulating valve to open to its 675 gpm upper limit. The analysis assumes full flow for 10 minutes before the steam generator is isolated in accordance with the Emergency Operating Procedure (EOP). The EOP directs the operators to isolate AFW flow to the faulted steam generator.

Question #2: When an AFW flow instrument channel becomes INOPERABLE, it appears that operators can take manual action to control the associated AFW regulator valve. Is this correct?

Response: Yes. The Operator can take manual control of the regulator valve from the Control Room after AFW actuation by resetting the actuation signal. The EOPs direct the operators to monitor narrow range steam generator level and manually control AFW flow to maintain the level to greater than 14% narrow range. The wording below is typical for management of steam generator level. Note that the AFW flow rates are called out in the "Response Not Obtained" column. However, even if the AFW flow rate indication is not available, AFW operation will be indicated by the control board indication for the AFW pump and regulator valve position. Flow can be confirmed by observing the steam generator level indication.

STEP ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

__14 MONITOR Intact SG Levels:

___a. NR levels - GREATER THAN 14%[34%]

a. PERFORM the following:

- MAINTAIN total AFW flow GREATER THAN 576 GPM to restore NR level GREATER THAN 14% [34%] in at least one SG.
- 2) any AFW pump fails to start. THEN:
 - a) RESET all SG LO-LO level AFW actuations.
 - b) CLOSE applicable AFW regulating valve.
 - c) OPEN applicable AFW cross connects.
 - d) CONTROL AFW flow to LESS THAN 675 GPM per AFW pump

Question #2.a: Is this what operators do when an AFW flow channel instrument becomes INOPERABLE? If so, answer these associated questions:

No. Manual control is taken after AFW actuation by resetting the actuation signal, then taking manual control of the valve at the control board. There is not a manual/auto selector for the regulator valve.

- Question #2.a.1: How do operators know that an AFW flow channel instrument has become inoperable with AFW not running? Consider an AFW flow instrument failed high, failed low, instrument failed at an intermediate AFW flow, and no signal from the AFW flow instrument to QPDS. How would operators be alerted to these conditions? Alarms? Flow meter indications?
- Response: PAMS TS surveillance requirements include a monthly channel check and 18 month channel calibration for AFW Flow. Flow is observed in the TS surveillance for the AFW pumps that is performed once per 31 days on a staggered test basis.
- Question #2.b.1:Is taking manual control required by plant procedures? If so, please
explain the procedure(s) entered (alarm procedure(s),
operating/abnormal procedure(s)) and provide the procedures to the
NRC.

Response: See the response to Question 2 above.

Question #2.b.2: If taking manual control is an action required by procedures, in what amount of time after an AFW flow instrument is determined to be INOPERABLE will this action be taken?

Response: See the response to Question 2 above. Taking manual control of AFW is a response to steam generator level indication, not a response to a loss of AFW flow indication.

Question #2.b.3: Does taking manual control of the AFW regulator valve restore that AFW train to OPERABLE?

- Response: No. With the AFW flow input to the regulator valve inoperable, the associated train of AFW would be inoperable because it would not be able to meet its surveillance requirements. However, this question does not relate to the proposed TS change for PAMS, but pertains to the application of TS 3.7.1.2 for the AFW system.
- Question #2.c: Does taking manual control of the AFW regulator valve allow that train of AFW to return to being able to perform its design function of delivering a pre-set range of feed flow given a valid AFW actuation signal?

Response: As described above, the Operator can only take manual control of AFW flow by resetting the AFW actuation signal after an AFW actuation. The Operators can control AFW Flow to perform its design function to maintain SG level based on SG level indication. AFW will actuate after being reset if it receives an actuation signal.

Question #2.d: Consider an INOPERABLE AFW flow channel instrument, with the associated AFW regulator valve in manual. What would be the response of that AFW train to a LOOP (and associated loss of normal feedwater)? Compare this to the FSAR analysis, where that train's AFW pump will start on the LOOP, but go into recirculation until a valid S/G low-low level signal is received. Please justify any differences in AFW response.

Response: This question relates to the actuation and flow control function for AFW which is governed by TS 3.7.1.2, not by the PAMS TS that STPNOC is seeking to change.

However, the response of the AFW system would not be significantly affected. As previously described, the regulator valves are normally open and control flow between the limits of 550 gpm and 675 gpm when in automatic mode. With the valves in manual, upon receipt of an AFW actuation signal and a LOOP, the pump would start as required and initial flow to the SG would occur at a rate established by the AFW regulator valve position at the time of actuation (normally full-open). As discussed in the responses to Questions 2 and 5, the operators will manually manage AFW flow using SG level indication.

OTHER CONCERNS

Question #3: Please provide a listing for how the plant uses/where the output signal goes for each of the instruments considered in the proposed TS change:

RCS Loop Thot wide range RCS Loop Tcold wide range S/G water level wide range AFW flow

Use plant drawings as necessary, and include any alarms, feeds to other instruments, interlocks, and displays where the outputs of the instruments are utilized. (The NRC would like to verify that Thot, Tcold, S/G level are primarily for indication ONLY, and that only AFW flow serves another purpose (regulator valve control).

Response: The STPNOC application identifies the other functions provided by the PAMS instrumentation.

• The wide range reactor coolant inlet and outlet temperatures are input to the cold overpressure mitigation system (COMS). However, the TS for the COMS

function is not applicable in MODE 1, 2 & 3 where the Accident Monitoring Instrumentation TS applies.

- Steam Generator Level WR provides no control or protective function.
- AFW flow functions were described in the application and are the subject of other questions in this RAI. As discussed in the STPNOC responses, where the control functions are affected, the actions of TS 3.7.1.2 for the associated AFW train would apply.
- Question #4: Although in the LAR for STP it was stated how these instruments are used during an accident (e.g., EOP implementation, emergency action level classification), the NRC would like to conduct an independently verification. Would it be possible for STP to provide their entire EOP network to the NRC on a CD-rom? By entire EOP network, the NRC means:
 - 1. The "baseline" EOPs (e.g., E-0, E-1, E-2, E-3).
 - 2. Critical safety function status trees.
 - 3. Functional restoration procedures.

4. Emergency contingency procedures (i.e., loss of all AC power, loss of ECCS recirculation, uncontrolled depressurization of all steam generators, LOCA outside containment, steam generator tube rupture without pressurizer pressure control, etc.)

4. Event specific procedures (i.e., reactor trip response, re-diagnosis, natural circulation cooldown, SI termination, post LOCA cooldown and depressurization, transfer to cold leg recirculation, etc.).

5. Emergency action level classification procedure which includes what plant data are used and how to classify events.

Response: As discussed with the NRC staff in teleconference on November 8, 2006, STPNOC resolved this request by agreeing to provide pertinent excerpts from the procedures in the responses to these questions.

Specifically, STPNOC agreed to provide the procedural guidance and references to respond to a loss of off-site power with a reactor trip and loss of normal feedwater, assuming a failure of Train A and Train D actuation. The relevant procedural requirements are the procedure for Reactor Trip or Safety Injection (0POP05-EO-EO00) and Reactor Trip Response (0POP05-EO-ES01). These procedures are provided in Attachment 2 and Attachment 3, respectively. The operator action to manage AFW flow is highlighted. STPNOC considers this event successfully mitigated when the plant is stable in MODE 3, which is the licensing basis safe shutdown condition for STP. Success criteria includes keeping the pressurizer from going water-solid.

STP EOPs also contain certain conditions where AFW flow value is specified. For an ATWS, AFW flow of greater than 1080 gpm is required due to the heat load. For uncontrolled depressurization of all steam generators, AFW flow of 100 gpm per SG is required to maintain the SGs in a wetted condition and to minimize the cooldown. However, with AFW flow indication to two SGs still available in the control room, STP has adequate information to cooldown to RHR conditions.

OTHER EVENT-SPECIFIC CONCERNS

Question #5: For a Loss of Normal Feedwater flow event, as described in section 15.2.7 of the licensee's UFSAR, a LOOP is assumed with a failure of ESFAS train A to actuate. This results in only the B and C motor-driven AFW pumps running, with operator action required within 15 minutes to manually start either the A motor-driven AFW pump or the D steam-driven AFW pump. Explain how the operator action to manually start these AFW pumps would be affected assuming that the associated AFW flow indicator was INOPERABLE. Specifically:

Question #5.a: Do you believe operators can successfully manually start an AFW pump without flow indication?

Response: Yes. Not having AFW flow indication does not affect the ability of the operator to manually start the AFW pumps. Indication of AFW flow will be evidenced by control board indication of the running AFW pumps, regulator valve position, and by observing increasing steam generator water level.

In discussion with the staff, STPNOC agreed that the Bases for the PAMS AFW flow indication would be improved by identifying the alternate indications of AFW flow. The alternate flow indications on the control board (AFW pump running, regulator valve position, and steam generator water level), are STPNOC's pre-planned alternate indication of AFW flow should one channel of AFW flow be expected to be inoperable for longer than 30 days or if more than one channel of AFW flow is inoperable. Operators will be briefed on the use of the alternate indications if their use is required.

The STPNOC license amendment request dated October 2, 2006 included the Bases changes excerpted below. STPNOC will enhance these Bases with the highlighted additions.

For channels of AFW flow instrumentation, ACTION 35 applies only for loss of the indication function. If the control function of an AFW flow channel is inoperable, TS 3.7.1.2 is applicable.

ACTION 35.b requires entry into the 7-day shutdown action if two or more of the four required SG wide range level instruments or two or more of the four required AFW flow instruments are inoperable. The Completion Time of 7 days is based on the relatively low probability of an event requiring PAM instrument operation and the availability of alternate means to obtain the required information. There is safety margin in this requirement in the availability of the functionally diverse indications and that unavailability of the level indication or the AFW flow indication does not make the associated steam generator unavailable as a heat sink if it is receiving flow. In the absence of direct AFW flow indication, AFW flow is evidenced by control board indication of the running AFW pumps, the regulator valve position, and by observation of increasing steam generator water level. With respect to the LONF event, the 7-day action is conservative because control board indication of running AFW pumps and steam generator level can still be used to confirm AFW flow to the

steam generator in the absence of direct AFW flow indication in the unlikely event of an accident with two of the AFW flow channels inoperable. The alternate flow indications on the control board (AFW pump running, regulator valve position, and steam generator water level), are STPNOC's pre-planned alternate indication of AFW flow should one channel of AFW flow be expected to be inoperable for longer than 30 days or if more than one channel of AFW flow is inoperable. Operators will be briefed on the use of the alternate indications if their use is required.

With respect to the post-accident decay heat removal and determination of the need to initiate feed and bleed, the 7-day action is conservative because it can reasonably be expected that AFW flow and indication and steam generator level indication will be available for at least one generator in the unlikely event of an accident with two or more channels of either function inoperable. Continuous operation with two or more required channels inoperable in a function is not acceptable because the alternate indications may not fully meet all performance qualification requirements applied to the PAM instrumentation. Therefore, requiring restoration of at least three operable channels of the function limits the risk that the PAM function will be in a degraded condition should an accident occur.

Question #5.b:	How will operators verify that they have successfully started an
	AFW pump without flow indication?

Response:	See response to #5.a
Question #5.c:	Is there adequate guidance in plant procedures to ensure that operators can manually start an AFW pump without flow indication?
Response:	Yes. See Addendum 7 of attached procedure 0POP05-EO-ES01 as an example.
Question #5.d:	Are operators adequately trained such that they will be able to manually start an AFW pump without flow indication?
Response:	Yes. Operators understand that AFW flow can be determined by observing steam generator level and pump running indication.
Question #5.e:	Will operators be able to meet the 15-minute required action time, assuming one INOPERABLE AFW flow indicator (i.e., AFW flow indicator A is INOPERABLE, and operators choose to start the A AFW pump)
Response:	Yes. Inoperable AFW flow indication has no effect on operator response time.

Question #5.f:Will operators be able to meet the 15-minute required action time,
assuming two INOPERABLE AFW flow indicators (AFW flow
indicators A and D INOPERABLE).

Response: Yes. Inoperable AFW flow indication has no effect on operator response time.

Question #6: Consider again a steam generator tube rupture accident, assuming the ruptured tube occurs on S/G A. Also assume that the AFW flow indicator for train A is failed high and INOPERABLE, such that the AFW regulator valve for train A will be more closed due to the failed instrument. Under these circumstances, AFW flow to the A S/G will be less than the AFW flow to the B, C, and D S/Gs. Could this lower AFW flow to the A S/G perhaps mask the A S/G level increasing from the tube rupture, when compared to the B, C, and D S/Gs which have higher AFW flow?

Response: This question is relevant to the flow regulating function governed by TS 3.7.1.2 rather than the flow indication function governed by the PAMS TS that STPNOC is seeking to change. However, in addition to rising steam generator level, the operators will use the steam line radiation monitors, the N-16 monitors, and the steam generator blowdown radiation monitors to identify the ruptured steam generator. Consequently, any difference in AFW flow would not be expected to affect the proper identification of the ruptured steam generator.

Attachment 2 Reactor Trip or Safety Injection (0POP05-EO-EO00)



The AFW Flow references are highlighted on the following pages: Step 8 on p. 9 of 23 Step 22 on p. 18 of 23 Addendum 5, Step 5 on p. 3 of 8 Conditional Information Pages (last two pages of the procedure) Note that management of SG level is the success criteria for AFW flow. Isolation of AFW on the Conditional Information Pages does not require flow indication.

08/09/05 DATE EFFECTIVE

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION

0P0P05-E0-E000 Rev. 19

REACTOR TRIP OR SAFETY INJECTION

DEPARTMENT PROCEDURE

SAFETY RELATED (Q)

USAGE CONTROL: In Hand Controlling Station

LIST OF ATTACHMENTS:

......

o Addendum 1, Phase A Isolation Verification
o Addendum 2, Establishing Alternate Charging Flow Control
o Addendum 3, Emergency Electrical Loading Requirements
o Addendum 4, Sequencer Loading Verification - Mode III
o Addendum 5, Verification of SI Equipment Operation
o Conditional Information Page

This procedure is applicable in Modes 1, 2, and 3 with RCS pressure GREATER THAN 1000 PSIG.

REV. 19

PAGE 1 OF 23

PURPOSE

This procedure provides actions to verify proper response of the automatic protection systems following manual or automatic actuation of a Reactor Trip or Safety Injection, to assess plant conditions, and identify the appropriate recovery procedure.

SYMPTOMS OR ENTRY CONDITIONS

- 1) The following are symptoms that require a REACTOR TRIP, if one has not occurred:
 - o SR High Flux. 1/2 channels GREATER THAN OR EQUAL TO 10⁵ CPS
 - o IR High Flux, 1/2 channels current GREATER THAN OR EQUAL TO 25% power
 - o PR (Low Setpoint). 2/4 channels GREATER THAN OR EQUAL TO 25% power
 - o PR (High Setpoint), 2/4 channels GREATER THAN OR EQUAL TO 109% power
 - o PR High Rate, 2/4 channels rise GREATER THAN OR EQUAL TO 5% in LESS THAN OR EQUAL TO 2 SECONDS
 - o OTDT, 2/4 channels GREATER THAN setpoint
 - o OPDT, 2/4 channels GREATER THAN setpoint
 - o Pressurizer Low Pressure, 2/4 channels LESS THAN OR EQUAL TO 1870 PSIG
 - o Pressurizer High Pressure, 2/4 channels GREATER THAN OR EQUAL TO 2380 PSIG
 - o Pressurizer High Level, 2/4 channels GREATER THAN OR EQUAL TO 92%
 - o Single Loop Low Flow. 2/3 channels on 1/4 loops LESS THAN OR EQUAL TO 91.8% flow when power is GREATER THAN P-8
 - o Two Loop Low Flow, 2/3 channels on 2/4 loops LESS THAN OR EQUAL TO 91.8% flow when power is GREATER THAN P-7
 - o RCP Undervoltage. 2/4 sensors LESS THAN OR EQUAL TO 10,014 V
 - o RCP Underfrequency, 2/4 LESS THAN OR EQUAL TO 57.2 HZ
 - o SG LO-LO Level, 2/4 channels on 1/4 SG(s) LESS THAN OR EQUAL TO 20% of NR
 - o Turbine Trip, 2/3 channels auto stop oil pressure LESS THAN OR EQUAL TO 1245.8 psig OR 2/4 turbine throttle valves shut
 - o Safety Injection, 1/2 trains
 - o SSPS Urgent Failure, 2/2 logic train General Warning alarms

PAGE 2 OF 23

- 2) The following are symptoms of a REACTOR TRIP:
 - o Any reactor trip annunciator lit
 - o Rapid lowering in neutron level
 - o All shutdown and control banks fully inserted with rod bottom lights lit
- 3) The following are symptoms that require a REACTOR TRIP AND SAFETY INJECTION, if one has not occurred:
 - Pressurizer Low Pressure, 2/4 channels LESS THAN OR EQUAL TO 1857 PSIG and NOT BLOCKED
 - o Containment High Pressure, 2/3 channels GREATER THAN OR EQUAL TO 3 PSIG
 - o Low Compensated SG Pressure. 2/3 channels LESS THAN OR EQUAL TO 735 PSIG on any SG and NOT BLOCKED
- 4) The following are symptoms of a REACTOR TRIP AND SAFETY INJECTION:
 - o Any SI annunciator lit
 - o SI pumps running
 - o Phase A isolation
 - o STBY DGs running
- 5) The following are entry conditions for OPOPO5-EO-EOOO, REACTOR TRIP OR SAFETY INJECTION:
 - o A Reactor Trip resulting from the Manual actuation of 1/2 Reactor Trip handswitches.
 - o A Safety Injection resulting from the Manual actuation of 1/2 Safety Injection handswitches.
 - o An automatic Reactor Trip or Safety Injection.

ADVERSE CONTAINMENT CONDITIONS

IF any of the following conditions are met. THEN USE adverse containment values:

- o Containment pressure GREATER THAN OR EQUAL TO 5 PSIG.
- o Containment radiation levels GREATER THAN OR EQUAL TO 10⁵ R/HR.
- o Containment integrated radiation dose GREATER THAN OR EQUAL TO 10⁶ RADS.

REV. 19

PAGE 3 OF 23

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

 NOTE

 o
 Steps 1 through 4 are IMMEDIATE ACTION steps.

 o
 Foldout CIP page should be open.

 ____1
 VERIFY Reactor Trip:

 o
 Rod bottom lights - LIT

 a. Manually TRIP reactor using both

- o Reactor trip and bypass breakers OPEN
- o Neutron flux LOWERING

- a. Manually TRIP reactor using both reactor trip switches.
- b. IF reactor will NOT trip, THEN:
 - OPEN 480V LC 1K1(2K1) and 1L1(2L1) feeder breakers.
 - 2) <u>IF</u> reactor will <u>NOT</u> trip, <u>THEN</u> GO TO OPOP05-EO-FRS1, RESPONSE TO NUCLEAR POWER GENERATION -ATWS, Step 1 <u>AND</u> MONITOR Critical Safety Functions.
 - 3) <u>IF</u> reactor trip and bypass breakers DO <u>NOT</u> open, <u>THEN</u> DISPATCH an operator to open reactor trip and bypass breakers.

(60 ft EAB RM 323)

"REACTOR TRIP BREAKER R"

"REACTOR TRIP BREAKER S"

"BYPASS BREAKER R"

"BYPASS BREAKER S"

4) <u>WHEN</u> the reactor is verified tripped, <u>THEN</u> CLOSE 480V LC 1K1(2K1) and 1L1(2L1) feeder breakers.

0POP05-E0-E000 REACTOR TRIP OR S	REV. 19
UPOPOJ-EO-EOOO REACIOR IRIF OR S	PAGE 4 OF 23
STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
2 VERIFY Turbine Trip:	
a. VERIFY all turbine throttle	a. PERFORM the following:
valves – CLOSED	1) Manually TRIP turbine.
	2) <u>IF</u> turbine will <u>NOT</u> trip, <u>THEN</u> :
	a) PLACE EH pumps in PULL TO LOCK.
	b) Manually RUNBACK turbine.
	c) <u>IF</u> turbine throttle valves can <u>NOT</u> be closed, <u>THEN</u> CLOSE MSIVs and MSIBs.
b. Main generator output breaker - OPEN	b. Manually OPEN breaker.
c. Main steam to deaerator valves - CLOSED	c. Manually CLOSE valves.

I

.

0P0P05-E0-E000 REACTOR TRIP (DR SAFETY INJECTION PAGE 5 OF 23
STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
3 VERIFY Power To AC ESF Busses:	
<pre>a. AC ESF busses - AT LEAST ONE ENERGIZED o 4.16KV ESF bus o 480V ESF LCs o 480V ESF MCCs</pre>	 a. PERFORM the following: RESTORE power to at least one AC ESF bus by EMERGENCY STARTING STBY DG. <u>WHEN</u> STBY DG is running. <u>THEN</u> ENSURE STBY DG Output Breaker CLOSED. ENSURE "SPLY" and "OUTP" breakers for "4.16KV/480V XFMR"(s) - CLOSED. 4) <u>IF</u> power can <u>NOT</u> be restored to at least one AC ESF bus. <u>THEN</u> GO TO OPOPO5-EO-ECOO. LOSS OF ALL AC POWER. Step 1 <u>AND</u> MONITOR Critical Safety Functions
<pre>b. AC ESF busses - ALL ENERGIZED</pre>	 b. TRY to restore power to deenergized AC ESF busses. o Emergency START STBY DG. o <u>WHEN</u> STBY DG is running. <u>THEN</u> ENSURE STBY DG Output Breaker CLOSED. o ENSURE "SPLY" and "OUTP" breakers for "4.16KV/480V XFMR"(s) - CLOSED.

.

· ··-

٠

0P0P05	-EO-E000
--------	----------

REACTOR TRIP OR SAFETY INJECTION

PAGE 6 OF 23

ACTION/EXPECTED RESPONSE **RESPONSE NOT OBTAINED** STEP __4 CHECK SI Status: _____a. CHECK if SI is actuated a. PERFORM the following: o SI reactor trip first out 1) CHECK if SI is required: annunciator - LIT o Pressurizer pressure - LESS o ESF status monitoring red SI THAN OR EQUAL TO 1857 PSIG status lights - LIT AND NOT BLOCKED. OR o Containment pressure -GREATER THAN OR EQUAL TO 3 PSIG. OR o Any SG pressure - LESS THAN OR EQUAL TO 735 PSIG AND NOT BLOCKED. OR o As directed by US/SS. 2) IF SI is required, THEN manually ACTUATE. 3) IF SI is NOT required, THEN GO TO OPOPO5-EO-ESO1, REACTOR TRIP RESPONSE, Step 1 AND MONITOR Critical Safety Functions. ____b. VERIFY all trains of SI - ACTUATED b. Manually ACTUATE SI. o Train A ESF status monitoring red SI status lights - LIT o Train B ESF status monitoring red SI status lights - LIT o Train C ESF status monitoring red SI status lights - LIT _____

REV. 19

PAGE

0P0P05-E0-E000

REACTOR TRIP OR SAFETY INJECTION

. _

REV. 19

PAGE 7 OF 23

STEP

ACTION/EXPECTED RESPONSE

_. _._.

RESPONSE NOT OBTAINED

<u>NOTE</u>

ADDENDUM 5, VERIFICATION OF SI EQUIPMENT OPERATION is required to be completed before Functional Restoration Procedures are implemented. Addendum 5 ensures that the ESF systems are functional.

____5 VERIFY Proper SI Equipment Operation Per ADDENDUM 5, VERIFICATION OF SI EQUIPMENT OPERATION

0P0P05-E0-E000 REACTOR TRIP OR	REV. 19 SAFETY INJECTION PAGE 8 OF 23
STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
6 MONITOR If Containment Spray Is Required:	
a. Containment pressure - GREATER	a. PERFORM the following:
THAN 9.5 PSIG (QDPS)	 CHECK Containment pressure - HAS EXCEEDED 9.5 PSIG (CP-018)
	o "PRESS PR-0934"
	OR
	o "EXTD RNG PRESS PR-9759"
	2) <u>IF</u> containment pressure HAS EXCEEDED 9.5 PSIG, <u>THEN</u> GO TO Step 6.b.
	3) <u>IF</u> containment pressure HAS REMAINED LESS THAN 9.5 PSIG. <u>THEN</u> GO TO Step 7.
	·····
b. VERIFY containment spray ~ INITIATED	b. Manually INITIATE containment spray.
c. VERIFY containment isolation Phase B valves - CLOSED	c. Manually CLOSE valve(s).
o "INL OCIV MOV-0318"	
O "INL OCIV MOV-0291"	
O "OUTL ICIV MOV-0542"	
O "OUTL ICIV MOV-0403"	
O "OUTL OCIV MOV-0404"	
0 "OUTL OCIV FV-4493"	
d. STOP ALL RCPs	

000005	-EO-EOOO REACTOR TRIP OR		REV. 19
	EO EOOO KENCION INII OK	REACTOR TRIP OR SAFETY INJECTION	
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
7	CHECK RCP Seal Cooling:		
	a. Seal injection flow - NORMAL	a. PERFORM the follow	ving:
	o MAINTAIN seal injection flow between 6 and 13 gpm	injection. c) MAINTAIN sea	CCP(s) is lost, P(s). charging pump supplying seal
	<u>NOT</u> RCS temperature is controlled by the S be maintained at approximately 571°F a	G PORVs in AUTO, <u>THEN</u> RCS t	emperature
8	(MONITOR RCS Temperatures) • WITH ANY RCP RUNNING, RCS TAVG	PERFORM Substep a. OR based on RCS temperat	

STABLE AT OR TRENDING TO 567°F

OR

- WITHOUT ANY RCP RUNNING, RCS TCOLD STABLE AT OR TRENDING TO 567°F
- a. <u>IF</u> temperature LESS THAN 567°F <u>AND</u> lowering. <u>THEN</u>:
 - (1) MAINTAIN total AFW flow GREATER THAN 576 GPM until NR level is GREATER THAN 14% [34%] in at least one SG.)

.

N	REV. 19
14	PAGE 10 OF 23
OT OBTAINED	
LATE steam loa	ads as follows:
SOLATE steam d	ump drains.
SOLATE steam c	hest drains.
SOLATE steam t team line drai	
RIP all SGFPTs	۶.
TOP dumping st	eam.
SOLATE SGBD.	
cooldown conti SE MSIVs and M	
perature GREAT sing <u>THEN</u> :	'ER THAN 567°F
condenser is a <u>N</u> DUMP steam t	
condenser <u>NOT</u> <u>N</u> ENSURE SG PO trolling tempe O.	RVs
VERIFY SG PORV 1225 psig.	setpoints at
<u>IF</u> SG PORVs ar controlling te AUTO, <u>THEN</u> :	
1) Place SG PO	RV in MANUAL.
2) CONTROL tem STABLE AT 5	perature 67°F.
د ۲	controlling te AUTO, <u>THEN</u> : 1) Place SG PO 2) CONTROL tem

.

- - - - ·

.

		REV. 19
0P0P05 - 1	EO-EOOO REACTOR TRIP OR SA	PAGE 11 OF 23
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
9	CHECK Pressurizer Status:	
	_a. PORVs – CLOSED	a. <u>WHEN</u> pressurizer pressure LESS THAN 2335 PSIG, <u>THEN</u> :
	· · ·	1) Manually CLOSE PORVs.
		2) <u>IF</u> any PORV can <u>NOT</u> be closed. <u>THEN</u> manually CLOSE its isolation valve.
		3) <u>IF</u> isolation valve can <u>NOT</u> be closed, <u>THEN</u> PERFORM the following:
		 GO TO OPOPO5-EO-EO10, LOSS OF REACTOR OR SECONDARY COOLANT, Step 1.
		o MONITOR Critical Safety Functions.
		o <u>WHEN</u> Addendum 5 of this procedure is complete, <u>THEN</u> Functional Restoration Procedures may be IMPLEMENTED.
	b. Normal pressurizer spray valves – CLOSED	b. <u>WHEN</u> pressurizer pressure LESS THAN 2260 PSIG, <u>THEN</u> :
		 Manually CLOSE normal spray valve(s).
	N	2) <u>IF</u> spray valve(s) can <u>NOT</u> be closed, <u>THEN</u> PERFORM the following:
		a) STOP RCP 1A(2A).
		b) STOP RCP 1D(2D).
		c) <u>İF</u> RCS pressure continues to lower, <u>THEN</u> STOP all but one RCP.

Step 9 continued on next page.

0P0P05-I	EO-EOOO REACTOR TRIP OR SA	REV. 19
	NO ECCO REACTOR TRIP OR SA	PAGE 12 OF 23
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
Step 9 d	continued from previous page.	
	c. Auxiliary spray valve - CLOSED	c. Manually CLOSE auxiliary spray valve.
	d. Excess letdown isolation valves - CLOSED	d. Manually CLOSE excess letdown isolation valves.
10	MONITOR If RCPs Should Be Stopped:	
	a. HHSI pump - AT LEAST ONE RUNNING	a. GO TO Step 11.
	х.	
	b. RCS pressure – LESS THAN 1430 PSIG	b. GO TO Step 11.

.

ν.

____c. STOP all RCPs

OPOP05-E0-E000 REACTOR TRIP OR SAFETY INJECTION		REV. 19	
		SAFELL INJECTION	PAGE 13 OF 23
TEP ACTION/EXP	ECTED RESPONSE	RESPONSE NOT OBTAINED	
11 VERIFY The Fol Isolation Valv	lowing Containment ves – CLOSED		
a. Seal returr	n isolation valves	a. <u>IF</u> neither seal r valve CAN be veri <u>THEN</u> DISPATCH ope seal return OCIV	fied closed, rator to ensure closed
		(29 ft MAB RM 108 "1(2)-CV-MOV-0079 "RCP SEAL WATER R "CONTAINMENT ISOL "OPERATOR"	" ETURN ORC"
	atmosphere radiation lation valves	b. <u>IF</u> neither valve line <u>OR</u> neither v return line CAN b closed, <u>THEN</u> DISP ensure OCIVs clos	alve in the e verified ATCH operator to
	(41 ft MAB RM 216 "1(2)-RA-MOV-0004 "RCB EXHAUST RT-8 "ORC ISOLATION MO	" 011 SUPPLY"	
		"1(2)-RA-MOV-0006 "RCB EXHAUST RT-8 "ORC ISLOATION MO	011 RETURN"

.

OPOP05-EO-E000 REACTOR TRIP OR SAFET		AFETY INJECTION
		PAGE 14 OF 2
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
12	CHECK If SG Secondary Pressure Boundary Intact:	
_	_a. CHECK pressures in all SGs - o CONTROLLED <u>OR</u> RISING	a. <u>IF</u> any faulted SG is <u>NOT</u> isolated, <u>AND</u> is <u>NOT</u> needed for RCS cooldown, <u>THEN</u> PERFORM the
	• GREATER THAN CONTAINMENT PRESSURE	 following: o GO TO OPOPO5-E0-E020, FAULTED STEAM GENERATOR ISOLATION, Step 1. o MONITOR Critical Safety Functions. o <u>WHEN</u> Addendum 5 of this procedure is complete. <u>THEN</u> Functional Restoration Procedures may be IMPLEMENTED.
13	<pre>CHECK If SG Tubes Are Intact: o Main steamline radiation - NORMAL o IF SG blowdown in service, THEN SG blowdown radiation - NORMAL o CARS pump radiation - NORMAL o NO SG level rising in an uncontrolled manner</pre>	 GO TO OPOPO5-EO-EO30. STEAM GENERATOR TUBE RUPTURE. Step 1. MONITOR Critical Safety Functions. <u>WHEN</u> Addendum 5 of this procedure is complete. <u>THEN</u> Functional Restoration Procedures may be IMPLEMENTED.
14	CHECK IF RCS Is Intact: o Containment radiation – NORMAL o Containment pressure – NORMAL o Containment wide range water level – NORMAL	 GO TO OPOPO5-EO-EO10, LOSS OF REACTOR OR SECONDARY COOLANT. Step 1. o MONITOR Critical Safety Functions. o <u>WHEN</u> Addendum 5 of this procedure is complete. <u>THEN</u> Functional Restoration Procedures may be IMPLEMENTED.

.

.

· · · · · · · · ·

._....

0P0P05-	E0-E000
---------	---------

REACTOR TRIP OR SAFETY INJECTION

REV. 19

PAGE 15 OF 23

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
	ONITOR If SI Flow Should Be erminated:	
a	. RCS subcooling based on core exit T/Cs - GREATER THAN 35°F	a. GO TO Step 16.
b	. Secondary heat sink criteria - SATISFIED	b. GO TO Step 16.
	o Total AFW flow to SGs – GREATER THAN 576 GPM	
	OR	
	o NR level in at least one SG – GREATER THAN 14%	
c	. RCS pressure criteria – SATISFIED	c. GO TO Step 16.
	o Pressure – GREATER THAN 1745 PSIG	
	o Pressure – STABLE OR RISING	
d	. Pressurizer level - GREATER THAN	d. PERFORM the following:
	8%	1) USE normal pressurizer spray to stabilize RCS pressure.
		2) <u>IF</u> normal spray <u>NOT</u> evailable, <u>THEN</u> :
		a) ENSURE normal spray valves are closed.
		b) USE auxiliary spray if available.
		3) GO TO Step 16.

Step 15 continued on next page.

0P0P05-E0-E000 REACTOR TRIP OR SAFETY INJECTION		REV. 19 FETY INJECTION
		PAGE 16 OF 2
TEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
tep 1	5 continued from previous page. e GO TO OPOPO5-EO-ES11. SI TERMINATION. Step 1	
	1) Addendum 5 of this procedure complete	 PERFORM the following: a) MONITOR Critical Safety Functions. b) <u>WHEN</u> Addendum 5 of this procedure is complete, <u>THEN</u> Functional Restoration Procedures may be IMPLEMENTED.
16	2) MONITOR Critical Safety Functions <u>AND</u> IMPLEMENT Functional Restoration Procedures as required INITIATE MONITORING of Critical	
_	Safety Functions a. Addendum 5 of this procedure complete	 a. PERFORM the following: 1) MONITOR Critical Safety Functions. 2) <u>WHEN</u> Addendum 5 of this procedure is complete, <u>THEN</u>
		Functional Restoration Procedures may be IMPLEMENTED. 3) GO TO Step 17.
. ت 	b. MONITOR Critical Safety Functions <u>AND</u> IMPLEMENT Functional Restoration Procedures as required	
17	RESET SI	
18	RESET ESF Load Sequencers	
19	RESET Containment Isolation Phase A	

0P0P05-E0-E000

REACTOR TRIP OR SAFETY INJECTION

REV. 19

. . .

PAGE 17 OF 23

STEP ACTIO

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

_____20 RESET Containment Isolation Phase B

CAUTION

<u>IF</u> a pressurizer spray valve has been determined to be stuck open <u>AND</u> more than one RCP is operating, <u>THEN</u> IA to containment should <u>NOT</u> be established to avoid the initiation of an uncontrolled RCS depressurization.

____21 ESTABLISH IA To Containment:

____a. IA pressure - GREATER THAN 95 PSIG

____b. OPEN IA OCIV

- a. DISPATCH an operator to start an IA compressor.
- b. <u>IF</u> IA OCIV can <u>NOT</u> be opened from the control room, <u>THEN</u> DISPATCH operator to perform the following:

(10 ft MAB PENT space)

1) UNLOCK and CLOSE:

"1(2)-IA-1515" "RCB INSTRUMENT AIR HEADER" "FV-8565 MANUAL OVERRIDE" "VALVE"

2) UNLOCK and OPEN:

"1(2)-IA-1516" "RCB INSTRUMENT AIR HEADER" "FV-8565 MANUAL OVERRIDE" "VALVE"

0P0P05 - E0	-E000 REACTOR TRIP OR S	AFETY INJECTION	REV. 19 PAGE 18 OF 23
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	FAGE 16 UF 23
22) (N	ONITOR SG Levels:)		
.	. (NR levels) () (GREATER THAN 14%)	 (a. PERFORM the follow (1) MAINTAIN total GREATER THAN 57 (restore NR leve (14% in at least (2) IF any AFW pump (start, THEN:) (a) RESET all SG (AFW actuation) (b) CLOSE applic (regulating v) (c), OPEN applica (connects.) (d) CONTROL AFW (THAN 675 GPM) 	AFW flow 6 GPM to) 1 GREATER THAN one SG. 5 fails to 6 LO-LO Level ons. 2 able AFW valve. ble AFW cross
(<u>∵</u>)b	. (CONTROL AFW flow to maintain NR) (levels BETWEEN 22% and 50%)	b. <u>IF</u> NR level in any to rise in an unco manner. <u>THEN</u> GO TO OPOPO5-EO-EO30, ST TUBE RUPTURE, Step Critical Safety Fu	ntrolled EAM GENERATOR 1. <u>AND</u> MONITOR
c	. CHECK for a tube rupture – NO UNEXPLAINED SG LEVEL TREND WHERE LEVEL IS CONSTANT OR RISING WITH LOW AFW FLOW	-	30. STEAM TURE. Step 1

...

.

.. _..... _

0P0P05-E0-E000

REACTOR TRIP OR SAFETY INJECTION

REV. 19

PAGE 19 OF 23

.

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
23	CHBCK Secondary Radiation:	
	a. PERFORM the following:	
	1) RESET SG LO-LO level AFW actuations	
	2) RESET SG blowdown and sample isolations	
	3) NOTIFY Chemistry to sample all SGs for activity	
	b. VERIFY SG sample activity – NORMAL	b. <u>IF</u> SG sample activity is verified abnormal, <u>THEN</u> GO TO OPOP05-EO-EO30, STEAM GENERATOR TUBE RUPTURE, Step 1 <u>AND</u> MONITOR Critical Safety Functions.
24	CHECK MAB Radiation - NORMAL	<u>IF</u> the cause of the abnormal condition is a loss of RCS inventory outside containment. <u>THEN</u> GO TO OPOPO5-EO-EC12, LOCA OUTSIDE CONTAINMENT, Step 1 <u>AND</u> MONITOR Critical Safety Functions.
25	CHECK FHB Conditions:	<u>IF</u> the cause of the abnormal condition is a loss of RCS inventory
	o FHB area radiation monitors – NORMAL	outside containment, <u>THEN</u> GO TO OPOPO5-EO-EC12, LOCA OUTSIDE CONTAINMENT, Step 1 <u>AND</u> MONITOR
	o FHB ECCS pump sump levels – NORMAL	Critical Safety Functions.
26	CHECK PRT Conditions - NORMAL	EVALUATE cause of abnormal condition.
		••••••

0P0P05-E0-E000 REACTOR TRIP OR SAFETY INJECTION		REV. 19
CTOROS-BO-BOOD REACTOR TRIP OR SA		PAGE 20 OF 2
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
27	MONITOR IF LHSI Pumps Should Be Stopped:	
	a. CHECK RCS pressure:	1
	1) Pressure – GREATER THAN 415 PSIG	1) GO TO OPOPO5-EO-EO1O, LOSS OF REACTOR OR SECONDARY COOLANT, Step 1 <u>AND</u> MONITOR Critical Safety Functions.
	2) Pressure - STABLE OR RISING	2) GO TO Step 28.
_	b. STOP LHSI pumps and PLACE in AUTO	
28	ESTABLISH Charging Flow:	
	a. CCPs - AT LEAST ONE RUNNING	a. PERFORM the following:
		 CLOSE seal injection isolation valves.
		 "SEAL INJ ISOL MOV-0033A" "SEAL INJ ISOL MOV-0033B" "SEAL INJ ISOL MOV-0033C" "SEAL INJ ISOL MOV-0033D"
		2) CLOSE the CCP discharge valve for the CCP to be started.
		 CLOSE the charging flow control valve.
		 4) <u>IF</u> charging flow control valve will <u>NOT</u> close, <u>THEN</u> ESTABLISH charging flow to maintain pressurizer level GREATER THAN 8% per ADDENDUM 2. ESTABLISHING ALTERNATE CHARGING FLOW CONTROL <u>AND</u> GO TO Step 29.
		5) OPEN the recirculation valve for the CCP to be started.
Step 2	8 continued on next page.	6) START one CCP.

0P0P05-E	POPO5-EO-EOOO REACTOR TRIP OR SAFETY INJECTION		REV. 19 PAGE 21 OF 21
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
Step 28	continued from previous page.	· · · · · · · · · · · · · · · · · · ·	
		7) OPEN the CCP discharge valve for the pump that was started	Ι.
			• • •
	b. Charging flow - ESTABLISHED	b. PERFORM the following:	
	· · ·	1) ENSURE Containment Isolation Phase A RESET.	
		 ENSURE charging flow control valve CLOSED. 	
		3) <u>IF</u> charging flow control valv will <u>NOT</u> close, <u>THEN</u> ESTABLIS: charging flow to maintain pressurizer level GREATER THA 8% per ADDENDUM 2. ESTABLISHING ALTERNATE CHARGING FLOW CONTROL <u>AND</u> GO TO Step 29.	H
		4) ENSURE CCP discharge valves open.	
	· · · ·	5) ENSURE normal or alternate charging isolation valve open	ι.
		6) ENSURE charging OCIV open.	
		7) <u>IF</u> charging OCIV will <u>NOT</u> open, <u>THEN</u> DISPATCH operator to open charging OCIV:	
		(29 ft MAB RM 108C)	
	•	"1(2)-CV-MOV-0025" "CVCS CHARGING" "ORC CONTAINMENT ISOLATION" "MOV OPERATOR"	

0POP05-E0-E000 REACTOR TRIP OR SA		FETY INJECTION	REV. 19
			PAGE 22 OF 2
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
Step 28	continued from previous page.		
	c. CONTROL charging flow to maintain pressurizer level GREATER THAN 8%	c. <u>IF</u> charging flow of will <u>NOT</u> control of <u>THEN</u> MAINTAIN press GREATER THAN 8% pe ESTABLISHING ALTER FLOW CONTROL.	charging flow, ssurizer level er ADDENDUM 2,
	MONITOR If STBY DGs Should Be Stopped:		
	a. VERIFY AC ESF busses – ENERGIZED BY OFFSITE POWER	a. TRY to restore off	-
o 4.16KV^ESF b	o 4.16KV^ESF bus	1) REFER TO OPOPO4 RESPONSE TO LOS 13.8 OR 4.16 BU	S OF ANY OR ALL
	o 480V ESF LCs	2) <u>IF</u> offsite powe	
o 480V ESF MCCs	restored, <u>THEN</u> desired equipme ESF busses, REE 3, EMERGENCY EI LOADING REQUIRE	manually LOAD ent on the AC 'ER TO ADDENDUM JECTRICAL	
		3) <u>WHEN</u> offsite po	
		restored, <u>THEN</u> Steps 29.b. 29.	
			· • • • • • • • • • • • • • • • • • • •
	b. RESET any unloaded DG(s) non-emergency trips		
	c. RELEASE any unloaded DG(s) from EMERGENCY mode		
	d. STOP any unloaded DG(s)		

· --- --- ·

. . . .

TEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
a. CHECK CCW pumps - AT LEAST TWO RUNNING	a. PERFORM the following:
	 CONSULT TSC Staff regarding the operation of SFPC with LESS THAN two CCW pumps running.
	2) <u>IF</u> SFPC can <u>NOT</u> be placed in service, <u>THEN</u> GO TO Step 31.
b. ALIGN CCW flow to SFPC heat exchanger(s)	b. <u>IF</u> CCW flow can <u>NOT</u> be aligned from the main control board, <u>THE</u> DISPATCH operator to open the following valves:
	(29 ft MAB RM 106A SW corner)
	o "1(2)-CC-MOV-0447" "SPENT FUEL POOL HXS" "CCW SUPPLY HEADER" "FIRST ISOLATION MOV OPERATOR"
	<pre>o "1(2)-CC-MOV-0032" "SPENT FUEL POOL HXS" "CCW SUPPLY HEADER" "SECOND ISOLATION MOV OPERATOR</pre>
c. CHECK SFPC pump - RUNNING	c. START SFPC pump(s). REFER TO OPOPO2-FC-0001. SPENT FUEL POOL COOLING AND CLEANUP SYSTEM.
	·
31 RETURN TO Step 8	· ·

REACTOR TRIP OR SAFETY INJECTION

0P0P05-E0-E000

REV. 19

PAGE 23 OF 23

REACTOR TRIP OR SAFETY INJECTION

REV. 19

PAGE 1 OF 4

ADDENDUM 1 PHASE A ISOLATION VERIFICATION

DEVICE	POSITION	<u>CHECK</u>
<u>Fire Water to Containment</u>	,	
*FIRE WTR OCIV FV-0756"	CLOSED	
<u>Instrument Air to Containment</u>		
"IA OCIV FV-8565"	CLOSED	
Containment H2 Monitoring		
"VPI FV-4101"	CLOSED	
"VPI FV-4127"	CLOSED	
"VPI FV-4104"	CLOSED	
"VPI FV-4133"	CLOSED	
"H2 SAMPLE INL ICIV FV-4135"	CLOSED	
"H2 SAMPLE DISCH ICIV FV-4128"	CLOSED	
"H2 SAMPLE INL ICIV FV-4136"	CLOSED	
"H2 SAMPLE DISCH ICIV FV-4134"	CLOSED	
Primary Sampling System		
"RHR SAMPLE ICIV FV-4823"	CLOSED	
"RHR SAMPLE OCIV FV-4461"	CLOSED	
"SI ACC SAMPLE ICIV FV-4824"	CLOSED	<u> </u>
"SI ACC SAMPLE OCIV FV-4466"	CLOSED	
"RCS LOOP 1A(2A) Th SAMPLE ICIV" "FV-4454"	CLOSED	

.

REACTOR TRIP OR SAFETY INJECTION

_ -- -

.

REV. 19

PAGE 2 OF 4

ADDENDUM 1 PHASE A ISOLATION VERIFICATION

DEVICE	POSITION	CHECK
<u>Primary Sampling System (con't)</u>		
"RCS LOOP 1C(2C) Th SAMPLE ICIV" "FV-4455"	CLOSED	
"RCS SAMPLE OCIV FV-4456"	CLOSED	
"PRZR LIQ SAMPLE OCIV FV-4451B"	CLOSED	
"PRZR LIQ SAMPLE ICIV FV-4451"	CLOSED	
"PRZR VAPOR SAMPLE OCIV FV-4452"	CLOSED	
"PRZR VAPOR SAMPLE ICIV FV-4450"	CLOSED	
<u>Post Accident Sample System</u>		
"CNTMT SUMP SAMPLE OCIV FV-2453"	CLOSED	
"RHR SAMPLE OCIV FV-2454"	CLOSED	
"RCS SAMPLE OCIV FV-2455/2455A"	CLOSED	
"CNTMT AIR SAMPLE OCIV FV-2456"	CLOSED	
"RETURN TO PRT OCIV FV-2458"	CLOSED	
"CNTMT AIR RETURN OCIV FV-2457"	CLOSED	
SI Accumulators		
"TEST LN ICIV FV-3970"	CLOSED	
"TEST LN OCIV FV-3971"	. CLOSED	
"N2 SPLY OCIV FV-3983"	CLOSED	

REV. 19

· ··-·

0P0P05-E0-E000

. · · .

. –

REACTOR TRIP OR SAFETY INJECTION

PAGE 3 OF 4

· · · · --

PHASE A ISOLATION VERIFICATION									
DEVICE	POSITION	CHECK							
<u>Pressur</u> izer Relief Tank									
"OCIV FV-3652"	CLOSED								
"ICIV FV-3653"	CLOSED								
"OCIV FV-3651"	CLOSED								
<u>Reactor Coolant Drain Tank</u>									
"OCIV FV-4913"	CLOSED								
"ICIV MOV-0312"	' CLOSED								
"OCIV FV-4919"	CLOSED								
"ICIV FV-4920"	CLOSED								
<u>Containment Normal Sump</u>									
"DISCH ICIV MOV-0064"	CLOSED								
"DISCH OCIV FV-7800"	CLOSED								
<u>CVCS Letdown</u>									
"OCIV MOV-0024"	CLOSED								
"ICIV MOV-0023"	CLOSED								
"LTDN ORIF HDR ISOL FV-0011"	CLOSED								
<u>CVCS Seal Return</u>									
"SEAL RTN ICIV MOV-0077"	CLOSED								
"SEAL RTN OCIV MOV-0079"	CLOSED								

<u>ADDENDUM 1</u> PHASE A ISOLATION VERIFICATION

REACTOR TRIP OR SAFETY INJECTION

REV. 19

PAGE 4 OF 4

ADDENDUM 1 PHASE A ISOLATION VERIFICATION

DEVICE	POSITION	<u>CHECK</u>
CVCS Charging		
"OCIV MOV-0025"	CLOSED	
Personnel Airlock Seal OCIVs		
"INNER SEAL FV-1025"	CLOSED	
"INNER SEAL FV-1028"	CLOSED	<u> </u>
"OUTER SEAL FV-1026"	CLOSED	
"OUTER SEAL FV-1027"	CLOSED	

STEP

REACTOR TRIP OR SAFETY INJECTION

PAGE 1 OF 5

REV. 19

ADDENDUM_2

ESTABLISHING ALTERNATE CHARGING FLOW CONTROL

CAUTION IF RCP Seal Cooling has been lost and Seal Inlet Temperature has exceeded 230°F, THEN Seal Injection SHALL NOT be established to affected RCPs per this Addendum. 1 DISPATCH operator to perform the following: _____ a. THROTTLE charging flow control manual bypass two turns open (19 ft MAB RM 79) "1(2)-CV-0255" "CVCS CHARGING DISCHARGE" "FCV-0205 BYPASS VALVE" _ b. CLOSE charging flow control manual inlet isolation valve (19 ft MAB RM 79) *1(2)-CV-0254A* "CVCS CHARGING DISCHARGE"

2 CHECK normal or alternate charging loop isolation valve - OPEN

"FCV-0205 INLET VALVE"

OPEN normal or alternate charging loop isolation valve.

PAGE

ACTION/EXPECTED RESPONSE

RESPONSE

RESPONSE NOT OBTAINED

.

. _____

. .

REACTOR TRIP OR SAFETY INJECTION

REV. 19

PAGE 2 OF 5

	<u>ADDENDI</u> ESTABLISHING ALTERNATE (
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
3	CHECK charging OCIV - OPEN	PERFORM the following:
		a. OPEN charging OCIV.
		b. <u>IF</u> charging OCIV will <u>NOT</u> open. <u>THEN</u> DISPATCH operator to open charging OCIV.
		(29 FT MAB RM 108C)
		"1(2)-CV-MOV-0025" "CVCS CHARGING ORC" "CONTAINMENT ISOLATION" "MOV OPERATOR"
4	VERIFY RCP seal inlet temperatures have remained – LESS THAN 230°F	CLOSE affected seal injection OCIV's:
		 SEAL INJ ISOL MOV-0033A" "SEAL INJ ISOL MOV-0033B" "SEAL INJ ISOL MOV-0033C" "SEAL INJ ISOL MOV-0033D"
5	ESTABLISH charging flow:	
	a. CHECK CCP suction aligned to RWST	a. PERFORM the following:
		 OPEN at least one CCP suction valve from the RWST.
		 CLOSE at least one CCP suction valve from the VCT.
	b. CHECK CCP(s) - RUNNING	b. PERFORM the following:
		 OPEN the recirculation valve(s for the CCP(s) to be started.
		2) START the CCP(s)
	_c. ENSURE running CCP(s) discharge valve – OPEN	c. <u>IF</u> running CCP(s) discharge valve can <u>NOT</u> be OPENED. <u>THEN</u> GO TO Step 6 of this Addendum. OBSERVE the CAUTION prior to Step 6.

Step 5 continued next page

ł

REACTOR TRIP OR SAFETY INJECTION

PAGE 3 OF 5

REV. 19

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
	antimud continued from energieus pess	······································
	continued continued from previous page	
	d. DISPATCH operator to adjust charging flow control manual bypass to achieve desired flowrate	
	(19 ft MAB RM 79)	
	"1(2)-CV-0255" "CVCS CHARGING DISCHARGE" "FCV-0205 BYPASS VALVE"	
	e. CLOSE the running CCP(s) recirculation valve	:
	f. CHECK RCP seal injection flow - BETWEEN 6 and 13 GPM	f. <u>IF</u> RCP Seal Inlet temperatures have remained - LESS THAN 230°F, <u>THEN</u> PERFORM the following:
		 ENSURE seal injection OCIV(s) OPEN.
		 "SEAL INJ ISOL MOV-0033A" "SEAL INJ ISOL MOV-0033B" "SEAL INJ ISOL MOV-0033C" "SEAL INJ ISOL MOV-0033D"
		2) ADJUST 1(2)-CV-HCV-0218 to establish between 6 and 13 gpm RCP seal injection flow.

.

STEP

ACTION/EXPECTED RESPONSE

REACTOR TRIP OR SAFETY INJECTION

PAGE 4 OF 5

REV. 19

ADDENDUM 2 ESTABLISHING ALTERNATE CHARGING FLOW CONTROL

RESPONSE NOT OBTAINED

CAUTION IF RCP seals have <u>NOT</u> been previously isolated. <u>THEN</u> maximum seal injection flow will result when CCP 1B(2B) Alt Discharge to RCP Seal Isolation CV-0236B OR CCP 1A(2A) Bypass Isol MOV-8348 is opened. DISPATCH operator to perform 6 the following: _ a. OPEN alternate discharge to RCP seals: (19 ft MAB RM 79) "1(2)-CV-0236B" "CENTRIFUGAL CHARGING PUMP 1B(2B)" "ALT RCP SEAL SUPPLY VALVE" 7 ENSURE CCP 1B(2B) discharge isolation MOV-8377B - CLOSED 8 CHECK CCP 1A(2A) - RUNNING GO TO Step 9 of this Addendum. ____ a. OPEN CCP 1A(2A) Bypass Isol MOV-8348 ____ b. ENSURE CCP 1A(2A) discharge isolation MOV-8377A - CLOSED

•"

REACTOR TRIP OR SAFETY INJECTION

REV. 19

PAGE 5 OF 5

	ADDENDUM 2 ESTABLISHING ALTERNATE CHARGING FLOW CONTROL								
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED							
9	ESTABLISH desired charging flow: a. DISPATCH operator to open: (19 ft MAB RM 79) "1(2)-CV-0255" "CVCS CHARGING DISCHARGE"								
10	 "FCV-0205 BYPASS VALVE" b. THROTTLE HCV-0206 to establish desired charging flow c. CLOSE the running CCP(s) recirculation valve VERIFY RCP seal injection temperature has remained - LESS THAN 230°F a. ENSURE seal injection OCIV(s) OPEN 	RETURN TO procedure step in effect.							
	 o "SEAL INJ ISOL MOV-0033A" o "SEAL INJ ISOL MOV-0033B" o "SEAL INJ ISOL MOV-0033C" o "SEAL INJ ISOL MOV-0033D" 								
11	DISPATCH operator to THROTTLE Alternate Source to Seal Injection Isolation valve to establish between 6 and 13 gpm RCP seal flow (19 ft MAB RM 79)								
12	"1(2)-CV-0246" "CVCS CHARGING PUMPS" "ALT RCP SEAL SUPPLY ISOL VALVE" RETURN TO procedure step in effect								

1

REACTOR TRIP OR SAFETY INJECTION

REV. 19

PAGE 1 OF 1

Components	Train A (KW)	Train B (KW)	Train C (KW)
Pressurizer Heaters 1A/1B (2A/2B)	431.0	N/A	431.0
Hydrogen Recombiners	N/A	75.0	75.0
RHR Pumps	222.2	222.2	222.2
RMW Pumps	N/A	41.0	41.0
SFPC Pumps	N/A	140.4	140.4
SFP Air Handling Unit	N/A	0.5	0.5
Centrifugal Charging Pumps	450.0 ·	N/A	450.0
BA Transfer Pump Room Fans	0.4	N/A	0.4
BAT Pumps	27.0	N/A	27.0
RMW Pumps Air Handling Unit	N/A	1.9	1.9
480V MCCs 1A5/1B5/1C5 (2A5/2B5/2C5)	171.5	72.5	92.2

ADDENDUM 3 EMERGENCY ELECTRICAL LOADING REQUIREMENTS

REV. 19

0P0P05-E0-E000

REACTOR TRIP OR SAFETY INJECTION

PAGE 1 OF 2

ADDENDUM 4

SEQUENCER LOADING VERIFICATION - MODE III

STEP

.....

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION

- o Equipment should NOT be manually loaded on an ESF Bus until the respective ESF Load Sequencer has completed its automatic sequence OR it has been determined that the respective ESF Load Sequencer has failed to operate.
- o <u>IF</u> a LOOP or SI occurs, <u>AND</u> a train of Essential Chilled Water fails, <u>THEN</u> the corresponding EAB HVAC fans should be stopped within 30 minutes to prevent heat buildup in the Auxiliary Shutdown/QDPS rooms.
 - 1 VERIFY the following equipment -LOADED ON ESF BUS

Manually LOAD equipment <u>NOT</u> previously secured by the Emergency Operating Procedures.

·· ---- ·· ·

COMPONENT	TRAIN "A"	TRAIN "B"	TRAIN "C"
480V LC Feeder Breakers			
ECW Pumps			
HHSI Pumps			
LHSI Pumps			
CNTMT Spray Pumps (If Required,max. of 2)			
RCFCs			
CCW Pumps			
AFW Pumps			
Essential Chill Water Pumps			
EAB Supply and Return HVAC Fans			
CRE Supply, Return, C/U and M/U HVAC Fans			
FHB Main Exhaust HVAC Fans			
Essential Chillers			

¥

:

REACTOR TRIP OR SAFETY INJECTION

REV. 19

PAGE 2 OF 2

	<u>ADDENDUM 4</u> SEQUENCER LOADING VERIFICATION - MODE III							
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED						
2	RESET ESF load sequencers							
3	RESET Pressurizer Backup Heaters 1A(2A) and 1B(2B)							
	 a. Place Control Room Handswitches in OFF b. RETURN Control Room Handswitches to the position required for current plant conditions 	1						
4	STOP any equipment previously secured by any Emergency Operating Procedure							
5	CHECK ALL Essential Chilled Water Trains - IN SERVICE	SECURE the corresponding EAB HVA(for any Essential Chilled Water Train that is <u>NOT</u> operating.						
6	RETURN TO procedure step in effect							

0	P	0	P	0	5	-	ΕO		Ε	0	0	0	
---	---	---	---	---	---	---	----	--	---	---	---	---	--

REACTOR TRIP OR SAFETY INJECTION

REV. 19

PAGE 1 OF 8

	AI	DEN	<u>1DUM 5</u>	
VERIFICATION	OF	SI	EQUIPMENT	OPERATION

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION Functional Restoration Procedures (FRPs) SHOULD NOT be IMPLEMENTED until this Addendum is complete. See OPOPO1-ZA-0018 "EOP User's Guide" for exceptions. VERIFY FW isolation: 1 a. SGFPTs - TRIPPED a. Manually TRIP SGFPTs. **.** b. SU SGFP - TRIPPED b. Manually TRIP SU SGFP. c. VERIFY the following valves c. Manually CLOSE valves. - CLOSED ___ o FWIVs ____ o FWIBs ____ o FW preheater bypass valves ____ o FW regulating valves ____ o Low power FW regulating valves ___ o SG blowdown isolation valves ____ o SG sample isolation valves

...

REACTOR TRIP OR SAFETY INJECTION

REV. 19

PAGE 2 OF 8

	ADDENDUM 5 VERIFICATION OF SI EQUIPMENT OPERATION		
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
	CHECK if main steamline should be isolated:		
8	a. CHECK for any of the following conditions:	a. GO TO Step 3 of this Addendum.	
	o Containment pressure - GREATER THAN OR EQUAL TO 3 PSIG		
	OR		
	o SG pressure (without low steamline pressure SI blocked) - LESS THAN OR EQUAL TO 735 PSIG		
	OR		
	o SG pressure (with low steamline pressure SI blocked) - LOWERING AT A RATE GREATER THAN OR EQUAL TO 100 PSI/SEC.BY OBSERVANCE OF THE STEAMLINE PRESSURE RATE BISTABLES		
1	b. VERIFY main steamline isolation:	b. Manually CLOSE valves.	
	o MSIVs - CLOSED		
	o MSIBs - CLOSED		

REACTOR TRIP OR SAFETY INJECTION

PAGE 3 OF 8

REV. 19

		PAGE 3 OF 8
	<u>ADDENDU</u> VERIFICATION OF SI EQ	
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
	CAUTIO	N
Load	ment should NOT be manually loaded on an Sequencer has completed its automatic se espective ESF Load Sequencer has failed	quence OR it has been determined that
3	VERIFY AFW system status:	
	a. Motor-driven pump – RUNNING	a. <u>WHEN</u> the respective ESF Load Sequencer has completed its automatic sequence <u>OR</u> it is determined that the respective ESF Load Sequencer has failed, <u>THEN</u> manually START pump(s).
	b. Turbine-driven pump - RUNNING	b. Manually OPEN steam supply valves.
4	VERIFY AFW valve alignment – PROPER EMERGENCY ALIGNMENT	Manually ALIGN valves.
5)	(VERIFY total AFW Flow) -) (GREATER THAN) . 576 (GPM)	PERFORM the following:) (a. Manually START pumps <u>AND</u> ALIGN (valves to feed SGs.)
		(b. CONTROL AFW flow to maintain NR) (level BETWEEN 14%)[[34%]](and 50%.)

1

.

- -

REACTOR TRIP OR SAFETY INJECTION

REV. 19

۰,

ADDEND VERIFICATION OF SI E CCTED RESPONSE	
CTED RESPONSE	
	RESPONSE NOT OBTAINED
ment isolation phase A:	
CTUATED	a. Manually ACTUATE phase A.
1, PHASE A ISOLATION	b. Manually CLOSE valves.
	<u>WHEN</u> the respective ESF Load Sequence has completed its automatic sequence
	OR it is determined that the
charge isolation	respective ESF Load Sequencer has failed. <u>THEN</u> PERFORM the following:
IN	a. Manually START pump(s).
	b. Manually OPEN discharge isolation valve(s).
	c. <u>IF</u> any ECW pump can <u>NOT</u> be started <u>OR</u> its discharge isolation valve c <u>NOT</u> be opened, <u>THEN</u> :
	1) TRIP associated STBY DG, <u>AND</u> PLACE in PULL-TO-STOP.
	2) TRIP associated Essential Chiller(s).
pន - RUNNING	<u>WHEN</u> the respective ESF Load Sequence has completed its automatic sequence <u>OR</u> it is determined that the respective ESF Load Sequencer has failed, <u>THEN</u> manually START pump(s).
	ment isolation phase A: ACTUATED Aves - CLOSED, REFER (1, PHASE A ISOLATION N atus: RUNNING charge isolation N

.

REACTOR TRIP OR SAFETY INJECTION

PAGE 5 OF 8

		PAGE 5 OF 8
	ADDENDI VERIFICATION OF SI E	
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
9	VERIFY RCFC status:	
	a. RCFCs - RUNNING	a. <u>WHEN</u> the respective ESF Load Sequencer has completed its automatic sequence <u>OR</u> it is determined that the respective ESF Load Sequencer has failed, <u>THEN</u> manually START RCFC(s).
	b. Cooling water - TRANSFERRED TO CCW	 b. PERFORM the following: 1) <u>IF</u> RCFC inlet temperatures are LESS THAN OR EQUAL TO 116°F, <u>THEN</u> Manually TRANSFER cooling to CCW.
		2) <u>IF</u> RCFC inlet temperatures are GREATER THAN 116°F. <u>THEN</u> CONTACT the TSC prior to transferring cooling.
10	VERIFY SI pump status:	<u>WHEN</u> the respective ESF Load Sequencer has completed its automatic sequence <u>OR</u> it is determined that the
	o HHSI pumps – RUNNING o LHSI pumps – RUNNING	respective ESF Load Sequencer has failed, <u>THEN</u> manually START pump(s).
11	VERIFY SI valve alignment – PROPER EMERGENCY ALIGNMENT	Manually ALIGN valves.

REV. 19

--- -

-- --

_-- -- ----

REACTOR TRIP OR SAFETY INJECTION

REV. 19

PAGE 6 OF 8

<u>ADDENDUM 5</u> VERIFICATION OF SI EQUIPMENT OPERATION		
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
12	VERIFY SI flow:	
	a. RCS pressure – LESS THAN 1745 PSIG	a. GO TO Step 13 of this Addendum.
	b. HHSI pump flow - INDICATED	b. Manually START pumps <u>AND</u> ALIGN valves.
	c. RCS pressure – LESS THAN 415 PSIG	c. GO TO Step 13 of this Addendum.
	d. LHSI pump flow - INDICATED	d. Manually START pumps <u>AND</u> ALIGN valves.
2.0		
13	VERIFY containment ventilation isolation:	
	a. Containment atmosphere radiation monitor isolation valves - CLOSED	a. Manually CLOSE valves.
	b. Normal purge supply and exhaust fans - STOPPED	b. Manually STOP fans.
	c. Supplemental purge supply and exhaust fans - STOPPED	c. Manually STOP fans.
1. 	d. Purge Dampers – CLOSED	d. Manually CLOSE dampers.

.

........

-

REACTOR TRIP OR SAFETY INJECTION

• • •

PAGE 7 OF 8

REV. 19

		PAGE / OF 8	
ADDENDUM 5 VERIFICATION OF SI EQUIPMENT OPERATION			
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
<u> </u>	VERIFY ventilation actuation:		
⁸	a. Control room HVAC - OPERATING IN EMERGENCY RECIRC	a. Manually PLACE control room HVAC in EMERGENCY RECIRC.	
ł	D. EAB HVAC - OPERATING IN EMERGENCY RECIRC	b. Manually PLACE EAB HVAC in EMERGENCY RECIRC.	
0	E FHB HVAC - OPERATING IN EMERGENCY MODE	c. Manually PLACE FHB HVAC in EMERGENCY MODE.	
c	1. FHB Exhaust Fans – ONLY TWO TRAINS OPERATING	d. PERFORM the following:	
	o Exhaust booster fans	 <u>IF</u> three trains FHB exhaust fans running, <u>THEN</u> PLACE one train FHB exhaust fans in PULL TO LOCK. 	
	o Main exhaust fans	2) <u>IF</u> LESS THAN two trains FHB exhaust fans running, <u>THEN</u> manually START train(s).	
		o Exhaust booster fans o Main exhaust fans	
e	e. SECURE one FHB filter train by PERFORMING the following:		
	 PLACE the outlet damper Controller in manual Manually close the outlet damper VERIFY proper operation of filter train in service 	• ·	
f	. Essential chilled water pumps - RUNNING	f. PERFORM the following:1) Manually START essential chilled	
Step 14 c	continued next page.	water pumps. 2) <u>IF</u> an Essential Chilled Water Train fails to start, <u>THEN</u> SECURE the corresponding train of EAB HVAC.	

,

.....

•••••

REACTOR TRIP OR SAFETY INJECTION

--- . .

_ . . _ . . .

REV. 19

PAGE 8 OF 8

ADDENDUM 5 VERIFICATION OF SI EQUIPMENT OPERATION		
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
Step 14	continued from previous page.	
8	g. Essential chillers – RUNNING	g. PERFORM the following:
		 <u>WHEN</u> the respective ESF Load Sequencer has completed its automatic sequence <u>OR</u> it is determined that the respective ESF Load Sequencer has failed. <u>THEN</u> manually START essential chiller(s).
		2) <u>IF</u> an Essential Chiller fails to start, <u>THEN</u> SECURE the corresponding train of EAB HVAC.
	h. ECCS pump room fan coolers – RUNNING	h. Manually START ECCS pump room fan coolers.
	i. AFW pump cubicle fans – RUNNING	i. Manually START AFW pump cubicle fans.
	j. FHB truck bay doors – CLOSED	j. CONTACT designated personnel to close the FHB truck bay doors.
15	NOTIFY Unit Supervisor that Addendum 5 is COMPLETE	
16	IMPLEMENT Functional Restoration Procedures as required	

____ 17 RETURN TO procedure step in effect.

REACTOR TRIP OR SAFETY INJECTION

Rev. 19

CONDITIONAL INFORMATION PAGE

Page 1 of 2

RCP TRIP CRITERIA

IF BOTH conditions listed below occur, THEN TRIP ALL RCPs:

- a. HHSI pumps AT LEAST ONE RUNNING
- b. RCS pressure LESS THAN 1430 PSIG

FAULTED SG ISOLATION

<u>IF</u> a faulted SG(s) is <u>NOT</u> required to maintain at least two SGs available for RCS cooldown, THEN the US or SS may direct actions be taken to isolate the faulted SG(s).

MSIV AND MSIB CLOSURE CRITERIA

<u>IF</u> a loss of secondary support systems occurs that impairs the ability of secondary systems to provide a heat sink for the Steam Generators, <u>THEN</u> CLOSE MSIVs and MSIBs. (for example loss of Condenser Availability, C-9)

LHSI PUMP RESTART CRITERIA

<u>IF</u> RCS pressure lowers in an uncontrolled manner to LESS THAN 415 PSIG. <u>THEN</u> START the LHSI pumps to supply water to the RCS.

SEQUENCER LOADING VERIFICATION

<u>IF</u> a LOOP occurs, after the performance of Step 4, <u>THEN</u> PERFORM Addendum 4; Sequencer Loading Verification - Mode III.

AFW FLOW CRITERIA

(IF AFW flow must be reduced to limit RCS cooldown, <u>THEN</u> THROTTLE AFW flow as necessary) while maintaining total AFW flow <u>GREATER</u> THAN 576 GPM until NR level in at least one (SG is <u>GREATER THAN 14% [34%]</u>

AFWST MAKEUP CRITERIA

<u>IF</u> AFWST level lowers to LESS THAN 138,000 GALLONS (26%), <u>THEN</u> INITIATE makeup to the AFWST per OPOPO2-AF-0001, AUXILIARY FEEDWATER, to prevent inventory problems during cooldown.

CONTAINMENT SPRAY / PHASE B ACTUATION

IF a Containment Spray / Phase B actuation occurs. THEN STOP all RCPs.

CONDITIONAL INFORMATION PAGE

Page 2 of 2

(RUPTURED SG ISOLATION) (all bulleted substeps below are part of this CIP action)

<u>IF</u> a ruptured SG(s) is <u>NOT</u> required to maintain at least one SG available for RCS cooldown. <u>THEN</u> the US or SS SHALL direct actions be taken to isolate the ruptured SG(s) as follows:

o SG 1D(2D) ISOLATION

WHEN at least one motor driven AFW pump is available. <u>THEN</u> ISOLATE main steam to AFW pump 14(24):

- 1) RESET SI
- 2) RESET SG LO-LO level AFW actuations
- 3) TRIP turbine-driven AFW pump
- 4) CLOSE turbine-driven AFW pump steam inlet valve (AF-MOV-0143)
- 5) DISPATCH operator to OPEN breaker for AF-MOV-0143, E1D11(E2D11)/5C
- 6) ENSURE turbine-driven AFW pump trip/throttle valve closed (AF-MOV-0514)
- 7) CROSS-CONNECT AFW to 1D(2D) SG until SG NR level is GREATER THAN 14%[34%]

(o AFW ISOLATION CRITERIA)

<u>WHEN</u>(BOTH of the following conditions below are satisfied, <u>THEN</u> ISOLATE AFW flow) (to the ruptured SG(s):

• After ruptured SG(s) NR level is GREATER THAN 14%[34%]

• Before RCS and ruptured SG(s) pressures have been equalized)

o SG PORV ISOLATION CRITERIA

<u>IF</u> the affected SG(s) PORV(s) is open with SG pressure LESS THAN setpoint,<u>THEN</u> manually CLOSE the SG PORV <u>OR</u> dispatch an operator to isolate the SG PORV locally.

O MSIV AND MSIB CLOSURE CRITERIA

<u>IF</u> a loss of secondary support systems occurs that impairs the ability of secondary systems to provide a heat sink for the Steam Generators, <u>THEN</u> CLOSE MSIVs and MSIBs.(for example loss of Condenser Availability, C-9)

RWST CONSERVATION CRITERIA

<u>IF</u> all three CS pumps are injecting. <u>THEN</u> secure one containment spray pump.

Rev. 19

Attachment 3 Reactor Trip Response (0POP05-EO-ES01)

中国和国际



31945060

The steps relevant to AFW flow are identified below. Step 1 on p.2 of 22 Step 3 on p. 4 of 22 Step 8 on p. 15 of 22 Addendum 6 Step 11 on p. 3 of 3 Addendum 7 in general Note that management of SG level is the success criteria for AFW. Flow.

10-21-05 DATE EFFECTIVE

SOUTH TEXAS PROJECT ELECTRIC GENERATING STATION

0P0P05-E0-ES01 Rev. 23

REACTOR TRIP RESPONSE

DEPARTMENT PROCEDURE

SAFETY RELATED (Q)

USAGE CONTROL: In Hand Controlling Station

LIST OF ATTACHMENTS:

Addendum 1. Establishing Normal Letdown
Addendum 2. Establishing Excess Letdown
Addendum 3. Emergency Electrical Loading Requirements
Addendum 4. Saturation Curve
Addendum 5. Sequencer Loading Verification - Mode II
Addendum 6 Placing Main Feedwater in Service
Addendum 7. Placing AFW in Service
Addendum 8. Failing Air to MSIVs and MSIBs
Addendum 9. Establishing Alternate Charging Flow Control
Conditional Information Page

This procedure is applicable in Modes 1, 2, and 3.

REACTOR TRIP RESPONSE

PURPOSE

This procedure provides the necessary instructions to stabilize and control the plant following a Reactor Trip without a Safety Injection.

SYMPTOMS OR ENTRY CONDITIONS

This procedure is entered from OPOP05-EO-EOOO, REACTOR TRIP OR SAFETY INJECTION, Step 4. when SI is neither actuated nor required.

REACTOR TRIP RESPONSE

REV. 23

PAGE 2 OF 22

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE Foldout CIP page should be open. 0 WHEN RCS temperature is controlled by the SG PORVs in AUTO. THEN RCS ο temperature will be maintained at approximately 571°F at the SG PORV setpoint. MONITOR RCS Temperatures PERFORM Substep a. OR Substep b. _1] based on RCS temperature: o WITH ANY RCP RUNNING, RCS TAVG STABLE AT OR TRENDING TO 567°F a IF temperature LESS THAN 567°F) and lowering. THEN: o WITHOUT ANY RCP RUNNING, RCS TCOLD STABLE AT OR TRENDING TO 567°F 1) IF the AFW system is in) service, <u>THEN</u> THROTTLE AFW) flow such that total AFW flow is GREATER THAN 576 GPM until (NR level is GREATER THAN)14% in at least one SG. 2) ISOLATE steam loads as follows: o ISOLATE steam dump drains. o ISOLATE steam chest drains. o ISOLATE steam to MSR and steam line drains to MSRs. o TRIP all SGFPTs. o STOP dumping steam. o ISOLATE SGBD. 3) IF cooldown continues, THEN CLOSE MSIVs and MSIBs. o IF MSIV(s) and MSIB(s) can NOT be closed, THEN DISPATCH operator to close MSIV(s) and MSIB(s) per ADDENDUM 8. FAILING AIR TO MSIVS AND MSIBS. Step 1 continued on next page.

·		REACTOR 1	RIP RESPONSE	REV. 23
				PAGE 3 OF 22
TEP	ACTION/EXPECTED RES	SPUNSE	RESPONSE NOT OBTAINE	D
tep 1	continued from previous	page.		
			4) <u>IF</u> temperatur <u>AND</u> lowering,	ce LESS THAN 550°F . <u>Then</u> :
			a) INITIATE e	emergency boration
			to GREATER RCS Cb GRE	cemperature rises & THAN 550°F <u>OR</u> EATER THAN 2800 SECURE emergency
			b. <u>IF</u> temperature G <u>AND</u> rising, <u>THEN</u>	
			1) <u>IF</u> condenser DUMP steam to	
			2) <u>IF</u> condenser <u>THEN</u> ENSURE S controlling t AUTO.	
			a) VERIFY SG 1225 psig.	PORV setpoints at
			b) <u>IF</u> SG PORV controllin AUTO. <u>THEN</u>	ng temperature in
				affected SG in MANUAL.
				RCS temperature AT <u>OR</u> TRENDING TO
				· · · · · · · · · · · · · · · · · · ·

......

_ -

020205	-E0-ES01 REA	ACTOR TRIP RESPONSE	REV. 23
			PAGE 4 OF 22
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
2	CHECK FW Status:		
	_a. CHECK RCS TAVG - LESS THAI	v 574°F a. PERFORM the follo	wing:
		1) <u>WHEN</u> RCS TAVG 574°F. <u>THEN</u> PE and 2.c.	is LESS THAN RFORM Steps 2.b
		2) GO TO Step 3	
	b. VERIFY FW isolation:	b. Manually CLOSE va	lves.
	o FWIVs - CLOSED	÷.	
	o FWIBs - CLOSED		
	o FW preheater bypass valv CLOSED	ves -	
	o FW regulating valves -Cl	LOSED	
	o Low Power FW regulating -CLOSED	valves	
		•••••	
	_c. TRIP all SGFPTs		
3	VERIFY Feedwater Flow Estable GREATER THAN OR EQUAL TO three SGs:	ished. ToESTABLISH feedwateree (3)THAN OR EQUAL TO thr	
	o Main Feedwater flow – REFE ADDENDUM 6, PLACING MAIN FE IN SERVICE		
	OR	OR	
	OK O AFW flow)(-)(REFER TO ADDENDU (PLACING AFW IN SERVICE)	o AFW flow - REFER T JM 7) PLACING AFW IN SER	
	OR		
	OAFW flow on SG LO-LO level (actuation)	AFW	

· ----

-- -

. . .

-. -. .- .

0P0P05-E0-ES01		REACTOR TRIP	RESPONSE PAGE 5 OF 22
STEP	ACTION/EXPECTED RESPONS	E	RESPONSE NOT OBTAINED
4	VERIFY Control Rods Fully]	Inserted	PERFORM the following:
	o All rod bottom lights – I	LIT	a. <u>IF</u> two <u>OR</u> more control rod bottom lights <u>NOT</u> lit, <u>THEN</u> :
			 Emergency BORATE 940 GALLONS of boric acid (60 ppm) for each control rod 18 steps OR LESS.
			X 940 gals = gals # of rods
			2) Emergency BORATE 3600 GALLONS of boric acid (228 ppm) for each control rod GREATER THAN 18 steps.
			$\frac{1}{\# \text{ of rods}} X 3600 \text{ gals} = \frac{1}{2} \text{ gals}$
			<u>OR</u>
			3) Emergency BORATE until RCS Cb GREATER THAN 2800 ppm.
			b. <u>IF</u> DRPI has failed, <u>THEN</u> :
			1) INITIATE emergency boration.
			2) <u>WHEN</u> DRPI has been restored <u>OR</u> RCS Cb GREATER THAN 2800 ppm, <u>THEN</u> SECURE emergency boration.

1

....

. ...

-- ----

.

0P0P05-E0-ES01	REACTOR TRIP RESPONSE	REV. 23
	KERCIOK IKIT KESIONSE	PAGE 6 OF 22
STEP ACTION/EXPECTED RESP	PONSE RESPONSE NOT OBTA	INED
5 CHECK STBY DG Status:	· · · ·	
a. STBY DG - RUNNING	a. GO TO Step 6.	
	••••••••••	
b. VERIFY ECW to STBY D	G b. PERFORM the f	following:
	1) Manually S pump.	START applicable ECW
	2) Manually C isolation	DPEN discharge valve.
• ,	to a runni	n <u>NOT</u> be established Ing STBY DG, <u>THEN</u> affected STBY DG.
	to Essenti	n <u>NOT</u> be established al Chillers, <u>THEN</u> associated Essential

.

· ---

.

...

___ __ __ __

REACTOR TRIP RESPONSE

REV. 23

PAGE 7 OF 22

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION

The Charging Pump should <u>NOT</u> be manually loaded on an ESF bus until the respective Load Sequencer has completed its automatic sequence <u>OR</u> it has been determined that the respective Load Sequencer has failed to operate.

____6 CHECK Pressurizer Level Control:

- ____a. Pressurizer level ~ GREATER THAN 17%
- a. PERFORM the following:
 - 1) ENSURE normal letdown isolation.
 - 2) ENSURE excess letdown isolation.
 - 3) ENSURE pressurizer heaters off.
 - CONTROL charging to restore pressurizer level GREATER THAN 17%.
 - 5) <u>WHEN</u> pressurizer level is GREATER THAN 17%, <u>THEN</u>:
 - a) ENSURE ESF load sequencers reset.
 - b) TURN ON pressurizer heaters as necessary.
 - c) ESTABLISH normal letdown per ADDENDUM 1, ESTABLISHING NORMAL LETDOWN.
 - d) <u>IF</u> normal letdown can <u>NOT</u> be established, <u>THEN</u> ESTABLISH excess letdown per ADDENDUM 2, ESTABLISHING EXCESS LETDOWN.

Step 6 continued on next page.

POP05-E0-ES01	REACTOR TRIP RESPONSE		REV. 23	
LOLO2 - FO - FOOT	REACTOR 1		PAGE 8 OF 22	
TEP ACTION/EXPECTED	RESPONSE	RESPONSE NOT OBTAINED		
tep 6 continued from previ	ous page.			
b. VERIFY charging	- IN SERVICE	b. ESTABLISH charging flow	·:	
		 <u>IF</u> a charging pump i available, <u>THEN</u> GO T step 6.c.3 (RNO) of procedure. 	0	
		2) <u>IF</u> a charging pump i running, <u>THEN</u> :	s <u>NOT</u>	
		a) CLOSE all seal in isolation valves.		
		b) CLOSE the chargin discharge isolati for the pump to b	on valve	
		c) CLOSE charging fl valve.	ow control	
		d) <u>IF</u> charging flow valve will <u>NOT</u> cl ESTABLISH chargin ADDENDUM 9, ESTAB ALTERNATE CHARGIN CONTROL and GO TO of this RNO.	ose. <u>THEN</u> g flow per LISHING G FLOW	
		e) OPEN recirculatio for the charging started.		
		f) RESET the ESF loa sequencers.	d	
		g) START a charging	pump.	
		3) OPEN charging pump d isolation valve.	ischarge	
		4) OPEN normal or alter charging isolation v		
		5) OPEN charging OCIV.		
		CONTROL charging flo necessary.	W as	

••

PAGE 9 OF 22 PONSE NOT OBTAINED 7) CLOSE the recirculation valve for the running charging pump. PERFORM the following: 1) <u>IF</u> seal injection flow is established. <u>THEN</u> ADJUST "FLOW CONT" HCV-0218 to return RCP
 7) CLOSE the recirculation valve for the running charging pump. PERFORM the following: 1) <u>IF</u> seal injection flow is established. <u>THEN</u> ADJUST "FLOW CONT" HCV-0218 to return RCP
 for the running charging pump. PERFORM the following: 1) <u>IF</u> seal injection flow is established. <u>THEN</u> ADJUST "FLOW CONT" HCV-0218 to return RCP
 for the running charging pump. PERFORM the following: 1) <u>IF</u> seal injection flow is established. <u>THEN</u> ADJUST "FLOW CONT" HCV-0218 to return RCP
1) <u>IF</u> seal injection flow is established. <u>THEN</u> ADJUST "FLOW CONT" HCV-0218 to return RCP
established. <u>THEN</u> ADJUST "FLOW CONT" HCV-0218 to return RCP
seal injection flow to BETWEEN 6 AND 13 GPM.
 <u>IF</u> seal injection flow is <u>NOT</u> established, <u>THEN</u>:
a) PERFORM the following:
 CHECK seal water inlet and lower seal water bearing temperatures LESS THAN 230°F for each RCP. (Plant computer RC-010)
2) <u>IF</u> seal water inlet OR lower seal water bearing temperatures GREATER THAN OR EQUAL TO 230°F, <u>THEN</u> DO <u>NOT</u> establish seal injection to that RCP. GO TO Step 6.d.
b) CLOSE "FLOW CONT" HCV-0218.
c) ENSURE seal injection isolation valves open for applicable RCPs.
d) MONITOR seal water inlet

.....

Step 6 continued on next page.

--- -

_ _ _ _

0P0P05-E0-ES01	REACTOR 1	RIP RESPONSE	REV. 23	
		PAGE 10) OF 2	
STEP ACTION/EXPECTED	RESPONSE	RESPONSE NOT OBTAINED		
Step 6 continued from previ	ous page.			
		e) Slowly OPEN "FLOW CONT" HCV-0218 to establish a 1°F per minute cooldown rate on seal water inle and lower seal water bearing temperatures.	L	
		f) ESTABLISH normal seal injection flow of betwe 6 and 13 gpm.	en	
		3) <u>IF</u> a charging pump is not running, <u>THEN</u> :		
		a) PERFORM the following:		
		 CHECK seal water inland lower seal water bearing temperatures LESS THAN 230°F for RCP. (Plant computer RC-010) 	each	
		2) <u>IF</u> seal water inlet lower seal water bea temperatures GREATER THAN OR EQUAL TO 230 <u>THEN</u> DO <u>NOT</u> establis seal injection to th RCP. GO TO Step 6.0	ering PF, h at	
		b) ENSURE seal injection isolation valves open f applicable RCPs.	or	
		c) ENSURE PDP recirculation valve is 100% open.	'n	
		d) START the PDP.		
		e) MONITOR seal water inle and lower seal water bearing temperatures. (Plant computer display RC-010)		

0P0P05-E0-E	S01	REACTOR TRIP RESPONSE		REV. 23 PAGE 11 OF 23
STEP	ACTION/EXPECTED RESPON	SE	RESPONSE NOT OBTAINE	
Step 6 cont	inued from previous pag	ge.		
			f) Slowly CLOSE the PDP recirculation value to establish a 1°F/min. cooldown rate on seal wat inlet and lower seal wate bearing temperatures.	
			g) ESTABLISH injection 6 and 13 g	flow of between
d.	VERIFY letdown – IN SE	RVICE	d. PERFORM the foll	owing:
·			1) ESTABLISH let	down:
			per ADDEND	normal letdown UM 1. NG NORMAL LETDOWN.
			be establi ESTABLISH per ADDEND	letdown can <u>NOT</u> shed. <u>THEN</u> excess letdown UM 2. NG EXCESS LETDOWN.
			2) <u>IF</u> letdown es GO TO Step 6.	
			emergency bor	<u>T</u> established <u>AND</u> ation <u>NOT</u> in <u>N</u> PERFORM the
			a) ISOLATE ch	arging.
				l injection flow mately 6 gpm.
			.c) GO TO Step	7.
				····

r

.........

- - -- -

Step 6 continued on next page.

0P0P05 -	EO-ESO1	REACTOR TRIP	RESPONSE REV. 23
STEP	ACTION/EXPECTED RES	PONSE	PAGE 12 OF 22 RESPONSE NOT OBTAINED
-	continued from previous _e. Pressurizer level to		e. CONTROL charging and letdown to maintain level at 25%.
7	MONITOR Pressurizer Pro _a. Pressure - GREATER 1		
	_a. riessule - GALAILK I	IAN 1037 7316	 a. PERFORM the following: 1) ENSURE SI actuated. 2) GO TO 0POP05-E0-E000. REACTOR TRIP OR SAFETY INJECTION. Step 1.
	b. Pressure - STABLE AT TO 2235 PSIG	F OR TRENDING	 b. PERFORM the following: 1) <u>IF</u> pressure LESS THAN 2235 PSIG <u>AND</u> lowering, <u>THEN</u>: a) ENSURE pressurizer PORVs closed. b) <u>IF</u> any PORV can <u>NOT</u> be closed, <u>THEN</u> manually CLOSE its isolation valve <u>AND</u> REFER TO TECH SPECS 3.4.4. c) ENSURE pressurizer spray valves closed.

•

	0 7000			
0P0P05-E	.0-8201	REACTOR J	RIP RESPONSE	PAGE 13 OF 22
STEP	ACTION/EXPECTE	D RESPONSE	RESPONSE NOT OBTA	INED
Step 7 c	continued from prev	vious page.		
			valve(s	surizer spray) can <u>NOT</u> be closed. RFORM the following:
			1) STOP	RCP 1A(2A).
			2) STOP	RCP 1D(2D).
			cont	CS pressure inues to lower, <u>THEN</u> all but one RCP.
			cont	CS pressure still finues to lower, <u>THEN</u> ORM the following:
				OLATE IA to ntainment.
			in AD ES	ACE Excess Letdown service per DENDUM 2. TABLISHING EXCESS TDOWN.
			e) ENSURE reset.	ESF load sequencers
			f) ENSURE on.	pressurizer heaters

-

REV. 23

Step 7 continued on next page.

DODOF FO	RC01 DI	EACTOR TRIP RESPONSE	REV. 23
0P0P05-E0-ES01			PAGE 14 OF 22
STEP	ACTION/EXPECTED RESPONSE	RESPONSE	NOT OBTAINED
Step 7 con	tinued from previous page	•	
			pressure GREATER THAN 35 PSIG <u>AND</u> rising. <u>THEN</u> :
		a)	ENSURE pressurizer heaters off.
		ь)	CONTROL pressure using normal pressurizer spray.
		с)	<u>IF</u> normal spray <u>NOT</u> available, <u>THEN</u> :
	Α.		 ENSURE normal spray valves are closed.
			2) USE auxiliary spray.
		d)	<u>IF</u> auxiliary spray <u>NOT</u> available. <u>THEN</u> USE one pressurizer PORV.
			•
			,
			ι.

......

.

0P0P05-E	CO-ESO1 REACTOR TRIE	REV. 23 RESPONSE PAGE 15 OF 22
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
8	MONITOR SG Levels:	
	a. (NR) (level) () GREATER THAN 14%)	 a. PERFORM the following: (1) IF NR level in all SGs LESS) (THAN 14%, <u>THEN</u> MAINTAIN total (AFW) flow GREATER THAN 576 GPM.) (2) WHEN NR level in at least one (SG is GREATER THAN 14%, <u>THEN</u>) (CONTROL AFW) flow to maintain (NR level (BETWEEN) 22% and 50%.) (3) IF any AFW pump fails to (start, <u>THEN</u>) (a) RESET all SG LO-LO level (AFW) (actuations.) (b) CLOSE (applicable) (AFW) (regulating valve.) (c) OPEN (applicable) (AFW) (regulating valve.) (d) CONTROL AFW) flow to (LESS) (THAN 675 GPM (per (AFW) (pump)) (4) GO TO Step 8.c)
	b. CHECK AFW system - IN SERVICE o AFW pump(s) - RUNNING	b. GO TO Step 9.
	c. CONTROL AFW)flow to maintain)NR levels BETWEEN 22% and 50%	c. <u>IF</u> NR/level in any SG continues to rise. <u>THEN</u> STOP AFW flow to that SG.
	d. <u>IF</u> Main Feedwater is available, <u>THEN</u> PLACE Main Feedwater in service per ADDENDUM 6, PLACING MAIN FEEDWATER IN SERVICE	

.....

000005	-EO-ESO1 REACTOR TRIP	REV. 23
020202	-EO-ESO1 REACTOR TRIP	PAGE 16 OF 22
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
9	VERIFY A11 13.8KV And 4.16KV AC Busses - ENERGIZED BY OFFSITE POWER	 PERFORM the following: a. TRY to restore offsite power per OPOP04-AE-0001, FIRST RESPONSE TO LOSS OF ANY OR ALL 13.8KV OR 4.16KV BUS. b. <u>IF</u> offsite power can <u>NOT</u> be restored, <u>THEN</u> manually LOAD desired equipment on the AC ESF busses, REFER TO ADDENDUM 3, EMERGENCY ELECTRICAL LOADING REQUIREMENTS.
10	ENSURE All Dilution Paths Are Isolated:	
	a. BTRS - BYPASSED	a. BYPASS BTRS.
-	b. Letdown Demineralizer – BYPASSED	b. BYPASS letdown demineralizer.
_	c. RMW pumps - PULL TO LOCK	c. PLACE RMW pumps in PULL TO LOCK.
_	d. RMW nonessential header isolation valves – CLOSED	d. CLOSE at least one RMW nonessential header isolation valve.

.

•

· · _

1

___ · ·

0	PC)P	0	5	- EO	-ES	0	1	
---	----	----	---	---	------	-----	---	---	--

REACTOR TRIP RESPONSE

REV. 23

PAGE 17 OF 22

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

Deaerator Level.

5

NOTE Approximately three (3) turns open on each Deaerator High Level Dump Bypass Valve is a good starting point to maintain Deaerator level. __11 MAINTAIN Deserator Level: ____a. DISPATCH Operator To Throttle Open Deaerator High Level Dump Bypass Valves To Maintain Deaerator Level (29 ft TGB S of CNDSR #13/23) o "1(2)-FW-0486" "DEAERATOR STORAGE TANK #2" "HIGH LEVEL DUMP BYPASS VALVE" o "1(2)-FW-0487" "DEAERATOR STORAGE TANK #1" "HIGH LEVEL DUMP BYPASS VALVE" ___b. VERIFY Deaerator Level Control b. Manually CONTROL Deaerator Level Valve - MAINTAINING LEVEL Control Valve to maintain

._.

POP05-E0-ES01 REA	REV. 23 CTOR TRIP RESPONSE
	PAGE 18 OF 22
TEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
12 CHECK MSIV Status:	
a. Condenser – AVAILABLE	a. PERFORM the following:
	1) ENSURE MSIVs and MSIBs - CLOSED.
	o <u>IF</u> MSIV(s) and MSIB(s) can <u>NOT</u> be closed. <u>THEN</u> DISPATCH operator to close MSIV(s) and MSIB(s) per ADDENDUM 8. FAILING AIR TO MSIVS AND MSIBS.
	 USE SG PORV(s) to maintain RCS temperature:
	o WITHOUT ANY RCP running, RCS TCOLD - STABLE. o WITH ANY RCP running, RCS TAVG - STABLE.
:	3) GO TO Step 14. OBSERVE note prior to Step 14.
b. MSIVs - OPEN	b. <u>IF</u> RCS TAVG is controlled <u>OR</u> rising, <u>THEN</u> :
	1) ENSURE MSIV and MSIB handswitches in CLOSED position.
	2) ENSURE Main Steam Isolation reset.
	3) Slowly OPEN MSIBs to pressurize the main steam header.
	4) <u>WHEN</u> the differential pressure across the MSIVs is LESS THAN 50 PSIG, <u>THEN</u> OPEN the MSIVs.
	5) <u>IF</u> RCS TAVG lowers in an uncontrolled manner, <u>THEN</u> CLOSE the MSIVs and MSIBs.
	

-

.....

.

REACTOR TRIP RESPONSE

REV. 23

PAGE 19 OF 22

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

The steam dump pressure controller setting to maintain 1185 psig is approximately 8.46 turns.

____13 TRANSFER Condenser Steam Dump To Pressure Control Mode:

- ____a. PLACE the steam dump controller in MANUAL and at minimum demand
- ____b. TRANSFER steam dump to pressure control mode
- _____c. ENSURE Steam dump pressure controller – SET TO MAINTAIN 1185 PSIG
- c. ADJUST steam dump controller.

___d. PLACE the steam dump controller in AUTO

<u>NOTE</u>

---- -

REACTOR TRIP RESPONSE

REV. 23

PAGE 20 OF 22

STEP

•ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

STAINED

NOTE The RCP preferred running order to provide normal pressurizer spray is as follows: o First - LOOP D o Second - LOOP A o Third - LOOPs B OR C _14 CHECK RCP Status - AT LEAST ONE PERFORM the following: RUNNING a. START one RCP per 0P0P02-RC-0004, OPERATION OF REACTOR COOLANT PUMP. b. IF an RCP can NOT be started, THEN VERIFY natural circulation. o RCS subcooling based on core exit T/Cs - GREATER THAN 35°F o SG pressures - STABLE OR LOWERING. o RCS hot leg temperatures -STABLE OR LOWERING. o Core exit T/Cs - STABLE OR LOWERING. o RCS cold leg temperatures - AT SATURATION TEMPERATURE FOR SG PRESSURE, REFER TO ADDENDUM 4. SATURATION CURVE. c. IF natural circulation can NOT be verified, THEN RAISE steam dumping rate. -----

0P0P05-E0-ES02	0	PC	PO	15 -	- EO -	ESO	1
----------------	---	----	----	------	--------	-----	---

REACTOR TRIP RESPONSE

.....

PAGE 21 OF 22

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
15	CHECK If SR Detectors Should Be Energized:	
	_a. CHECK IR flux – LESS THAN 10–10 AMPS	 a. PERFORM the following: 1) <u>WHEN</u> IR flux LESS THAN 10⁻¹⁰ AMPS. <u>THEN</u> PERFORM Steps 15.b and 15.c. 2) GO TO Step 16.
_	_b. VERIFY SR detectors - ENERGIZED	b. Manually ENERGIZE SR detectors.
16 17 18	 c. TRANSFER NR-45: o Highest SR channel o Highest IR channel SECURE Unnecessary Plant Equipment. REFER TO OPOP04-ZO-0003. SECONDARY PLANT STABILIZATION PERFORM OPSP10-ZG-0003. SHUTDOWN MARGIN VERIFICATION MODES 3. 4. AND 5 MAINTAIN Stable Plant Conditions: o Pressurizer pressure - BETWEEN 2220 PSIG AND 2250 PSIG o Pressurizer level - BETWEEN 23% AND 33% o SG NR levels BETWEEN 22% and 50% o RCS Temperature - o WITH ANY RCP RUNNING. RCS TAVG BETWEEN 565°F AND 569°F OR o WITHOUT ANY RCP RUNNING. RCS TCOLD BETWEEN 565°F AND 569°F 	

000005		REV. 23
010102	-EO-ESO1 REACTOR TRIE	PAGE 22 OF 22
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
19	DETERNINE If Natural Circulation Cooldown Is Required:	
	a. CHECK RCPs – ALL OFF	a. GO TO Step 20.
	b. Letdown – AVAILABLE	b. GO TO OPOPO5-EO-ESO5, NATURAL CIRCULATION COOLDOWN WITHOUT LETDOWN, Step 1.
_	_c. GO TO OPOPO5-EO-ESO2, NATURAL CIRCULATION COOLDOWN, Step 1	
20	PERFORM OPOPO1-ZQ-0022, PLANT OPERATIONS SHIFT ROUTINES, SAFETY FUNCTION CHECKLIST, For The Applicable Mode	
21	VERIFY Compliance With Technical Specifications Limiting Conditions For Operation	 PERFORM The following: a. CONTINUE monitoring of Critical Safety Function Status Trees. b. CONTINUE monitoring the Conditional Information Page. c. ATTEMPT to restore plant conditions to comply with Technical Specifications. d. CONSULT Plant Management. e. DO NOT exit this procedure until directed by Plant Management OR compliance with Technical Specification Limiting Conditions For Operation are achieved.
22	GO TO Appropriate Plant Procedure, As Determined By Unit Or Shift Supervisor	
	- EN	т D -

•

.

.....

..._ __

.......

-- . . .

REACTOR TRIP RESPONSE

.

PAGE 1 OF 7

REV. 23

·----

,

.

<u>ADDENDUM 1</u> ESTABLISHING NORMAL LETDOWN				
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED		
1	VERIFY charging - IN SERVICE	RETURN TO procedure step in effect.		
2	VERIFY at least one train of CCW – ALIGNED TO THE LETDOWN HEAT EXCHANGER	PERFORM the following: a. START CCW pump(s) and ALIGN valves to establish CCW flow to letdown heat exchanger.		
		b. <u>IF</u> CCW flow can NOT be established to letdown heat exchanger, <u>THEN</u> RETURN TO procedure step in effect.		
3	CLOSE RCS letdown orifice isolation valves	RETURN TO procedure step in effect.		

REACTOR TRIP RESPONSE

. .

REV. 23

PAGE 2 OF 7

•

	ESTABLISHING N	IDUM 1 IORMAL LETDOWN		
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED		
4	VERIFY letdown pressure – WITHIN 100 psi OF RCS PRESSURE	PERFORM the following: a. ENSURE CCW is in service and		
	(35 ft EAB Relay Room North Wall. Panel ZLC505)	aligned to supply cooling to the excess letdown heat exchanger.		
	"1(2)-CV-PI-0469" "EXCESS/NORMAL LETDN PRESSURE"	b. OPEN excess letdown loop isolation valves.		
		c. OPEN regenerative heat exchanger excess letdown heat exchanger crosstie valve.		
		(35 ft EAB Relay Room North Wall, panel ZLC505)		
		"1(2)-CV-HS-0469 EXCESS/NORMAL" "LETDN CROSS CONNECTION"		
		d. VERIFY pressure on "CV-PI-0469 EXCESS/NORMAL LETDN PRESSURE" WITHIN 100 PSI of RCS pressure.		
		e. <u>IF</u> letdown pressure is <u>NOT</u> WITHIN 100 psi of RCS pressure, <u>THEN</u> PERFORM the following:		
		 CLOSE regenerative heat exchang excess letdown heat exchanger crosstie valve. 		
		(35 ft EAB Relay Room North Wall. panel ZLC505		
		"1(2)-CV-HS-0469 EXCESS/NORMAL" "LETDN CROSS CONNECTION"		
		 CLOSE excess letdown loop isolation valves. 		
		 RETURN TO procedure step in effect. 		

REACTOR TRIP RESPONSE

REV. 23

.

PAGE 3 OF 7

	ADDEN ESTABLISHING N	
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
5	PLACE letdown pressure controller in MANUAL	
6	THROTTLE letdown pressure control valve 1(2)-PCV-0135 to 50%	
7	OPEN letdown line containment isolation valves	PERFORM the following:
	o OCIV MOV-0024	a. CLOSE regenerative heat exchanger excess letdown heat exchanger crosstie valve.
	o ICIV MOV-0023	(35 ft EAB Relay Room North Wall, panel ZLC505)
		"1(2)-CV-HS-0469 EXCESS/NORMAL" "LETDN CROSS CONNECTION"
		b. RETURN TO procedure step in effect
8	OPEN letdown line loop isolation valves	PERFORM the following:
	o LETDN ISOL LCV-0465	a. CLOSE letdown line containment isolation valves.
	o LETDN ISOL LCV-0468	b. CLOSE regenerative heat exchanger excess letdown heat exchanger crosstie valve.
		(35 ft EAB Relay Room North Wall, panel ZLC505)
		"1(2)-CV-HS-0469 EXCESS/NORMAL" "LETDN CROSS CONNECTION"
		c. RETURN TO procedure step in effect

REACTOR TRIP RESPONSE

REV. 23

PAGE 4 OF 7

	ADDENDUM 1 ESTABLISHING NORMAL LETDOWN		
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
9	OPEN letdown orifice header isolation valve o LTDN ORIF HDR ISOL FV-0011	 PERFORM the following: a. CLOSE letdown line loop isolation valves. b. CLOSE letdown line containment isolation valves. c. CLOSE regenerative heat exchanger excess letdown heat exchanger crosstie valve. (35 ft EAB Relay Room North Wall. panel ZLC505) *1(2)-CV-HS-0469 EXCESS/NORMAL* *LETDN CROSS CONNECTION* d. RETURN TO procedure step in effect 	

· ····

REACTOR TRIP RESPONSE

REV. 23

PAGE 5 OF 7

ADDI	ENDUM 1	
ESTABLISHING	NORMAL	LETDOWN

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION

Failure to perform Steps 10 and 11 in rapid sequence may cause excessive letdown line fluid flashing or lifting of letdown relief to PRT.

____ 10 OPEN desired letdown line orifice isolation valve:

.....

- o FV-0012
 - OR
- o EV-0013
 - OR
- o MOV-0014

- PERFORM the following:
- a. CLOSE letdown orifice header isolation valve.
- b. CLOSE letdown line loop isolation valves.
- c. CLOSE letdown line containment isolation valves.
- d. CLOSE regenerative heat exchanger excess letdown heat exchanger crosstie valve.

(35 ft EAB Relay Room North Wall. panel ZLC505)

"1(2)-CV-HS-0469 EXCESS/NORMAL" "LETDN CROSS CONNECTION"

e. RETURN TO procedure step in effect.

-- ...

REACTOR TRIP RESPONSE

PAGE 6 OF 7

REV. 23

.....

.

<u>ADDENDUM 1</u> ESTABLISHING NORMAL LETDOWN		
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
11	THROTTLE letdown pressure controller to maintain 375 to 385 psig	<pre>PERFORM the following: a. CLOSE applicable letdown line orifice isolation valve(s): o FV-0012 o FV-0013 o MOV-0014 b. CLOSE letdown orifice header isolation valve. c. CLOSE letdown line loop isolation valves. d. CLOSE letdown line containment isolation valves. e. CLOSE regenerative heat exchanger excess letdown heat exchanger crosstie valve. (35 ft EAB Relay Room North Wall. panel ZLCSO5) "1(2)-CV-HS-0469 EXCESS/NORMAL" "LETDN CROSS CONNECTION" f. RETURN TO procedure step in effect.</pre>
12	OPEN additional letdown orifice isolation valves as desired	
13	PLACE letdown pressure controller to AUTO	CONTROL letdown pressure in MANUAL.

. .

•

· ·····

REACTOR TRIP RESPONSE

REV. 23

PAGE 7 OF 7

	<u>ADDENDUM 1</u> ESTABLISHING NORMAL LETDOWN		
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
14	<pre>IF regenerative heat exchanger excess letdown heat exchanger crosstie valve 1(2)-CV-HV-0469 was opened. <u>THEN</u> CLOSE: (35 ft EAB Relay Room North Wall, Panel ZLC505) *1(2)-CV-HS-0469 EXCESS/NORMAL" *LETDN CROSS CONNECTION"</pre>		
15	VERIFY excess letdown loop isolation valves – CLOSED	Manually CLOSE excess letdown loop isolation valves.	

_____ 16 RETURN TO procedure step in effect

•••••

• · · · · · · ·

_...

.

REACTOR TRIP RESPONSE

REV. 23

PAGE 1 OF 3

•

____

ADDENDUM 2 ESTABLISHING EXCESS LETDOWN				
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED		
1	OPEN CCW isolation valves for excess letdown heat exchanger	RETURN TO procedure step in effect.		
2	CHECK seal return containment isolation valves – OPEN	PERFORM the following: a. OPEN seal return containment isolation valves.		
		b. <u>IF</u> seal return containment isolation valves can <u>NOT</u> be opened, <u>THEN</u> :		
·	•	 PLACE excess letdown divert valve to RCDT position. 		
		2) START RCDT pump.		
		3) ENSURE LV-4911 RCDT level control valve in AUTO.		
×		 OPEN RCDT discharge containment isolation valves. 		
		5) DISPATCH operator to open RCDT HX discharge flow orifice bypass.		
		(Valve located 29 ft MAB Rm 1080 operated via reach rod 41 ft MAB Penetration Area)		
		"1(2)-WL-0019" "RCDT HEAT EXCHANGER" "FE-4914 BYPASS VALVE"		
		6) ENSURE adequate capacity in the RHT to receive the discharged coolant.		
	`	7) OPEN excess letdown loop isolation valves.		
		8) GO TO Step 5 of this Addendum and OBSERVE caution prior to Step 5 of this Addendum.		
		X		

REACTOR TRIP RESPONSE

REV. 23

PAGE 2 OF 3

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
3	OPEN excess letdown loop isolation valves	RETURN TO procedure step in effect.	
4	PLACE excess letdown divert valve to VCT position	 ESTABLISH excess letdown flow to RCDT a. PLACE excess letdown divert valve to RCDT position. b. START RCDT pump. c. ENSURE LV-4911 RCDT level control valve in AUTO. d. OPEN RCDT discharge containment isolation valves. e. DISPATCH operator to open RCDT HX discharge flow orifice bypass. 	
		<pre>(Valve located 29 ft MAB Rm 108C, operated via reach rod 41 ft MAB, Penetration Area) "1(2)-WL-0019" "RCDT HEAT EXCHANGER" "FE-4914 BYPASS VALVE" f. ENSURE adequate capacity in the RHT to receive the discharged coolant.</pre>	

.

···· · · ·

REACTOR TRIP RESPONSE

PAGE 3 OF 3

REV. 23

ADDI	ENDUM 2	
ESTABLISHING	EXCESS	LETDOWN

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION

<u>IF</u> excess letdown heat exchanger outlet temperature rises to GREATER THAN 175° <u>OR</u> pressure rises to GREATER THAN 150 PSIG, <u>THEN</u> damage to excess letdown piping and components may occur.

____5 INITIATE excess letdown flow by throttling excess letdown temperature control valve PERFORM the following:

- a. CLOSE excess letdown loop isolation valves.
- b. <u>IF</u> 1(2)-WL-0019 was operated, <u>THEN</u> DISPATCH operator to CLOSE RCDT HX discharge flow orifice bypass valve.

(Valve located 29 ft MAB Rm 108C, operated via reach rod 41 ft MAB, Penetration Area)

"1(2)-WL-0019" "RCDT HEAT EXCHANGER" "FE-4914 BYPASS VALVE"

c. RETURN TO procedure step in effect.

_ 6 RETURN TO procedure step in effect

. .

REACTOR TRIP RESPONSE

PAGE 1 OF 1

REV. 23

Components	Train A (KW)	Train B (KW)	Train C (KW)
Pressurizer Heaters 1A/1B (2A/2B)	431.0	N/A	431.0
Hydrogen Recombiners	N/A	75.0	75.0
RHR Pumps	222.2	222.2	222.2
RMW Pumps	N/A	41.0	41.0
SFPC Pumps	- N/A	140.4	140.4
SFP Air Handling Unit	N/A	0.5	0.5e
Centrifugal Charging Pumps	450.0	N/A	450.0
BA Transfer Pump Room Fans	0.4	N/A	0.4
BAT Pumps	27.0	N/A	27.0
RMW Pumps Air Handling Unit	N/A	1.9	1.9
480V MCCs 1A5/1B5/1C5 (2A5/2B5/2C5)	171.5	72.5	92.2

.

ADDENDUM 3 EMERGENCY ELECTRICAL LOADING REQUIREMENTS

REACTOR TRIP RESPONSE

PAGE 1 OF 1

REV. 23

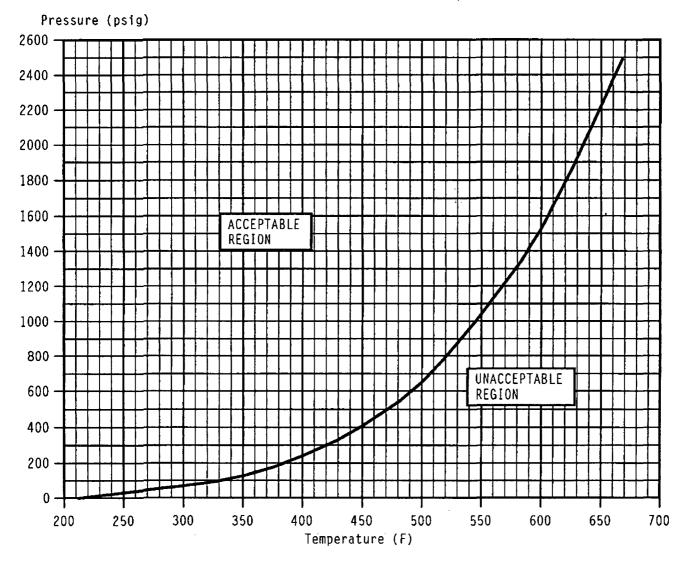
. .

ADDENDUM 4 SATURATION CURVE

STEP

RESPONSE NOT OBTAINED

Saturation Curve



RI

REACTOR TRIP RESPONSE

PAGE 1 OF 2

REV. 23

ADDENDUM 5 SEQUENCER LOADING VERIFICATION - MODE II

STEP

0P0P05-E0-ES01

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION

- o Equipment should NOT be manually loaded on an ESF Bus until the respective ESF Load Sequencer has completed its automatic sequence OR it has been determined that the respective ESF Load Sequencer has failed to operate.
- o <u>IF</u> a LOOP or SI occurs, <u>AND</u> a train of Essential Chilled Water fails, <u>THEN</u> the corresponding EAB HVAC fans should be stopped within 30 minutes to prevent heat buildup in the Auxiliary Shutdown/QDPS rooms.
- 1 VERIFY the following equipment -LOADED ON ESF BUS

Manually LOAD equipment <u>NOT</u> previously secured by the Emergency Operating Procedures.

COMPONENT	TRAIN "A"	TRAIN "B"	TRAIN "C"
480V LC Feeder Breakers			
ECW Pumps			
RCFCs			
CCW Pumps			
AFW Pumps			
Essential Chill Water Pumps			
EAB Supply and Return HVAC Fans			
CRE Supply. Return and C/U HVAC Fans			
FHB Main Exhaust HVAC Fans			
Essential Chillers			

ж. Ц П

•

REACTOR TRIP RESPONSE

PAGE 2 OF 2

REV. 23

<u>ADDENDUM 5</u> SEQUENCER LOADING VERIFICATION - MODE II			MODE II
STEP	ACTION/EXPECTED RESPONSE	RESPONSE	NOT OBTAINED
2	RESET ESF load sequencers		
3	RESET Pressurizer Backup Heaters 1A(2A) and 1B(2B)		
	a. Place Control Room Handswitches in - OFF	I	
	b. RETURN Control Room Handswitches to the position required for current plant conditions		
4	STOP any equipment previously secured by any Emergency Operating Procedure		arth T
5	CHECK ALL Essential Chilled Water Trains - IN SERVICE	fo	CURE the corresponding EAB HVAC r. any Essential Chilled Water ain that is <u>NOT</u> operating.
6	RETURN TO procedure step in effect		······································

•

- - ...

REACTOR TRIP RESPONSE

REV. 23

PAGE 1 OF 3

• _____

	<u>ADDENE</u> Placing Main Feedw	
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
1	VERIFY S/U SGFP - RUNNING	PERFORM the following:
		a. CLOSE S/U SGFP discharge valve.
		b. Manually START S/U SGFP.
		c. OPEN the S/U SGFP discharge valve
		d. <u>IF</u> S/U SGFP can <u>NOT</u> be started <u>THEN</u> ENSURE AFW in service per ADDENDUM 7, PLACING AFW IN SERVICE.
2	VERIFY S/U SGFP discharge valve - OPEN	Manually OPEN S/U SGFP discharge valve.
	÷	*
3	PLACE FW regulating and low power FW regulating valves in manual and minimum demand	
4	PLACE the handswitches for the following valves in the CLOSED position: o FWIVs	· ·
	o FWIBs	
	o Preheater Bypass Valves	
	o SGBD Isolation Valves	
	RESET "LOW TAVG" FW isolation	

REV. 23

0P0P05-E0-ES01

REACTOR TRIP RESPONSE

PAGE 2 OF 3

ADDENDUM 6 PLACING MAIN FEEDWATER IN SERVICE

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

<u>NOTE</u>

The status lights located on the feedwater isolation panel for the "FW CONT/BYP VALVES" may remain lit after the feedwater isolation has been reset.

____6 RESET "FW CONT/BYP VALVES" feedwater isolation

<u>NOTE</u>

To reset a Feedwater Isolation signal following an actual SG HI-HI (P-14) condition, the SI/SG HI-HI pushbuttons must be depressed with the reactor trip breakers closed and the SG HI-HI condition clear.

- ____7 RESET SI/SG HI-HI level feedwater isolation
- ____ 8 RESET all feedwater isolation safety-grade solenoids by momentarily placing handswitches for each train in the OPEN postion

CAUTION

<u>IF</u> any SG levels begin to rise in an uncontrolled manner when the respective preheater bypass valve is opened. <u>THEN</u> it may be necessary to manually isolate the main feedwater regulating valve for that SG(s).

____ 9 OPEN the preheater bypass valves

____ 10 ESTABLISH SG main feedwater flow with the low power FW regulating valves IF SG main FW flow can <u>NOT</u> be established to ALL SGs. <u>THEN</u> USE AFW to maintain level in affected SGs per ADDENDUM 7. PLACING AFW IN SERVICE.

1.

REV. 23

0P0P05-E0-ES01

- -

REACTOR TRIP RESPONSE

--

PAGE 3 OF 3

· - - - - -----

<u>ADDENDUM 6</u> Placing main feedwater in service		
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
(11)	(VERIFY SG NR level in at least one) (SG) (-) (GREATER THAN 14%)	 (PERFORM the following:) (a.) (MAINTAIN total AFW flow) (GREATER THAN)(576)(GPM.) (b.) WHEN SG NR level in at least one) (SG 1s GREATER THAN 14%.THEN) (PERFORM Step 12 of this addendum.) c. RETURN TO procedure step in affect.
<u> </u>	(CHECK AFW) () (IN SERVICE TO ANY SG)	GO TO Step 14 of this addendum.
13	SECURE AFW to SG(s) being supplied by main FW	
	.a. RESET all SG LO-LO level AFW actuations	
	b. CLOSE applicable AFW OCIV(s)	
	c. SECURE applicable AFW pump(s):	
	o Motor-driven AFW pumps - STOP PUMP AND PLACE IN AUTO	· · · ·
	o Steam-driven AFW pump - TRIP TURBINE AND CLOSE TRIP/THROTTLE VALVE	
	d. OPEN applicable AFW flow control valves	
14	CONTROL feedwater flow to maintain SG NR levels BETWEEN 22% and 50%	
15	RETURN TO procedure step in effect	

-

.

REACTOR TRIP RESPONSE

. . .

···· ··· ···· ····

REV. 23

--- --

. ------

PAGE 1 OF 3

····-

(ADDENDUM 7) (PLACING AFW IN SERVICE)

STEP

ACTION/EXPECTED RESPONSE

- ----

RESPONSE NOT OBTAINED

	<u>NOTE</u> Components are located on CP006 unless otherwise noted.		
1	VERIFY AFW to SG isolation valves - CLOSED	CLOSE valves.	
	o AFW to SG 1A(2A) "OCIV MOV-0048"		
	o AFW to SG 1B(2B) "OCIV MOV-0065"		
	o AFW to SG 1C(2C) "OCIV MOV-0085"		
	o AFW to SG 1D(2D) "OCIV MOV-0019"		
2	CLOSE AFW to SG regulating valves(s)	DISPATCH an operator to close valve(s). (25 ft IVC)	
	o AFW to SG 1A(2A) "AFW REG FV-7525"	· · · · · · · · · · · · · · · · · · ·	
	o AFW to SG 1B(2B) "AFW REG FV-7524"		
	o AFW to SG 1C(2C) "AFW REG FV-7523"		
	o AFW to SG 1D(2D) "AFW REG FV-7526"		
	<i>Z</i> 1		
3	<u>IF</u> cross train flow is desired, <u>THEN</u> OPEN the desired AFW Pump cross connect valve(s)	DISPATCH an operator to open the desired valve(s). (25 ft IVC)	
	o AFW Pump 11(21) "XCONN FV-7517"		
	o AFW Pump 12(22) "XCONN FV-7516"		
	o AFW Pump 13(23) "XCONN FV-7515"		
	o AFW Pump 14(24) "XCONN FV-7518"		

•

'n

REACTOR TRIP RESPONSE

REV. 23

PAGE 2 OF 3

ADDENDUM 7 PLACING AFW IN SERVICE

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION

AFW Pump flowrate SHALL NOT exceed 675 GPM per pump.

- ____ 4 START the selected motor-driven AFW pump(s)
 - o "AFW Pump 11(21)"
 - o "AFW Pump 12(22)"
 - o "AFW Pump 13(23)"

<u>IF</u> a motor-driven AFW pump can NOT be started, <u>THEN</u> START the turbine-driven AFW pump as follows:

- A. ENSURE "MN STM ISOL MOV-0143 -OPEN.
- b. ENSURE "T & T UNIT TRIP" annunciator (06M4 B-7) -EXTINGUISHED.
- c. ENSURE "T & T MECH OVERSP TRIP" annunciator (06M4 C-8) -EXTINGUISHED.
- d. OPEN "TURB TRIP/THROT MOV-0514

____ 5 OPEN AFW to SG isolation valves
 o AFW to SG 1A(2A) "OCIV MOV-0048"
 o AFW to SG 1B(2B) "OCIV MOV-0065"

o AFW to SG 1C(2C) "OCIV MOV-0085"

o AFW to SG 1D(2D) "OCIV MOV-0019"

DISPATCH an operator to open the valve(s). (25 ft IVC)

REACTOR TRIP RESPONSE

REV. 23

PAGE 3 OF 3

......

ADDENDUM 7 PLACING AFW IN SERVICE

STEP		ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
6	va	ROTTLE the AFW to SG regulating lves to maintain SG NR levels – TWBEN 22% and 50%	DISPATCH an operator to throttle valve(s). (25 ft IVC)
	o	AFW to SG 1A(2A) "AFW REG FV-7525"	
	0	AFW to SG 1B(2B) "AFW REG FV-7524"	
	о	AFW to SG 1C(2C) "AFW REG FV-7523"	
	0	AFW to SG 1D(2D) "AFW REG FV-7526"	
		•	

____ 7 RETURN TO procedure step in effect

REACTOR TRIP RESPONSE

PAGE 1 OF 1

REV. 23

· -

· — ···

.. ...

ADDENDUM 8 FAILING AIR TO MSIVS AND MSIBS

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

....

	NOTE			
	The following steps will vent air to both the MSIV and MSIB.			
1	CLOSE IA isolation to MSIV and MSIB:			
	(58 ft IVC, on wall by MSIV)			
	A - "1(2)-IA-0555" "INSTRUMENT AIR SUBHEADER ISOLATION VALVE"			
	B - "1(2)-IA-0551" "INSTRUMENT AIR SUBHEADER ISOLATION VALVE"			
	C - "1(2)-IA-0547" "INSTRUMENT AIR SUBHBADER ISOLATION VALVE"			
	D - "1(2)-IA-0559" "INSTRUMENT AIR SUBHEADER ISOLATION VALVE"			
_ 2	VENT IA line to atmosphere by opening petcock on air regulator to MSIB:(58 ft IVC on wall by MSIB)			
	A - "1(2)-IA-PCV-7037" or "1(2)-IA-PCV-7038" for "1(2)-MS-FV-7412" "STEAM GENERATOR 1A(2A) MAIN STEAM" "FSV-7414 BYPASS VALVE (ORC)"			
	B - "1(2)-IA-PCV-7050" or "1(2)-IA-PCV-7051" for "1(2)-MS-FV-7422" "STEAM GENERATOR 1B(2B) MAIN STEAM" "FSV-7424 BYPASS VALVE (ORC)"			
	C - "1(2)-IA-PCV-7063" or "1(2)-IA-PCV-7064" for "1(2)-MS-FV-7432" "STEAM GENERATOR 1C(2C) MAIN STEAM" "FSV-7434 BYPASS VALVE (ORC)"			
	D - "1(2)-IA-PCV-7025" or "1(2)-IA-PCV-7023" for "1(2)-MS-FV-7442" "STEAM GENERATOR 1D(2D) MAIN STEAM" "FSV-7444 BYPASS VALVE (ORC)"			
3	RETURN TO procedure step in effect			

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
	•	
	CAUTI	ION
	CP Seal Cooling has been lost and Seal I Injection SHALL <u>NOT</u> be established to a	Inlet Temperature has exceeded 230°F, <u>THEN</u> affected RCPs per this Addendum.
1	DISPATCH operator to perform the following:	
_	a. THROTTLE charging flow control manual bypass two turns open	
	(19 ft MAB RM 79)	
	"1(2)-CV-0255" "CVCS CHARGING DISCHARGE" "FCV-0205 BYPASS VALVE"	
	b. CLOSE charging flow control manual inlet isolation valve	
	(19 ft MAB RM 79)	
	"1(2)-CV-0254A" "CVCS CHARGING DISCHARGE" "FCV-0205 INLET VALVE"	
2	CHECK normal or alternate charging loop isolation valve - OPEN	OPEN normal or alternate charging loop isolation valve.

PAGE 1 OF 5

• •

REACTOR TRIP RESPONSE

ADDENDUM 9

ESTABLISHING ALTERNATE CHARGING FLOW CONTROL

0P0P05-E0-ES01

REV. 23

REV. 23

OPOPO5-EO-ESO1

REACTOR TRIP RESPONSE

PAGE 2 OF 5

•··· •·· ·

<u>ADDENDUM 9</u> ESTABLISHING ALTERNATE CHARGING FLOW CONTROL		
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
3	CHECK charging OCIV - OPEN	PERFORM the following:
•		a. OPEN charging OCIV.
		b. <u>IF</u> charging OCIV will <u>NOT</u> open, <u>THEN</u> DISPATCH operator to open charging OCIV.
		(29 FT MAB RM 108C)
		"1(2)-CV-MOV-0025" "CVCS CHARGING ORC" "CONTAINMENT ISOLATION" "MOV OPERATOR"
4	VERIFY RCP seal inlet temperatures have remained – LESS THAN 230°F	CLOSE affected seal injection OCIV's:
	• •	o "SEAL INJ ISOL MOV-0033A" o "SEAL INJ ISOL MOV-0033B" o "SEAL INJ ISOL MOV-0033C" o "SEAL INJ ISOL MOV-0033D"
5	ESTABLISH charging flow:	
	_ a. CHECK CCP suction aligned to RWST	a. PERFORM the following:
		 OPEN at least one CCP suction valve from the RWST.
		2) CLOSE at least one CCP suction valve from the VCT.
	_ b. CHECK CCP(s) - RUNNING	b. OPEN the recirculation valve(s) for the CCP(s) to be started.
	_ c. ENSURE a CCP running	
_	d. ENSURE running CCP(s) discharged. valve – OPEN	d. <u>IF</u> running CCP(s) discharge valve can <u>NOT</u> be OPENED, <u>THEN</u> GO TO Step 6 of this Addendum. OBSERVE the CAUTION prior to Step 6.

____ .

···**-**······

.....

Step 5 continued next page

_____ , ____

-

- ---- - - - - -----

٠

REACTOR TRIP RESPONSE

PAGE 3 OF 5

REV. 23

	<u>ADDENI</u> ESTABLISHING ALTERNATE	
STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
Step 5 co	ontinued continued from previous page	
¢	e. DISPATCH operator to adjust charging flow control manual bypass to achieve desired flowrate	
	(19 ft MAB RM 79)	
	"1(2)-CV-0255" "CVCS CHARGING DISCHARGE" "FCV-0205 BYPASS VALVE"	
f	C. CLOSE the running CCP(s) recirculation valve	
£	. CHECK RCP seal injection flow - BETWEEN 6 and 13 GPM	g. <u>IF</u> RCP Seal Inlet temperatures have remained - LESS THAN 230°F, <u>THEN</u> PERFORM the following:
	· · · · · · · · · · · · · · · · · · ·	 ENSURE seal injection OCIV(s) OPEN.
		 "SEAL INJ ISOL MOV-0033A" "SEAL INJ ISOL MOV-0033B" "SEAL INJ ISOL MOV-0033C" "SEAL INJ ISOL MOV-0033D"
		2) ADJUST 1(2)-CV-HCV-0218 to establish between 6 and 13 gpm RCP seal injection flow.
}	a. RETURN TO procedure step in effect	

--- -

RESPONSE NOT OBTAINED

OPOPO5-EO-ESO1

ACTION/EXPECTED RESPONSE

STEP

REACTOR TRIP RESPONSE

PAGE 4 OF 5

REV. 23

ADDENDUM 9

ESTABLISHING ALTERNATE CHARGING FLOW CONTROL

CAUTION IF RCP seals have <u>NOT</u> been previously isolated, <u>THEN</u> maximum seal injection flow will result when CCP 1B(2B) Alt Discharge to RCP Seal Isolation CV-0236B OR CCP 1A(2A) Bypass Isol MOV-8348 is opened. 6 DISPATCH operator to perform the following: ۰. 🔄 a. OPEN alternate discharge to RCP seals: (19 ft MAB RM 79) "1(2)-CV-0236B" "CENTRIFUGAL CHARGING PUMP 1B(2B)" "ALT RCP SEAL SUPPLY VALVE" 7 ENSURE CCP 1B(2B) discharge isolation MOV-8377B - CLOSED _ 8 CHECK CCP 1A(2A) - RUNNING GO TO Step 9 of this Addendum. ____ a. OPEN CCP 1A(2A) Bypass Isol MOV-8348 ____ b. ENSURE CCP 1A(2A) discharge isolation MOV-8377A - CLOSED

REACTOR TRIP RESPONSE

PAGE 5 OF 5

REV. 23

ADDENDUM 9 ESTABLISHING ALTERNATE CHARGING FLOW CONTROL		
STEP	ACTION/EXPECTED RESPONSE ESTABLISH desired charging flow:	RESPONSE NOT OBTAINED
9		
	_ a. DISPATCH operator to open:	
	(19 ft MAB RM 79)	
	"1(2)-CV-0255" "CVCS CHARGING DISCHARGE" "FCV-0205 BYPASS VALVE"	
	b. THROTTLE HCV-0206 to establish desired charging flow	
	<pre>_ c. CLOSE the running CCP(s) recirculation valve</pre>	
10	VERIFY RCP seal injection temperature has remained - LESS THAN 230°F	RETURN TO procedure step in effect.
<u> </u>	_ a. ENSURE seal injection OCIV(s) OPEN	
	o "SEAL INJ ISOL MOV-0033A" o "SEAL INJ ISOL MOV-0033B" o "SEAL INJ ISOL MOV-0033C" o "SEAL INJ ISOL MOV-0033D"	
	· ·	
11	DISPATCH operator to THROTTLE Alternate Source to Seal Injection Isolation valve to establish between 6 and 13 gpm RCP seal flow	
	(19 ft MAB RM 79)	
	"1(2)-CV-0246" "CVCS CHARGING PUMPS" "ALT RCP SEAL SUPPLY ISOL VALVE"	
12	RETURN TO procedure step in effect	

REACTOR TRIP RESPONSE

Rev. 23

CONDITIONAL INFORMATION PAGE

SI ACTUATION CRITERIA

<u>IF</u> EITHER condition listed below occurs, <u>THEN</u> ACTUATE SI and GO TO OPOPO5-EO-E000, REACTOR TRIP OR SAFETY INJECTION, Step 1.

- o RCS subcooling based on core exit T/Cs LESS THAN 35°F.
- o Pressurizer level LESS THAN 8%.

AFWST MAKEUP CRITERIA

<u>IF</u> AFWST level lowers to LESS THAN 138,000 GALLONS (26%), <u>THEN</u> INITIATE makeup to the AFWST per OPOPO2-AF-0001, AUXILIARY FEEDWATER, to prevent inventory problems during cooldown.

BOOO TRANSITION CRITERIA

<u>IF</u> an SI actuation occurs during this procedure. <u>THEN</u> GO TO OPOPO5-EO-EOOO; REACTOR TRIP OR SAFETY INJECTION, Step 1.

RCFC_COOLING_CRITERIA

RESTORE CCW to RCFCs within 30 minutes after a LOOP signal.

SEQUENCER LOADING VERIFICATION

IF a LOOP has occurred. THEN PERFORM Addendum 5, Sequencer Loading Verification - Mode II.

MSIV AND MSIB CLOSURE CRITERIA

<u>IF</u> a loss of secondary support systems occurs that impairs the ability of secondary systems to provide a heat sink for the Steam Generators, <u>THEN</u> CLOSE MSIVs and MSIBs. (For example: loss of Condenser Availability, C-9.)