



January 31, 2007

Kevin J. Moles
Manager Regulatory Affairs

RA 07-0012

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Subject: Docket no. 50-482: Inservice Inspection Program Third
Interval, First Period, Refueling Outage 15 Owner's Activity
Report

Gentlemen:

Enclosed is Wolf Creek Nuclear Operating Corporation's (WCNOC's) Owner's Activity Report (Form OAR-1, Report Number WCRE-16, I3-P1-RF-15) for inservice inspections performed prior to and during Wolf Creek Generating Station's (WCGS) fifteenth refueling outage (RF15) that concluded on November 10, 2006. RF15 was the first outage of the first period of the third interval of the WCNOC Inservice Inspection Program. The enclosed report is submitted pursuant to the requirements of paragraph IWA-6240 of the 1998 Edition through 2000 Addenda of Section XI of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code.

There are no commitments contained within this letter.

If you have any questions concerning this matter, please contact me at (620) 364-4126 or Ms. Diane Hooper at (620) 364-4041.

Very truly yours,

A handwritten signature in dark ink, appearing to read "Kevin J. Moles". The signature is fluid and cursive, with a long horizontal flourish extending to the right.

Kevin J. Moles

KJM/rlt

A047

Enclosure

cc: J. N. Donohew (NRC), w/e
V. G. Gaddy (NRC) w/e
B. S. Mallett (NRC), w/e
Senior Resident Inspector (NRC), w/e

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number WCRE-16, 13-P1-RF-15

Owner Wolf Creek Nuclear Operating Corporation, 1550 Oxen Lane Northeast, Burlington, Kansas 66839
(Name and Address of Owner)

Plant Wolf Creek Generating Station, 1550 Oxen Lane Northeast, Burlington, Kansas 66839
(Name and Address of Plant)

Unit No. 1 Commercial service date 9-3-85 Refueling outage no. 15
(If applicable)

Current inspection interval 3rd
(1st. 2nd. 3rd. 4th. other)

Current inspection period 1st
(1st. 2nd. 3rd)

Edition and Addenda of Section XI applicable to the inspection plan 1998 edition through 2000 addenda

Date and revision of inspection plan WCRE-16 Rev. 1, dated 9-29-2006

Edition and Addenda of Section XI applicable to repairs and replacements, if different than the inspection plan same

CERTIFICATE OF CONFORMANCE

I certify that the statements made in this Owner's Activity Report are correct, and that the examinations, tests, repairs, replacements, evaluations, and corrective measures represented by this report conform to the requirements of Section XI.

Certificate of Authorization No. N/A Expiration Date N/A
(If applicable)

Signed Dennis E. Tougan Engineer Date January 25, 2007
Dennis E. Tougan
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Kansas and employed by The Hartford Steam Boiler Inspection and Insurance Company of Connecticut of Hartford, Connecticut have inspected the items described in this Owner's Activity Report, during the period September 3, 2005 to November 10, 2006, and state that to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests, repairs, replacements, evaluations and corrective measures described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Q. B. C. Full Commissions KS#594 A, N, I
Inspector's Signature National Board, State, Province, and Endorsements

Date 1/25/07

This form (E00127) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300

TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS
Completion of RF-15 (I-3, P-1)

Examination Category	Total Examinations Required For The Interval	Total Examinations Credited for This Period*	Total Examinations Credited (%) for The Period**	Total Examinations Credited (%) to Date for the Interval	Remarks
B-A	24	0	0	0	
B-B	5	1	20	20	
B-D	36	0	0	0	
B-F	Note 1	Note 1	Note 1	Note 1	Note 1
B-G-1	219	0	0	0	
B-G-2	15	0	0	0	Note 2
B-J	Note 1	Note 1	Note 1	Note 1	Note 1
B-K	1	0	0	0	
B-L-2	1	0	0	0	Note 3
B-M-2	7	1	14	14	Note 4
B-N-1	3	0	0	0	
B-N-2	6	0	0	0	
B-N-3	1	0	0	0	
B-O	4	0	0	0	
B-P	6	1	17	17	Note 5
B-Q	Note 6	Note 6	Note 6	Note 6	Note 6
C-A	11	0	0	0	
C-B	5	0	0	0	
C-C	21	2	9	9	
C-D	1	0	0	0	
C-F-1	Note 1	Note 1	Note 1	Note 1	Note 1
C-F-2	Note 1	Note 1	Note 1	Note 1	Note 1
C-G	5	0	0	0	
C-H	114	12	11	11	Note 7
D-A	44	1	2	2	
D-B	96	5	5	5	Note 8
F-A	279	22	8	8	
R-A	132	10	8	8	Note 9

* This column is interpreted to represent the cumulative number of exams performed to date in this period.

** This column is interpreted to represent the cumulative percentage of scheduled exams for the interval which have completed in this period to date.

- Note 1: Categories B-F, B-J, C-F-1, and C-F-2 welds are now examined under the RI-ISI program. These welds are listed under Category R-A.
- Note 2: For pumps and valves, examinations are limited to components selected for examination under Examination Categories B-L-2 and B-M-2.
- Note 3: Examination is required only when a pump is disassembled for maintenance, repair or examination.
- Note 4: Examination is required only when a valve is disassembled for maintenance, repair or examination.
- Note 5: All Category B-P components receive a visual examination (VT-2) each refueling outage.
- Note 6: Steam Generator tubing results are reported in accordance with Plant Technical Specifications. See Attachment 1 for a summary of the Steam Generator tubing results.
- Note 7: All Category C-H components receive a visual examination (VT-2) each period.
- Note 8: All Category D-B components receive a visual examination (VT-2) each period.
- Note 9: The RI-ISI program is implemented by 10 CFR 50.55(a) request I3R-01. As of this date, NRC approval of this request has not been received. These examinations are not to be considered final until NRC approval of this request is received.

TABLE 2
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT
REQUIRED EVALUATION FOR CONTINUED SERVICE

Examination Category	Item Number	Item Description	Flaw Characterization (IWA-3300)	Flaw or Relevant Condition Found During Scheduled Section XI Examination or Test (Yes or No)
There were no components containing flaws or relevant conditions that required an evaluation to determine acceptability for continued service.				

TABLE 3
ABSTRACT OF REPAIRS, REPLACEMENTS, OR CORRECTIVE MEASURES
REQUIRED FOR CONTINUED SERVICE

Code Class	Repair, Replacement, or Corrective Measure	Item Description	Description of Work	Flaw or Relevant Condition Found During Scheduled Section XI Examination or Test (Yes/No)	Date Complete	Repair/ Replacement Plan Number
3	Repair/ Replacement	EDG "A" Crankcase Eductor Duct Angle Support Bracket, (Dwg/Part# 118704049), KKJ01A	Fabricate and install a missing support bracket (Dwg/Part# 11874049) for the Crankcase Vacuum Ejector for KKJ01A.	No	01/26/2006	2004-003
3	Repair/ Replacement	EDG "B" Crankcase Eductor Duct Angle Support Bracket, (Dwg/Part# 118704049), KKJ01B	Fabricate and install a missing support bracket (Dwg/Part# 11874049) for the Crankcase Vacuum Ejector for KKJ01B.	No	08/29/2006	2004-004
3	Repair/ Replacement	Carrier Type 28NW Room Cooler Coil, SGL09A	Remove H-Bends to allow installing jumper from Row 48 Tube 2 to Row 44 Tube 4 to remove tube at Row 47 Tube 3 from service. Replace tube nut at Row 37 Tube 2.	No	01/03/2007	2006-008
3	Repair/ Replacement	Carrier Type 28NW Room Cooler Coil, SGL12A	Remove H-Bends to allow installing a jumper from Row 64 Tube 2 to Row 62 Tube 3 to remove Row 64 Tube 3 from service. Cut tubes at header for Row 55 Tube 1 and Row 48 Tube 4 and install caps to remove the entire circuit containing Row 55 Tube 1 from service.	No	05/04/2006	2006-023
3	Repair/ Replacement	Daniel International, Bergen-Paterson, Pacific Scientific Snubber Pipe Support, EG07-R014/132	During implementation of DCP# 10355 for Snubber Reduction, one snubber removed from EG07R014132 failed to pass the hand stroke test when removed and required an evaluation which was documented on CCP 12241.	No	12/15/2006	2006-031
1	Repair/ Replacement	Pressurizer Vertical Tank, TBB03	Performed full structural repair weld overlays on TBB03 Nozzle To Safe End Dissimilar Metal Welds on Surge, Relief, And "C" Safety Nozzles in accordance with CCP 11977.	Yes	See Note A	2006-043
3	Repair/ Replacement	Containment Air Cooler, SGN01B	Remove one leaking cooling coil, S/N# 906897-A5, from service by cutting off nozzles and blind flanging the supply and return header connections per DCP 07111.	No	12/08/2006	2006-064

Note A – Although all work and reviews have been completed, certification of this repair/replacement activity is not complete. Certification is not complete because WCNOG has not yet received the NRC Safety Evaluation for Relief Request I3R-05. NRC verbal approval of the Relief Request was received by WCNOG on October 6, 2006.

ATTACHMENT 1 **SUMMARY OF RF15 STEAM GENERATOR TUBING (CATEGORY B-Q) RESULTS**

Eddy Current Inspections were performed on S/Gs A and D. The following inspection programs were completed:

- 1) A Full Length Bobbin probe inspection of 100% of the tubes in both S/G A and S/G D was performed except for the U-bend portion of the rows 1-4.
- 2) A 55% Rotating Pancake Coil (RPC) Hot Leg Top of Tubesheet inspection (+ and - 3") in S/G's A and D was performed. This inspection included the peripheral tubes to inspect for possible loose parts.
- 3) A 100% RPC Cold Leg Top of Tubesheet Peripheral (2 rows deep) inspection (+ and - 3") in S/G's A and D was performed to inspect for possible loose parts.
- 4) A 50% RPC of tubesheet expansion zone bulges and over expansion in S/G's A & D Tubesheet (+3" and -17") was performed. This sample was concentrated in the upper tubesheet region (+3" to -10").
- 5) A Special Interest RPC program included all bobbin "I" codes (signals that were new or changed after history review). The inspection of 5.00 volts or greater dents on the hot leg side including the u-bend in the current 90 EFPM inspection cycle was completed. A 5% sample of the total population of bobbin indications that have not changed since the prior inspection ("H" and "S" codes) was inspected by RPC. The sample was performed on the hot leg only.

As a result of the inspection, four tubes in S/G A and seventeen tubes in S/G D were plugged due to wear at Anti-Vibration Bar locations. Two tubes in S/G A with geometry signals at the tubesheet transition were preventively plugged.

No cracklike indications were found during the inspection.

The inspection results are reported to the Nuclear Regulatory Commission in accordance with Technical Specification 5.6.10.