

April 2, 2007

Mr. Charles D. Naslund  
Senior Vice President and  
Chief Nuclear Officer  
Union Electric Company  
Post Office Box 620  
Fulton, MO 65251

SUBJECT: CALLAWAY PLANT, UNIT 1 - ISSUANCE OF AMENDMENT RE:  
RELOCATION OF CYCLE-SPECIFIC PARAMETER LIMITS TO THE CORE  
OPERATING LIMITS REPORT (TAC NO. MD2873)

Dear Mr. Naslund:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 183 to Facility Operating License No. NPF-30 for the Callaway Plant, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated August 17, 2006 (ULNRC-05322).

The amendment revised TSs 2.1.1, "Reactor Core SLs [Safety Limits]," 3.3.1, "Reactor Trip System (RTS) Instrumentation," 3.4.1, RCS [Reactor Coolant System] Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits," and 5.6.5, "Core Operating Limits Report (COLR)." The changes (1) relocate certain operating cycle-specific core operating limits, including TS Figure 2.1.1-1, "Reactor Core Safety Limits," from the TSs to the plant COLR, (2) add two new safety limits for departure from nucleate boiling ratio and peak fuel centerline temperature, and (3) add topical reports to TS 5.6.5 and have the reports cited by only the report title and number. TS 5.6.5 is expanded to include the limits from TSs 2.1.1, 3.3.1, and 3.4.1. The changes are consistent with NRC-approved Standard Technical Specification Task Force (TSTF) Change Traveler TSTF-339, Revision 2, "Relocate TS Parameters to COLR," and TSTF-363, Revision 0, "Relocate Topical Report References in ITS [Improved Technical Specification] 5.6.5, COLR."

C. D. Naslund

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A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

***/RA/***

Jack Donohew, Senior Project Manager  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-483

Enclosures: 1. Amendment No. 183 to NPF-30  
2. Safety Evaluation

cc w/encls: See next page

C. D. Naslund

-2-

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**ADAMS Accession Nos.: Pkg. ML070600786** (Amendment ML070600768, License/TS Pages ML070610246)

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OFFICIAL RECORD COPY

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June 2006

UNION ELECTRIC COMPANY

CALLAWAY PLANT, UNIT 1

DOCKET NO. 50-483

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 183  
License No. NPF-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Union Electric Company (UE, the licensee), dated August 17, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications and paragraph 2.C.(2) of Facility Operating License No. NPF-30 as indicated in the attachment to this license amendment.

3. This amendment is effective as of its date of issuance, and shall be implemented within 90 days of the date of issuance. The final Technical Specification (TS) Bases changes including the licensee's application dated August 17, 2006, will be processed under the licensee's program for updates to the TS Bases, in accordance with TS 5.5.14, at the time this amendment is implemented. The final changes to the core operating limits report (COLR) including those in the licensee's application dated August 17, 2006, will be submitted to the NRC in accordance with the update process covered by TS 5.6.5.d.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

David Terao, Chief  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Facility  
Operating License and  
Technical Specifications

Date of Issuance: April 2, 2007

ATTACHMENT TO LICENSE AMENDMENT NO. 183

FACILITY OPERATING LICENSE NO. NPF-30

DOCKET NO. 50-483

Replace the following pages of the Facility Operating License No. NPF-30 and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Operating License

REMOVE

INSERT

- 3 -

- 3 -

Technical Specifications

REMOVE

INSERT

2.0-1

2.0-1

2.0-2

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3.3-23

3.3-23

3.3-24

3.3-24

3.4-1

3.4-1

3.4-2

3.4-2

5.0-20

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5.0-23

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 183 TO FACILITY OPERATING LICENSE NO. NPF-30

UNION ELECTRIC COMPANY

CALLAWAY PLANT, UNIT 1

DOCKET NO. 50-483

## 1.0 INTRODUCTION

By application dated August 17, 2006 (Agencywide Documents Access and Management System Accession No. ML062360244), Union Electric Company (the licensee) requested changes to the Technical Specifications (TSs) of the Facility Operating License No. NPF-30 for the Callaway Plant, Unit 1 (Callaway).

The amendment would revise TSs 2.1.1, "Reactor Core SLs [Safety Limits]," 3.3.1, "Reactor Trip System (RTS) Instrumentation," 3.4.1, RCS [Reactor Coolant System] Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits," and 5.6.5, "Core Operating Limits Report (COLR)." The proposed changes would (1) relocate certain operating cycle-specific core operating limits, including TS Figure 2.1.1-1, "Reactor Core Safety Limits," from the TSs to the plant COLR, (2) add two new safety limits for departure from nucleate boiling ratio and peak fuel centerline temperature, and (3) add topical reports to TS 5.6.5 and have the reports cited by only the report title and number. TS 5.6.5 would be expanded to also include the relocate cycle-specific reactor coolant system (RCS) related parameter limits from TSs 2.1.1, 3.3.1, and 3.4.1. The proposed changes are consistent with NRC-approved Standard Technical Specification Task Force (TSTF) Change Traveler TSTF-339, Revision 2, "Relocate TS Parameters to COLR," and TSTF-363, Revision 0, "Relocate Topical Report References in ITS [Improved Technical Specification] 5.6.5, COLR."

The overtemperature  $\Delta T$  and overpower  $\Delta T$  setpoint parameters will be relocated from TS Table 3.3.1-1, "Reactor Trip System Instrumentation," to the COLR. The pressurizer pressure and RCS average temperature values will be relocated from TS 3.4.1 to the COLR.

In Attachments 4 and 5, respectively, to its application, the licensee identified the changes to be made to the TS Bases and to the COLR for Callaway for this amendment. The NRC staff does not approve these changes; however, it reviews these changes to determine that these changes are consistent with the proposed amendment.

## 2.0 BACKGROUND

In Section 50.36 of Title 10 of the *Code of Federal Regulations* (10 CFR 50.36), the Commission established its regulatory requirements related to the content of the TSs. Pursuant

to 10 CFR 50.36, TSs are required to include items in the following five specific categories related to station operation: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements (SRs); (4) design features; and (5) administrative controls. The rule does not specify the particular requirements to be included in a plant's TSs. As stated in 10 CFR 50.36(c)(2)(i), the "[l]imiting conditions for operation are the lowest functional capability or performance levels of equipment required for safe operation of the facility. When a limiting condition for operation of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the technical specifications ..."

Guidance on the relocation of cycle-specific TS parameters to the COLR is provided to all power reactor licensees and applicants in Nuclear Regulatory Commission (NRC) Generic Letter (GL) 88-16, "Removal of Cycle-Specific Parameter Limits from Technical Specifications," dated October 3, 1988. In the GL, the NRC staff stated that license amendments are generally required every refueling outage to update the cycle-specific parameter limits in the TSs; however, there are methodologies developed for the licensee to determine these cycle-specific parameters that have been reviewed and approved by the staff. As a consequence, the NRC staff review of proposed changes to the TSs to update these parameter limits is primarily limited to the confirmation that the updated limits were calculated by the approved methodology and consistent with the appropriate plant-specific safety analysis. The COLR was created to place the NRC-approved methodologies in the TSs and allow licensees to use later revisions of these methodologies to update the parameters without requiring a change to the TSs.

The licensee stated that its justification to expand the COLR was provided in Westinghouse Energy System Topical Report WCAP-14483-A, "Generic Methodology for Expanding Core Operating Limits Report," dated November 1995, which was approved by the NRC. The NRC staff's letter and safety evaluation (SE) approving the use of the topical report were issued January 19, 1999, and are included in the beginning of the topical report. The topical report addresses the relocation of the (1) reactor core safety limits figure, (2) overtemperature  $\Delta T$  (OTDT) and overpower  $\Delta T$  (OPDT) setpoint parameter values for reactor trip instrumentation, and (3) DNB parameter limits in the TSs to the COLR. The NRC staff's SE for the topical report had no conditions specified on the use of the topical report.

The NRC staff has approved WCAP-14483-A as an acceptable method to relocate these TS requirements to the COLR consistent with GL 88-16. The NRC staff incorporated the generic methodology for expanding the COLR of WCAP-14483-A into NUREG-1431, the improved Standard Technical Specifications (STS) for Westinghouse plants, which are the STS that the Callaway TSs are based upon. In TSTF-339, Revision 2, "Relocate TS Parameters to COLR," the relocation of the (1) reactor core safety limits figure, (2) OTDT and OPDT setpoint parameter values for reactor trip instrumentation, and (3) DNB parameter limits in the TSs to the COLR in WCAP-14483-A was approved by the NRC staff for NUREG-1431. In TSTF-363, Revision 0, "Relocate Topical Report References in ITS [Improved Technical Specification] 5.6.5, COLR," the NRC-approved topical reports are listed only by a reference to the report number and title with a note that the COLR provides the complete citation of the report (i.e., report number, title, revision, date, and any supplements). These are the changes for TSs 2.1.1, 3.3.1, and 3.4.1 that the licensee has proposed in its application.

### 3.0 EVALUATION

The licensee proposed six changes to TSs 2.1.1, 3.3.1, 3.4.1, and 5.6.5. They are addressed in the next section with each change identified and evaluated separately.

#### 3.1 Proposed Changes to the TSs

In its application, the licensee proposed the following six changes to the TSs:

1. TS 2.1.1 would be revised by relocating Figure 2.1.1-1 to the COLR and replacing the figure with more specific safety limits for DNBR and peak fuel centerline temperature. Figure 2.1.1-1 would be deleted from TS 2.1.1.

The relocation of the DNB parameters limit values (including the TS figure) to the COLR allows the licensee the flexibility to utilize the available margins to increase cycle operating margins and improve core reload designs without the requirement of cycle-specific license amendments. This results in a more complete COLR containing not only cycle-specific core reload-related parameters, but also cycle-specific operating condition parameters. Thus, the safety analyses could credit the actual cycle-specific operating condition in the same way that the core reload designs currently do. The COLR and safety analyses will more closely reflect the cycle-specific conditions for which the plant control and protection systems are set for a given cycle.

The licensee stated in its application that (1) the methodology used to calculate TS Figure 2.1.1-1 is in WCAP-9272-P-A, "Westinghouse Reload Safety Evaluation Methodology," which has been reviewed and approved by the NRC on May 28, 1985, and (2) the existing specification for the COLR in TS 5.6.5 requires the WCAP-9272-P-A report to be used as one of the analytical methods to determine the core operating limits in the COLR. The changes proposed by the licensee are consistent with the changes identified in WCAP-14483-A with respect to deleting TS Figure 2.1.1-1 that were reviewed and approved by the NRC staff. The methodology used to determine the figure that would be in the COLR is in the WCAP-9272-P-A report. This report is listed in TS Section 5.6.5.b and the COLR. The licensee stated that the proposed requirements in Modes 1 and 2 for the DNBR and peak fuel centerline temperature (i.e., the proposed safety limits in TSs 2.1.1.1 and 2.1.1.2) are the design-basis limits for the plant and are the limits that must be satisfied for Condition I and II transients. Based on this, the NRC staff concludes that the proposed changes to TS 2.1.1 are acceptable and meet 10 CFR 50.36. In addition, the proposed changes to TS 2.1.1 are consistent with TSTF-339, Revision 2.

2. TS Table 3.3.1-1 would be revised to relocate values of certain parameters in Note 1 (the OTDT) and Note 2 (the OPDT) to the COLR. The values of these parameters in the OTDT and OPDT expressions would be replaced with asterisks and the following sentence would be added to the TSs: "The values denoted with \* are specified in the COLR."

The licensee stated in its application that placing these setpoints in the COLR allows the setpoints to be based on cycle-specific core design parameters, which are verified on a cycle-specific basis, thereby maintaining appropriate but not overly conservative TS limits. The NRC staff has previously approved the relocation of the OTDT and OPDT setpoint parameter

values to the COLR for the Catawba, McGuire, and Seabrook Nuclear Stations. The NRC staff agrees that placing these setpoints in the COLR allows them to be based on cycle-specific core design parameters, which are verified on a cycle-specific basis, thereby maintaining an appropriate but not excessive level of conservatism in the TS limits. The changes proposed for Callaway by the licensee are consistent with the approved changes identified in WCAP-14483-A for relocating parameter values in the OTDT and OPDT equations in Table 3.3.1-1. The applicable NRC-approved setpoint methodology is in WCAP-8745-P-A, "Design Bases for the Thermal Overpower  $\Delta T$  and Thermal Overtemperature  $\Delta T$  Trip Functions," dated September 1986. The licensee has proposed to add this reference to the list of NRC-approved analytical methods in TS 5.6.5.b. For the reasons stated above, the staff concludes that the proposed changes to Notes 1 and 2 of TS Table 3.3.1-1 and to relocate parameters to the COLR are acceptable and meet 10 CFR 50.36. In addition, the proposed changes to TS Table 3.3.1-1 are consistent with TSTF-339, Revision 2.

3. TS 3.4.1 would be revised to relocate the values of pressurizer pressure and RCS average temperature ( $T_{avg}$ ) to the COLR. The values of these parameters in LCO 3.4.1 and SRs 3.4.1.1 and 3.4.1.2 would be replaced with the following phrase: "the limit specified in the COLR." The mathematical symbols of " $\geq$ " and " $\leq$ " are also replaced by their meaning in text: "greater than or equal to" and "less than or equal to."

The licensee stated that the proposed change to the Applicability Note provides consistency in the Note with NUMARC 93-03, "Writers Guide for the Restructured Technical Specifications," and NUREG-1431, Revision 1, "Standard Technical Specifications Westinghouse Plants." The current TSs for Callaway are based on this standard. The limits for pressurizer pressure, RCS  $T_{avg}$ , and RCS total flow rate would be relocated from TS 3.4.1 to the COLR. However, the minimum RCS total flow rate of 382,630 gallons per minute, based on the maximum analyzed steam generator tube plugging, would be retained in SR 3.4.1.4 to assure that a lower RCS total flow rate than approved by the NRC will not be used. Because this minimum value is retained in the TSs, any reduction in the RCS flow rate would have to be reviewed by the NRC. The proposed changes to TS 3.4.1 are also consistent with WCAP-14483-A.

The licensee stated that the analytical methods that address the limits in TS 3.4.1 are in WCAP-9272-P-A, "Westinghouse Reload Safety Evaluation Methodology," WCAP-11397-P-A, "Revised Thermal Design Procedure," and WCAP-14565-P-A, "VIPRE-01 Modeling and Qualification for Pressurized Water Reactor Non-LOCA Thermal-Hydraulic Safety Analysis." The first topical report is listed as number 1 in TS 5.6.5.b and the licensee has proposed to add the other two topical reports to TS 5.6.5.b, as numbers 5 and 6. The NRC letters and SEs that approved these three topical reports are listed in Appendix A, pages 17 and 18, in Attachment 5, draft COLR markups, to the licensee's application. Therefore, the NRC has approved the use of these topical reports, and the NRC staff agrees that they are applicable to Callaway.

Based on the above, the NRC staff concludes that the proposed changes to TS 3.4.1 are acceptable and meet 10 CFR 50.36. In addition, the proposed changes to TS 3.4.1 are consistent with TSTF-339, Revision 2.

4. TS 5.6.5a would be modified to add (1) Figure 2.1.1-1 of TS 2.1.1, (2) OTDT and OPDT setpoint parameters of TS 3.3.1, and (3) RCS pressure and temperature DNB limits of TS 3.4.1 to the list of core operating limits specified in the COLR.

In items 1, 2, and 3 above, the licensee has proposed changes to TSs 2.1.1, 3.3.1, and 3.4.1 to replace these core operating limits in the TSs with references to the COLR of TS 5.6.5. In addition, the NRC staff has verified that the licensee has included these core operating limits in the draft COLR markups, which are in Attachment 5 to the licensee's application. Having TSs 3.1.1, 3.3.1, and 3.4.1 reference the COLR will require the licensee to use the core operating limits that are in the COLR and those limits will be the values that were removed from the TSs. Based on this and because TS 5.6.5a lists those core operating limits that are in the COLR, the NRC staff concludes that the proposed change to add the above operating limits in TSs 2.1.1, 3.3.1, and 3.4.1 to TS 5.6.5a is acceptable and, therefore, meets 10 CFR 50.36.

5. TS 5.6.5.b would be modified to replace the item 4 by the following topical reports which would be numbered 4 through 12:
  4. WCAP-12610-P-A, "Vantage+ Fuel Assembly Reference Core Report,"
  5. WCAP-11397-P-A, "Revised Thermal Design Procedure,"
  6. WCAP-14565-P-A, "VIPRE-01 Modeling and Qualification for Pressurized Water Reactor Non-LOCA Thermal-Hydraulic Safety Analysis,"
  7. WCAP-10851-P-A, "Improved Fuel Performance Models for Westinghouse Fuel Rod Design and Safety Evaluations,"
  8. WCAP-15063-P-A, "Westinghouse Improved Performance Analysis and Design Model,"
  9. WCAP-8745-P-A, "Design Bases for the Thermal Overpower  $\Delta T$  and Thermal Overtemperature  $\Delta T$  Trip Functions,"
  10. WCAP-10965-P-A, "ANC: A Westinghouse Advanced Nodal Computer Code,"
  11. WCAP-11596-P-A, "Qualification of the PHOENIX-P/ANC Nuclear Design System for Pressurized Water Reactor Cores," and
  12. WCAP-13524-P-A, "APOLLO: A One Dimensional Neutron Diffusion Theory Program."

The licensee has listed the NRC letters that approved the above topical reports in Attachment 5 to its application. The NRC letters are on page 17 and 18, Appendix A, of the draft COLR markups in Attachment 5. As verified by the NRC staff, numbers 5 through 12 of the above topical reports were either referenced in (1) the licensee's license amendment request dated September 17, 2004 (ADAMS Accession No. ML042870364), to replace its steam generators or (2) sections of the Final Safety Analysis Report for Callaway. The amendment associated with the licensee's amendment request dated September 17, 2004, is Amendment No. 168 issued

September 29, 2005 (ADAMS Accession No. ML052570054). Based on this, the NRC staff agrees with the licensee that these topical reports apply to Callaway.

The proposed topical report number 4 above is in fact the fourth topical report currently listed in TS 5.6.5b, but the reference is rewritten to only state "WCAP-12610-P-A, "Vantage+ Fuel Assembly Reference Core Report." This does not substantially change the reference listed in TS 5.6.5b, but rather rewrites the reference to WCAP-12610-P-A without listing the revision and date of the topical report. This proposed change to list references without giving their revision and date is addressed in item 6 below.

The licensee stated that the above topical reports numbered 5 through 12 are NRC-approved topical reports for the core operating limits being added to TS 5.6.5a and the COLR, as well as for limits already listed in TS 5.6.5a and the COLR. The "-A" in the title of a topical report indicates that the NRC staff has approved the use of the report at nuclear power plants; the NRC letter and SE approving the report is included in the topical report when the report is issued with the "-A." Because the licensee states that these topical reports are used for the core operating limits that are, or will be listed with this amendment, in TS 5.6.5a and the COLR, and the topical reports are approved by the NRC and, as stated above, are applicable to Callaway, the NRC staff concludes that the proposed addition of the above topical reports numbered 5 through 12 to TS 5.6.5b is acceptable, and meets 10 CFR 50.36. Topical report number 4 above does not have to be approved to be listed in TS 5.6.5b because it is the topical report number 4 already listed in TS 5.6.5b.

6. TS 5.6.5b would also be modified to revise the references to the existing topical reports listed in TS 5.6.5b to limit the references to the report number and title with the plant-specific COLR providing the complete citation of the report (i.e., report number, title, revision, date, and any supplements).

The licensee has also proposed to modify the references to the documents listed in TS 5.6.5b, as the NRC-approved analytical methods used to determine the core operating limits listed in TS 5.6.5a. The purpose of the proposed change is to have the references in TS 5.6.5b cite only the report number and title; the complete citation for the topical reports (i.e., the report number, title, revision, date, and any supplements) would be listed by the licensee in the plant COLR document. This change would allow the licensee to use the later edition NRC-approved topical report of the same report number and title to determine a core operating limit that is listed in TS 5.6.5a without having to request NRC approval. The subsequent edition of such a NRC-approved topical report would have to be reviewed and approved for use by the NRC, including the issuance of an SE approving the use of the topical report. In other words, because TS 5.6.5b requires that the methodology must be previously reviewed and approved by the NRC, the licensee would not be able to use a revised topical report unless the NRC had approved its use.

In approving references to the topical reports in TS 5.6.5.b using only the report number and title, the NRC staff emphasizes that the SE that approves the particular topical report may contain conditions on the use of the topical report that are not listed in the topical report itself. This is particularly true for plant-specific topical reports, but not for vendor (e.g., owners groups) topical reports, such as the Westinghouse WCAP topical reports. The practice for approving vendor topical reports is that the topical reports are issued containing a copy of the NRC letter

and SE; therefore, the NRC-approved vendor topical report will contain any conditions specified in the SE and the licensee would have to determine that it meets the conditions in the SE to use the topical report.

Based on the above, the NRC staff concludes that listing the WCAP topical reports in TS 5.6.5b with only the topical report number and title (and, thus, without the revision number and date) is acceptable and meets 10 CFR 50.36.

The identified changes to the Callaway COLR in Attachment 5 to the application show that the COLR will provide the complete citation of the report (i.e., report number, title, revision, date, and any supplements).

### 3.2 Conclusion

Based on the above evaluation, the NRC staff concludes that the cycle-specific core operating limits in TSs 2.1.1, 3.3.1, and 3.4.1 are in conformance with WCAP-14483-A and are not required to be in the TSs under 10 CFR 50.36. They are not required to be in the TSs to prevent the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety. Accordingly, the core operating limits may be relocated from the TSs to the COLR. Based on the above evaluation, the staff also concludes that incorporating the proposed changes to TS Table 3.3.1-1 and revising the list of specifications and references in TS Section 5.6.5 are acceptable and meet 10 CFR 50.36. Based on these proposed changes meeting 10 CFR 50.36, the NRC staff further concludes that the proposed amendment is acceptable.

The licensee agreed to incorporate the changes to the COLR and the TS Bases during the implementation of the amendment. Therefore, the implementation of the amendment will include: (1) the relocation of the requirements from TSs 2.1.1, 3.3.1, and 3.4.1 to the COLR, and (2) the changes to the TS Bases, as specified in the licensee's request of August 17, 2006, and this SE. In its letter dated August 17, 2006, the licensee stated the following: "Final Bases changes will be processed under [the licensee's] program for updates per TS 5.5.14, 'Technical Specifications Bases Control Program,' at the time this amendment is implemented. Final changes to the COLR will be submitted to the NRC per the update process covered by TS 5.6.5.d." These are the changes identified in Attachments 4 and 5, respectively, to the letter dated August 17, 2006. As agreed to by the licensee, these statements are included in the description of the implementation of the amendment.

In incorporating the additional core operating limits and topical reports in TS 5.6.5, the licensee moved text, which is not being changed, from (1) TS page 5.0-21 back onto TS page 5.0-20 and (2) TS page 5.0-22 to TS page 5.0-23. Because of this, TS pages 5.0-20 and 5.0-23 are also being issued with this amendment. In addition, the licensee is also replacing the periods at the end of the existing items 6 and 7 with commas in TS 5.6.5.a to be consistent with adding the items 7, 8, and 9 to TS 5.6.5.a. This is an editorial change with no effect on any requirements in the TSs.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Missouri State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and to the SRs. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (72 FR 1781, published January 16, 2007). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Date: April 2, 2007