



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION IV  
611 RYAN PLAZA DRIVE, SUITE 400  
ARLINGTON, TEXAS 76011-4005

February 23, 2007

Kevin T. Walsh, Vice President  
of Operations  
Waterford 3  
Entergy Operations, Inc.  
17265 River Road  
Killona, LA 70057-3093

SUBJECT: ERRATA FOR WATERFORD STEAM ELECTRIC STATION, UNIT 3 - NRC  
INTEGRATED INSPECTION REPORT 05000382/2006005

Dear Mr. Walsh:

Please replace the Summary of Findings page in Inspection Report 05000382/2006005, dated February 13, 2007, with the attached revised page. The following changes are necessary to change the number of findings from two to three and to delete the end of that sentence "and one apparent violation with potential safety significance greater than Green."

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, and its enclosure, will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Should you have any questions concerning this inspection, we will be pleased to discuss them with you.

Sincerely,

A handwritten signature in dark ink, appearing to read "J. Clark", is written over the typed name.

Jeff A. Clark, Chief  
Project Branch E  
Division of Reactor Projects

Docket: 50-482  
License: NPF-42

Enclosure:  
Errata pages for NRC Integrated Inspection Report 05000382/2006005

Entergy Operations, Inc.

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cc w/enclosure:

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ENCLOSURE  
Revised Pages for NRC Integrated Inspection  
Report 05000382/2006005

## SUMMARY OF FINDINGS

IR05000382/2006-005; 10/08/2006-12/31/2006; Waterford Steam Electric Station, Unit 3; Postmaintenance Testing; Refueling and Other Outage Activities; Emergency Plan Biennial Program Inspection

The report covered a 3-month period of inspection by resident inspectors, a project engineer, a senior operations engineer, a senior health physics inspector, a reactor inspector, and an emergency preparedness inspector. The inspectors identified three Green findings, which were noncited violations. The significance of most findings is indicated by their color (Green, White, Yellow, or Red) using Inspection Manual Chapter 0609, "Significance Determination Process." Findings for which the Significance Determination Process does not apply may be Green or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

### A. NRC-Identified and Self-Revealing Findings

#### Cornerstone: Initiating Events

- Green. A self-revealing violation of very low safety significance of Technical Specification 6.8.1.a was identified for an inadequate procedure for installing a bolted joint that provided structural support for the pressurizer. Specifically, the installation procedure required applying 8750 ft-lbs torque to make up a bolted joint. Following corrective actions, the licensee discovered that the break away torque on several bolts exceeded 13,400 ft-lbs. The improper bolt tensioning resulted in failure of 1 of 16 bolts and the partial cracking of 3 other bolts that potentially could affect the pressurizer's function in a safe shutdown earthquake event. The licensee has since replaced all pressurizer skirt bolting and installed the bolting to an approved torque specification.

This finding is more than minor because if left uncorrected it could have become a more safety significant concern. The finding was associated with the equipment performance attribute of the Initiating Events cornerstone, and it affected the cornerstone objective to limit the likelihood of those events that upset plant stability and challenge critical safety functions during power operations. This finding was determined to have very low safety significance because a seismic event would not have resulted in a loss-of-coolant accident that exceeded the Technical Specification limit for reactor coolant system leakage. Therefore, this issue screened out in Phase 1 of the Manual Chapter 0609 "Significance Determination Process," because there was no actual loss of safety function (Section 1R20).

Entergy Operations, Inc.

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**ROPreports**

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SUNSI Review Completed:      ADAMS: ☒ Yes ☐ No Initials:       
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