



U.S. NUCLEAR REGULATORY COMMISSION

STANDARD REVIEW PLAN

BRANCH TECHNICAL POSITION 7-3

GUIDANCE ON PROTECTION SYSTEM TRIP POINT CHANGES FOR OPERATION WITH REACTOR COOLANT PUMPS OUT OF SERVICE

REVIEW RESPONSIBILITIES

Primary - Organization responsible for the review of instrumentation and controls

Secondary - None

A. BACKGROUND

10CFR 50.55a(h), "Protection and Safety Systems," requires compliance with IEEE Std. 603-1991, "IEEE Standard Criteria for Safety Systems for Nuclear Power Generating Stations," and the correction sheet dated January 30, 1995. For nuclear power plants with construction permits issued before January 1, 1971, the applicant/licensee may elect to comply instead with their plant specific licensing basis. For nuclear power plants with construction permits issued between January 1, 1971, and May 13, 1999, the applicant/licensee may elect to comply instead with the requirements stated in IEEE Std. 279-1971, "Criteria for Protection Systems for Nuclear Power Generating Stations."

IEEE Std. 279-1971, Clause 4.15, and IEEE Std. 603-1991, Clause 6.8.2, include requirements that when a reactor operation requires a more restrictive setpoint, the means for ensuring the

Rev. 5 - [Month] 2007

USNRC STANDARD REVIEW PLAN

This Standard Review Plan, NUREG-0800, has been prepared to establish criteria that the U.S. Nuclear Regulatory Commission staff responsible for the review of applications to construct and operate nuclear power plants intends to use in evaluating whether an applicant/licensee meets the NRC's regulations. The Standard Review Plan is not a substitute for the NRC's regulations, and compliance with it is not required. However, an applicant is required to identify differences between the design features, analytical techniques, and procedural measures proposed for its facility and the SRP acceptance criteria and evaluate how the proposed alternatives to the SRP acceptance criteria provide an acceptable method of complying with the NRC regulations.

The standard review plan sections are numbered in accordance with corresponding sections in the Regulatory Guide 1.70, "Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (LWR Edition)." Not all sections of the standard format have a corresponding review plan section. The SRP sections applicable to a combined license application for a new light-water reactor (LWR) will be based on Regulatory Guide 1.206, "Combined License Applications for Nuclear Power Plants (LWR Edition)," until the SRP itself is updated.

These documents are made available to the public as part of the NRC's policy to inform the nuclear industry and the general public of regulatory procedures and policies. Individual sections of NUREG-0800 will be revised periodically, as appropriate, to accommodate comments and to reflect new information and experience. Comments may be submitted electronically by email to NRR_SRP@nrc.gov.

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use of the more restrictive setpoint shall be positive and must meet the other requirements of IEEE Std. 279-1971 or IEEE Std. 603-1991.

Before the development of IEEE Std. 279-1971 the Staff required automatic adjustment to more restrictive settings of trips affecting reactor safety by means of circuits satisfying the single-failure criterion. The basis for this requirement is that the function can be accomplished more reliably by automatic circuitry than by a human operator. This design practice, which has also been adopted independently by the national laboratories and by much of industry, served as the basis for Clause 4.15 of IEEE Std. 279-1971 and Clause 6.8.2 of IEEE Std. 603-1991.

More recently, applicants stated that their protection systems were designed to meet IEEE Std. 279-1971 or IEEE Std. 603-1991. A number of designs were proposed and accepted which reliably and simply satisfy this requirement. During the review of some applications, however, certain design deficiencies were found. The purpose of this position is to provide additional guidance on the application of Clause 4.15 of IEEE Std. 279-1971 and Clause 6.8.2 of IEEE Std. 603-1991.

B. BRANCH TECHNICAL POSITION

1. If more restrictive safety trip points are required for operation with a reactor coolant pump out of service, and if operation with a reactor coolant pump out of service is of sufficient likelihood to be a planned mode of operation, the change to the more restrictive trip points should be accomplished automatically.
2. Plants with designs not in accordance with the above should have included in the plant technical specifications a requirement that the reactor be shut down prior to changing the set points manually.

C. REFERENCES

1. IEEE Std. 279-1971, "Criteria for Protection Systems for Nuclear Power Generating Stations."
2. IEEE Std. 603-1991, "IEEE Standard Criteria for Safety Systems for Nuclear Power Generating Stations."

Branch Technical Position 7-3
Description of Changes

This Branch Technical Position updates the guidance previously provided in Revision 4, dated June 1997 of this Branch Technical Position. See ADAMS accession number ML052500524.

In addition this Branch Technical Position was administratively updated in accordance with NRR Office Instruction, LIC-200, Revision 1, "Standard Review Plan (SRP) Process."

The technical changes are incorporated in Revision 5, dated [Month] 2007:

Review Responsibilities - Reflects changes in review branches resulting from reorganization and branch consolidation. Change is reflected throughout the Branch Technical Position.

A. BACKGROUND

1. Updated to reflect the 1999 revision of 10 CFR 50.55a(h), "Protection and Safety Systems."

B. BRANCH TECHNICAL POSITION

None.

C. REFERENCES

None.