

FEB 21 2007



LR-N07-0036

United States Nuclear Regulatory Commission
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Washington, DC 20555

**SALEM GENERATING STATION – UNIT 1 AND UNIT 2
FACILITY OPERATING LICENSE NOS. DPR-70 AND DPR-75
NRC DOCKET NOS. 50-272 AND 50-311**

**Subject: LCR S06-03 SUPPLEMENT
REQUEST FOR CHANGE TO TECHNICAL SPECIFICATIONS
ACCIDENT MONITORING INSTRUMENTATION AND SOURCE
CHECK DEFINITION**

- References:**
- (1) Letter from PSEG to NRC: "LCR S06-03, Request for Change to Technical Specifications, Accident Monitoring Instrumentation and Source Check Definition, Salem Nuclear Generating Station, Units 1 and 2, Facility Operating Licenses DPR-70 and DPR-75, Docket Nos. 50-272 and 50-311", dated May 1, 2006
 - (2) Letter from PSEG to NRC: "Response to RAIs on LCR S06-03, Request for Change to Technical Specifications, Accident Monitoring Instrumentation and Source Check Definition, Salem Nuclear Generating Station, Units 1 and 2, Facility Operating Licenses DPR-70 and DPR-75, Docket Nos. 50-272 and 50-311", dated October 9, 2006

In accordance with the requirements of 10 CFR 50.90, PSEG Nuclear LLC (PSEG) previously submitted License Change Request (LCR) S06-03, dated May 1, 2006, to amend the Technical Specifications (TS) for Salem Generating Station Unit 1 and Unit 2 (Reference 1). LCR S06-03 contained two proposed changes: (1) Relocation of the Main Steamline Discharge (Safety Valves and Atmospheric Dumps) Radiation Monitors (R46) from the radiation monitoring instrumentation technical specification (3.3.3.1) to the accident monitoring technical specification (3.3.3.7), and (2) Modification of TS Definition 1.31, SOURCE CHECKS, to allow for different methods to comply with the SOURCE CHECK requirement.

PSEG received a Request for Additional Information (RAI) from the NRC on LCR S06-03. The response to the RAI was provided via Reference 2 dated October 9, 2006.

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On February 8, 2007, PSEG and the NRC Staff discussed (via telecon) the SOURCE CHECK definition change requested in LCR S06-03. PSEG agreed to revise the definition to include specific options for complying with the source check requirement. The revised wording is provided below:

"SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to either (a) an external source of increased radioactivity, (b) an internal source of radioactivity (keep-alive source), or (c) an equivalent electronic source check."

The revised TS pages are provided in Attachment 1.

The revised definition is an updated clarification, and meets the intent of the original definition and ensures the proper functioning of detectors, based on the detector design. If a detector is designed with source check capability via exposure to an external source (i.e., by an actual source attached to a mechanical device as part of the detector design), then this method would be employed. Improved technology detectors will be "qualitatively assessed" in a manner that is a significant improvement over the old methodology, and that is appropriate to their design. The proposed revised definition will allow for equivalent qualitative assessment methods, recognizing the technology of improved designs. As examples, the R46 detectors use a keep alive source, and the R44 detectors use an equivalent electronic source check (refer to Salem Amendments 272 and 253, NRC letter dated May 5, 2006)¹.

If you have any questions or require additional information, please do not hesitate to contact Mr. Jamie Mallon at (610) 765-5507.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on 2/21/07
(Date)

Sincerely,



Thomas P. Joyce
Site Vice President
Salem Generating Station

Attachments 1

¹ Note that the SOURCE CHECK requirement is only applicable for detectors in TS 3.3.3.1. The R44 detectors were relocated to TS 3.3.3.7, and the R46 detectors are proposed for relocation to TS 3.3.3.7.

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TECHNICAL SPECIFICATION PAGES WITH PROPOSED CHANGES

The following Technical Specifications for Facility Operating License DPR-70 (Salem Unit 1) are affected by this change request:

<u>Technical Specification</u>	<u>Page</u>
Definition 1.31	1-6

The following Technical Specifications for Facility Operating License DPR-75 (Salem Unit 2) are affected by this change request:

<u>Technical Specification</u>	<u>Page</u>
Definition 1.31	1-6

DEFINITIONS

REACTOR TRIP SYSTEM RESPONSE TIME

1.26 The REACTOR TRIP SYSTEM RESPONSE TIME shall be the time interval from when the monitored parameter exceeds its trip setpoint at the channel sensor until loss of stationary gripper coil voltage.

REPORTABLE EVENT

1.27 A REPORTABLE EVENT shall be any of those conditions specified in Section 50.73 to 10CFR Part 50.

SHUTDOWN MARGIN

1.28 SHUTDOWN MARGIN shall be the instantaneous amount of reactivity by which the reactor is subcritical or would be subcritical from its present condition assuming all full length rod cluster assemblies (shutdown and control) are fully inserted except for the single rod cluster assembly of highest reactivity worth which is assumed to be FULLY WITHDRAWN.

SITE BOUNDARY

1.29 The SITE BOUNDARY shall be that line beyond which the land is not owned, leased, or otherwise controlled by the licensee, as shown in Figure 5.1-3, and which defines the exclusion area as shown in Figure 5.1-1.

SOLIDIFICATION

1.30 Not Used

SOURCE CHECK

1.31 SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to either (a) an external source of increased radioactivity, or (b) an internal source of radioactivity (keep-alive source), or (c) an equivalent electronic source check a source of increased radioactivity.

STAGGERED TEST BASIS

1.32 A STAGGERED TEST BASIS shall consist of:

- a. A test schedule for (n) systems, subsystems, trains, or other designated components obtained by dividing the specified test interval into (n) equal subintervals.

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