

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or instrument calibration or when associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by operation of the facility.

3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter 1: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

A. Maximum Power Level

Omaha Public Power District is authorized to operate the Fort Calhoun Station, Unit 1, at steady state reactor core power levels not in excess of 1500 megawatts thermal (rated power).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 248 are hereby incorporated in the license. Omaha Public Power District shall operate the facility in accordance with the Technical Specifications.

C. Security and Safeguards Contingency Plans

The Omaha Public Power District shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Fort Calhoun Station Security Plan, Training and Qualification Plan, Safeguards Contingency Plan," submitted by letter dated May 19, 2006,

TECHNICAL SPECIFICATIONS

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- 2.13 DELETED
- 2.14 Engineered Safety Features System Initiation Instrumentation Settings
- 2.15 Instrumentation and Control Systems
- 2.16 River Level
- 2.17 Miscellaneous Radioactive Material Sources
- 2.18 DELETED
- 2.19 DELETED
- 2.20 Steam Generator Coolant Radioactivity
- 2.21 Post-Accident Monitoring Instrumentation
- 2.22 DELETED
- 2.23 Steam Generator (SG) Tube Integrity

3.0 SURVEILLANCE REQUIREMENTS

- 3.1 Instrumentation and Control
- 3.2 Equipment and Sampling Tests
- 3.3 Reactor Coolant System and Other Components Subject to ASME XI Boiler and Pressure Vessel Code Inspection and Testing Surveillance
- 3.4 DELETED
- 3.5 Containment Test
- 3.6 Safety Injection and Containment Cooling Systems Tests
- 3.7 Emergency Power System Periodic Tests
- 3.8 Main Steam Isolation Valves
- 3.9 Auxiliary Feedwater System
- 3.10 Reactor Core Parameters
- 3.11 DELETED
- 3.12 Radioactive Waste Disposal System
- 3.13 Radioactive Material Sources Surveillance
- 3.14 DELETED
- 3.15 DELETED
- 3.16 Residual Heat Removal System Integrity Testing
- 3.17 Steam Generator (SG) Tube Integrity

4.0 DESIGN FEATURES

- 4.1 Site
- 4.2 Reactor Core
- 4.3 Fuel Storage

TECHNICAL SPECIFICATIONS

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2-3	Instrument Operating Requirements for Engineered Safety Features	Section 2.15
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	of Miscellaneous Instrumentation and Controls	
3-3a	Minimum Frequency for Checks, Calibrations and Functional	Section 3.1
	Testing of Alternate Shutdown Panels (AI-185 and AI-212) and Emergency Auxiliary Feedwater Panel (AI-179) Instrumentation and Control Circuits	
3-4	Minimum Frequencies for Sampling Tests	Section 3.2
3-5	Minimum Frequencies for Equipment Tests	Section 3.2
3-6	Reactor Coolant Pump Surveillance	Section 3.3

TECHNICAL SPECIFICATIONS

TECHNICAL SPECIFICATIONS - TABLES

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<u>TABLE</u>	<u>DESCRIPTION</u>	<u>SECTION</u>
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2-9	RCS Pressure Isolation Valves	Section 2.1
3-6	Reactor Coolant Pump Surveillance	Section 3.3
1-1	RPS LSSS	Section 1.0

TECHNICAL SPECIFICATIONS

Not Used

TABLE 3-3 (Continued)

**MINIMUM FREQUENCIES FOR CHECKS, CALIBRATIONS AND TESTING
OF MISCELLANEOUS INSTRUMENTATION AND CONTROLS**

<u>Channel Description</u>	<u>Surveillance Function</u>	<u>Frequency</u>	<u>Surveillance Method</u>
25. Containment Purge Isolation Valves (PCV-742A, B, C, & D)	a. Check	M	a. Verify valve position using control room indication.
26. Not Used			
27. Containment Water Level Narrow Range (LT-599 & LT-600)	a. Check	M	a. CHANNEL CHECK
	b. Calibrate	R	b. CHANNEL CALIBRATION
Wide Range (LT-387 & LT-388)	a. Check	M	a. CHANNEL CHECK
	b. Calibrate	R	b. CHANNEL CALIBRATION
28. Containment Wide Range Pressure Indication	a. Check	M	a. CHANNEL CHECK
	b. Calibrate	R	b. CHANNEL CALIBRATION
29. Not Used			