### **EXAMINATION OUTLINE SUBMITTAL**

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# FOR THE LASALLE INITIAL EXAMINATION - NOVEMBER 2006

## **Examination Outline Quality Checklist**

Facility:	LaSalle County Station U1/U2 Date of Examination: No	vembe	or 13, 2	2006
Item	Task Description		Initial	
1.	a. Verify that the outline(s) fit(s) the appropriate model, in accordance with ES-401.	a	b*	C#
W R		ja_	NA	p.v.
	b. Assess whether the outline was systematically and randomly prepared in accordance with Section D.1 of ES-401 and whether all K/A categories are appropriately sampled.	Z	NA	m
T	c. Assess whether the outline over-emphasizes any systems, evolutions, or generic topics.	Z	NA	m
E N	d. Assess whether the justifications for deselected or rejected K/A statements are appropriate.	R	NA	bin
2. S	a. Using Form ES-301-5, verify that the proposed scenario sets cover the required number of normal evolutions, instrument and component failures, technical specifications, and major transients.	R	»/A	bin
M U L A T	b. Assess whether there are enough scenario sets (and spares) to test the projected number and mix of applicants in accordance with the expected crew composition and rotation schedule without compromising exam integrity, and ensure that each applicant can be tested using at least one new or significantly modified scenario, that no scenarios are duplicated from the applicants' audit test(s), and that scenarios will not be repeated on subsequent days.	æ.	N/A	den.
O R	c. To the extent possible, assess whether the outline(s) conform(s) with the qualitative and quantitative criteria specified on Form ES-301-4 and described in Appendix D.	R	Alk	km
3. W / T	<ul> <li>a. Verify that the systems walk-through outline meets the criteria specified on Form ES-301-2:</li> <li>(1) the outline(s) contain(s) the required number of control room and in-plant tasks distributed among the safety functions as specified on the form</li> <li>(2) task repetition from the last two NRC examinations is within the limits specified on the form</li> <li>(3) no tasks are duplicated from the applicants' audit test(s)</li> <li>(4) the number of new or modified tasks meets or exceeds the minimums specified on the form</li> <li>(5) the number of alternate path, low-power, emergency, and RCA tasks meet the criteria on the form.</li> </ul>	ΰZ	<i>ч </i> А	jam.
	<ul> <li>b. Verify that the administrative outline meets the criteria specified on Form ES-301-1:</li> <li>(1) the tasks are distributed among the topics as specified on the form</li> <li>(2) at least one task is new or significantly modified</li> <li>(3) no more than one task is repeated from the last two NRC licensing examinations</li> </ul>	Z	~ <i> </i> A	8
	c. Determine if there are enough different outlines to test the projected number and mix of applicants and ensure that no items are duplicated on subsequent days.	20	NIA	m
4.	<ul> <li>Assess whether plant-specific priorities (including PRA and IPE insights) are covered in the appropriate exam sections.</li> </ul>	Z	ri/a	sm
G	b. Assess whether the 10 CFR 55.41/43 and 55.45 sampling is appropriate.	2	N/A	on
N	c. Ensure that K/A importance ratings (except for plant-specific priorities) are at least 2.5.	Z	A JA	om
ER	d. Check for duplication and overlap among exam sections.	Z	4/A	)m
A	e. Check the entire exam for balance of coverage.	JZ.	NIA	5m
	f. Assess whether the exam fits the appropriate job level (RO or SRO).	JZ	NIA	Men
a. Auth	or <u>David W. Reeser / David W.</u>		9/1 9/1	1 1αρ
c. NRC	Chief Examiner (#) Dell R. McNeil ALLR. Muthol Supervisor Hironori Peterson		9/1. 2/1	100
Note:	# Independent NRC reviewer initial items in Column "c"; chief examiner concurrence rec	uired.		

### Administrative Topics Outline

Facility: <u>LaSalle Co. Stati</u> Examination Level (circle on		Date of Examination: <u>November 2006</u> Operating Test Number: <u>2006301</u>
Administrative Topic (see Note)	Type Code*	Describe activity to be performed
Conduct of Operations	NS	RO/SRO - Using plant electrical drawings determine the impact of pulling a relay. (2.1.24)
Conduct of Operations	MS	RO/SRO - Evaluate plant conditions following a power increase to determine if any limits have been exceeded. (2.1.32)
Equipment Control	DS	RO/SRO - Perform Technical Specification Weekly Offsite Power Lineup Verification. (2.2.12)
Radiation Control	N Plant	SRO - Initiate surveillance LOS-PC-M1, Primary Containment Integrity for Modes 1, 2, & 3 (2.3.11)
Emergency Plan	DS	SRO - Detemine whether KI should be issued to emergency workers. [EP-AA-112-100-F-01 & EP-AA-113] (2.4.40)
		ROs. RO applicants require only 4 items unless they are ics, when all 5 are required.
• Type Codes & Criteria:	(N)ew or (	om bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) M)odified from bank (≥ 1) 2 exams (≤ 1; randomly selected)

#### Control Room/In-Plant Systems Outline

	/: <u>LaSalle Co. Station <del>U1/U2</del> Level (circle one): RO SRO-I / SRO-U</u>			vember 2006 006301
Contro	NRoom Systems <sup>®</sup> (8 for RO; 7 for SRO-1; 2 or 3	3 for SRO-U, including	1 ESF)	
	System / JPM Title		Type Code*	Safety Function
a.	Reactor Manual Control/Respond to Loss of	RPIS Displays	N, S	1
b.	High Pressure Core Spray/Manually start and	l lineup for injection	A, D, S	2
C.	Safety Relief Valves/Close a stuck open SRV	/ by removing fuses.	D, S	3
d.	Main and Reheat Steam/Re-pressurize MSL inadvertent isolation.	following	L, N, S	4
e.	RHR/LPCI: Containment Spray Mode/Shift front to Suppression Pool Cooling Mode	om DW Spray Mode	A, N, S	5
f.	Rod Worth Minimizer/Perform RWM Operabi decreasing power below 10% in MODE 1	lity Check for	A, L, N, S	7
g.	Control Room HVAC/Emergency Makeup Un	iit Startup	N, S	9
h.	n/a		n/a	n/a
In-Plar	nt Systems <sup>@</sup> (3 for RO; 3 for SRO-I; 3 or 2 for S	RO-U)		
i.	Off-gas/SJAE Pressure Control Using Manua	al Bypass Valve	E, N, R	9
j.	Uninterruptible Power Supply/CX Inverter Sta	artup	N, R	6
k.	Instrument Air/Respond to Air Compressor ru to excessive surge condition.	inning unloaded due	E, N, R	8
@	All control room (and in-plant) systems must be d and functions may overlap those tested in the cor		ent safety functions;	in-plant systems
	Type Codes	Criteria f	for RO/SRO-I/SF	₹0-∪
(C)ontro (D)irect (E)merg (L)ow-F (N)ew c	nate path ol room t from bank gency or abnormal in-plant Power / Shutdown or (M)odified from bank including 1(A) ous 2 exams	≤ 3 / ≤ 3	$4-6 / 4-6 / 2-3$ $\leq 9 / \leq 8 / \leq 4$ $\geq 1 / \geq 1 / \geq 1$ $\geq 1 / \geq 1 / \geq 1$ $\geq 2 / \geq 2 / \geq 1$ $/ \leq 2 \text{ (randomly sele}$ $\geq 1 / \geq 1 / \geq 1$	ected)

**Control Room/In-Plant Systems Outline** 

ES-301

Facility: LaSalle Co. Station U1/U2 Exam Level (circle one): RO / SRO-I SRO-U	Date of Exa Operating		ember 13, 2006 6301
Control Room Systems <sup>@</sup> (8 for RO; 7 for SRO-I; 2	or 3 for SRO-U, incl	uding 1 ESF)	
System / JPM Title		Type Code*	Safety Function
a. n/a			
<ul> <li>b. High Pressure Core Spray/Manually start a injection</li> </ul>	and lineup for	A, D, S	2
c. Safety Relief Valves/Close a stuck open S fuses.	RV by removing	D, S	3
d. n/a			
e. n/a			
f. Rod Worth Minimizer/Perform RWM Oper decreasing power below 10% in MODE 1	ability Check for	A, L, N, S	7
g.			
h.			
In-Plant Systems <sup>@</sup> (3 for RO; 3 for SRO-I; 3 or 2 fo	or SRO-U)		
i. Off-gas/SJAE Pressure Control Using Manual B	lypass Valve	E, N, R	9
j. Uninterruptible Power Supply/CX Inverter Startu	р	N, R	6
k.			
All control room (and in-plant) systems must b in-plant systems and functions may overlap the			tions;
* Type Codes	Criteria f	or RO/SRO-1/S	RO-U
<ul> <li>(A)Iternate path</li> <li>(C)ontrol room</li> <li>(D)irect from bank</li> <li>(E)mergency or abnormal in-plant</li> <li>(L)ow-Power / Shutdown</li> <li>(N)ew or (M)odified from bank including 1(A)</li> <li>(P)revious 2 exams</li> <li>(R)CA</li> <li>(S)imulator</li> </ul>	≤ 3 / ≤ 3 /	$4-6/4-6/2-3$ $\leq 9/\leq 8/\leq 4$ $\geq 1/\geq 1/\geq 1$ $\geq 1/\geq 1/\geq 1$ $\geq 2/\geq 2/\geq 1$ $\leq 2 \text{ (randomly sel}$ $\geq 1/\geq 1/\geq 1$	ected)

### **Transient and Event Checklist**

Facility:	LaSalle C	ounty	Station	<u>U1/U</u>	2	Date	of Exan	n: N	ovemb	er 9, 20	006 (	Operati	ng Tes	t No.	: 20	063	01
A	E							Sc	enario	s							
P P	V E		1		<u></u>	2		<u> </u>	3			4		Т	<u> </u>	м	
Ļ	N					د		}	5			<b>.</b>		0	}	1	
C	T		CREW			CREV			CREW			CREW		A	ļ	N I	1
A N	T Y				<u> </u>						ļ	OSITIC	T	L		M	
T	Р	S R	A T	B	S R	A T	B	S R	A T	B	S R	A T	B		1	U M(*	)
	E	0	С	Р	0	C	Р	0	С	P	0	С	Р		R	Ī	U
RO	RX	1	<b> </b>		<b> </b>	1			<u>}</u>				<u> </u>	2	1	1	0
	NOR		<u> </u>					<u> </u>	<u>†</u>			[	<u>├</u> ───	0	1	1	1
SRO-I SRO-U	I/C	4				4				3				1	4	4	2
, í t	MAJ	2				1				2			<u> </u>	5	2	2	1
	TS	3												3	0	2	2
RO	RX		1											1	1	1	0
SRO-I	NOR						1		ļ					1	1	1	1
	I/C		2				1	5						8	4	4	2
SRO-U	MAJ		2			<u> </u>	1	2	 					5	2	2	1
	TS				<u> </u>			3	<b>├</b>		ļ		ļ	3	0	2	2
RO	RX	 	 		1	<b> </b>		┣	<b> </b>	<u> </u>				1	1	1	0
SRO-I	NOR I/C	<u> </u>		2	1	┣───			2				<u> </u>	1	1	4	2
SRO-U	MAJ		<u> </u>	2	1				2					5	2	2	
	TS			-	2	┢				<u> </u>				2	0	2	2
RO	RX					┨╼━━			<u> </u>			<u> </u>			1	1	0
	NOR		'			<u> </u>									1	1	1
SRO-I	1/C								1						4	4	2
SRO-U	MAJ														2	2	1
L	TS														0	2	2
Instructio	ns:																
	Check the event type and "balan two instru	e; TS a nce-of-	re not a plant (l	applic BOP)"	able fo positio	or RO a ons; Ir	applicar istant S	nts. RO ROs m	Ds mus iust do	t serve one sc	in botl enario	n the "a , includ	at-the-c ling at l	ontro east	ols (/		
	Reactivity Section D evolutions	.5.d) b	ut mus	t be si	ignifica	int per	Section	n C.2.a	of App	pendix I	D. (*) I	Reactiv	ity and	norr	nal		
[[ t	Wheneve that requi the minim	e verif	iable a	ctions	that p	rovide	insight	to the	applica	ant's co	mpete	nce co	unt tow	ard			
L																	

#### **BWR Examination Outline**

Facility: LaSal	le						Dat	e of	Exar	n:	Nov	vemt	ber 2006					
Tion	Crown					RO P	(/A (	Categ	ory	Poin	ts	1			SF	20-0	nly Poi	nts
Tier	Group	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	A	2	G*		Total
1.	1	3	4	3				3	3			4	20	2	1		3	7
Emergency & Abnormal	2	1	1	1		N/A		2	1	N	/A	1	7	4	2		1	3
Plant Evolutions	Tier Totals	4	5	4				5	4			5	27	6	6		4	10
2.	1	3 3 3 2 2 2 2 2 2 2 3 26 3 2													5			
Plant Systems	2	1	0	2	1	2	1	1	1	1	1	1	12		2		1	3
	Tier Totals         4         3         5         3         4         3         3         3         3         4         38         5         3         8															8		
3. Generic	Knowledge and Abilities     1     2     3     4     10     1     2     3     4     7       Categories     Image: Categories     Imag															7		
	Categories         2         3         2         3         2         2         1         2																	
Note: 1.	and SRO-only of	Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" n each K/A category shall not be less than two).																
2.	The point total f The final point t The final RO ex	otal f	or ea	ch gr	oup	and	tier m	iay de	eviate	by ±	1 fro	m tha	at specified	d in the				C revisions.
3.	Systems/evoluti at the facility sh on the outline s of inappropriate	ould hould	be de I be a	eletec Iddec	l and I. R	d just	ified;	oper	ationa	ally in	nport	ant, s	site-specifi	ic syste	ems th	at are	not inclu	
4.	Select topics fro selecting a seco									poss	ible; s	samp	le every s	ystem	or evo	lution	in the g	roup before
e.	Absent a plant- Use the RO and													) of 2.5	or hig	her sh	all be s	elected.
6.	Select SRO top	ics fo	or Tie	rs 1 a	and 2	2 fron	n the	shad	led sy	/stem	is and	d K/A	categorie	s.				
7.*	The generic (G) must be relevan									from	Sect	tion 2	of the K//	A Cata	log, bu	ut the t	opics	
8.	On the following for the applicab for each catego SRO-only exam pages for RO a	le lice ory in n, ent	ense the ta er it c	level, able a on the	and bov e left	d the re; if f : side	point uel h	total andlii	s (#) t ng eq	for ea uipm	ich sy ent is	ystem s sam	n and cate	gory. her tha	Enter t n Cate	the gro egory A	oup and A2 or G*	tier totals
9.	For Tier 3, sele and point totals																	

ES-401 Emergency	and	Abr					nation Outline plutions - Tier 1/Group 1 (RO / SRO)	Form E	S-401-1
E/APE # / Name / Safety Function	K 1	K 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4				0 3			Ability to operate and/or monitor RMCS as it applies to	2.6	1/0
295003 Partial or Complete Loss of AC / 6					0 2		Ability to determine and/or interpret reactor power/pressure/and level as they apply to	4.2	1/0
295004 Partial or Total Loss of DC Pwr / 6						х	2.1.16 – Ability to operate the plant phone/paging system/and two way radio.	2.9	1/0
295005 Main Turbine Generator Trip / 3			0 6		0 1		K3.06 – Knowledge of the reasons for realignment of electrical distribution response as it applies to A2.01(SRO Only) – Ability to determine and/or interpret turbine speed as it applies to	3.3 2.7	1/1
295006 SCRAM / 1				0 5		×	A1.05 – Ability to operate and/or monitor the Neutron Monitoring System it applies to 2.4.27(SRO Only) – Knowledge of fire in the plant procedures.	4.2 3.5	1/1
295016 Control Room Abandonment / 7		0 1					Knowledge of the interrelations between and the Remote Shutdown Panel.	4.4	1/0
295018 Partial or Total Loss of CCW / 8		0 2					Knowledge of the interrelations between and plant operatons.	3.4	1/0
295019 Partial or Total Loss of Inst. Air / 8			0 2				Knowledge of the reasons for standby air compressor response as it applies to	3.5	1/0
295021 Loss of Shutdown Cooling / 4				0 1			Ability to operate and/or monitor the RWCU system as it applies to	3.4	1/0
295023 Refueling Acc / 8					0 1		Ability to determine and/or interpret area radiation levels as they apply to	3.6	1/0
295024 High Drywell Pressure / 5						х	2.4.11 – Knowledge of abnormal condition procedures.	3.4	1/0
295025 High Reactor Pressure / 3					0 6 / 0 4		A2.06 – Ability to determine and/or interpret reactor water level as it applies to A2.04(SRO Only) – Ability to determine and/or interpret suppression pool level as it applies to	3.7 3.9	1/1
295026 Suppression Pool High Water Temp. / 5						x	<ul> <li>2.2.11 – Knowledge of the process for controlling temporary changes.</li> <li>2.4.14(SRO Only) – Knowledge of general guidelines for EOP flowchart use.</li> </ul>	2.5 3.9	1/1
295027 High Containment Temperature / 5							SUPPRESSED – LaSalle has a Mark II Containment		
295028 High Drywell Temperature / 5					0 4	x	<ul> <li>2.1.1 – Knowledge of the conduct of operations.</li> <li>A2.04(SRO Only) – Ability to determine and/or interpret drywell pressure as it applies to</li> </ul>	3.7 4.2	1/1
295030 Low Suppression Pool Wtr Lvl / 5	0 2				0 4		K1.02 – Knowledge of the operational implications on pump NPSH as it applies to A2.04(SRO Only) – Ability to determine and/or interpret drywell/suppression chamber d/P as it applies to	3.5 3.7	1/1

ES-401 Emergency	Emergency and Abnormal Plant Evolutions - Tier 1/Group 1 (RO / SRO)														
E/APE # / Name / Safety Function	К 1	K 2	К 3		A 2	G	K/A Topic(s)	IR	#						
295031 Reactor Low Water Level / 2	0	0				×	<ul> <li>K1.01 – Knowledge of the operational implications on adequate core cooling as it applies to</li> <li>K2.01 – Knowledge of the interrelations between  and reactor water level indication.</li> <li>2.1.25(SRO Only) – Ability to obtain and interpret station reference material such as graphs/ monographs/and tables which contain performance data.</li> </ul>	4.6 4.4 3.1	2/1						
295037 SCRAM Condition Present and Power Above APRM Downscale or Unknown / 1	3.4	1/0													
295038 High Off-site Release Rate / 9		0 3					Knowledge of the interrelations between and the Post Accident Sample System.	3.8	1/0						
600000 Plant Fire On Site / 8			0 4				Knowledge of the reasons for the actions contained in the abnormal procedure for	2.8	1/0						
K/A Category Totals:	3	4	3	3	3 / 4	4 / 3	Group Point Total:	-	20/7						

ES-401 Emergency	/ and	d Ab	nor				ination Outline F volutions - Tier 1/Group 2 (RO / SRO)	orm ES	-401-1
E/APE # / Name / Safety Function	K 1	K 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
295002 Loss of Main Condenser Vac / 3									
295007 High Reactor Pressure / 3									
295008 High Reactor Water Level / 2									
295009 Low Reactor Water Level / 2									
295010 High Drywell Pressure / 5									
295011 High Containment Temp / 5							SUPPRESSED – LaSalle has a Mark II Containment		
295012 High Drywell Temperature / 5				0 2			Ability to operate and/or monitor the Drywell Cooling system as it applies to	3.8	1/0
295013 High Suppression Pool Temp. / 5					0 1		Ability to determine and/or interpret the following as they apply to HIGH SUPPRESSION POOL TEMPERATURE:	3.8	1/0
295014 Inadvertent Reactivity Addition / 1						х	2.2.33 – Knowledge of control rod programming.	2.5	1/0
295015 Incomplete SCRAM / 1					0 2		A2.02(SRO Only) – Ability to determine and/or interpret control rod position as it applies to	4.2	0/1
295017 High Off-site Release Rate / 9						х	2.4.6(SRO Only) – Knowledge of symptom-based EOP mitigation stategies.	4.0	0/1
295020 Inadvertent Cont. Isolation / 5 & 7	0 5						Knowledge of the operational implications of a loss of dywell/containment cooling as it applies to	3.3	1/0
295022 Loss of CRD Pumps / 1		0 7					Knowledge of the interrelations between and reactor pressure(SCRAM assist)	3.4	1/0
295029 High Suppression Pool Wtr Lvl / 5			0 1				Knowledge of the reasons for emergency depressurization as it applies to	3.5	1/0
295032 High Secondary Containment Area Temperature / 5				0 4			Ability to operate and/or monitor the leak detection systems/components as they it apply to	3.4	1/0
295033 High Secondary Containment Area Radiation Levels / 9									
295034 Secondary Containment Ventilation High Radiation / 9									
295035 Secondary Containment High Differential Pressure / 5					0 1		A2.01(SRO Only) – Ability to determine and/or interpret secondary containment pressure as it applies to	3.9	0/1
295036 Secondary Containment High Sump/Area Water Level / 5									
500000 High CTMT Hydrogen Conc. / 5									
K/A Category Point Totals:	1	1	1	2	1 / 2	1 / 1	Group Point Total:	<u> </u>	7/3

ES-401     BWR Examination Outline Plant Systems - Tier 2/Group 1 (RO / SRO)       System # / Name     K     K     K     K     A     A     A     A     G     K/A Topic(s)													Form ES	S-401-1
System # / Name											G	K/A Topic(s)	IR	#
203000 RHR/LPCI: Injection Mode		0 3								0 3		K2.03 – Knowledge of the electrical power supplies to the initiation logic. A4.03 – Ability to manually operate and/or monitor the keep fill system.	2.7 3.4	2/0
205000 Shutdown Cooling			0 1								x	<ul> <li>K3.01 – Knowledge of the effect that a loss or malfunction of the will have on reactor pressure.</li> <li>2.2.1 – Ability to perform pre-startup procedures including those controls that could affect reactivity.</li> </ul>	3.3 3.7	2/0
206000 HPCI												SUPPRESSED – LaSalle does not have a HPCI system.		
207000 Isolation (Emergency) Condenser												SUPPRESSED – LaSalle does not have an Isolation Condenser		
209001 LPCS				0 8				02				K4.08 – Knowledge of design feature(s) and/or interlocks which provide for automatic system initiation. A2.02(SRO Only) – Ability to predict the impacts of valve closures on the; and use procedures to correct,control, or mitigate the consequences.	3.8 3.2	1/1
209002 HPCS					0 1							Knowledge of the operational implications of an indication of pump cavitation as it applies to	2.5	1/0
211000 SLC						0 1						Knowledge of the effect that a loss or malfunction of the plant air systems will have on the Plant Obj. # 028.00.16	2.4	1/0
212000 RPS							0 3					Ability to predict and/or monitor changes in RPS motor-generator output frequency. Plant Obj. # 049.00.05/06	2.4	1/0
215003 IRM								0 5				Ability to predict the impacts of faulty or erratic operation of detectors/system on the; and use procedures to correct,control, or mitigate the consequences.	3.3	1/0
215004 Source Range Monitor									0 2			Ability to monitor automatic operation of the including annunciator and alarm signals.	3.4	1/0
215005 APRM / LPRM										0 6		Ability to manually operate and/or monitor verification of proper functioning/operability.	3.6	1/0
217000 RCIC											х	<ol> <li>2.1.30 – Ability to locate and operate components/including local controls.</li> </ol>	3.9	1/0
218000 ADS	0 4							0 4				K1.04 – Knowledge of the physical connections and/or cause-effect relationships between and drywell/containment pressure. A2.04(SRO Only) – Ability to predict the impacts of an system failure to initiate; and use procedures to correct, control, or mitigate the consequences.	3.9 4.2	1/1

ES-401				F	Plant							Outline p 1 (RO / SRO)	Form ES	6-401-1
System # / Name	K 1	K 2	К 3	K 4	K 5	K 6		A 2	A 3	A 4	G	K/A Topic(s)	IR	#
223002 PCIS/Nuclear Steam Supply Shutoff		0									х	K2.01 – Knowledge of the electrical power supplies to the logic power supplies. Plant Obj # 091.00.16 2.2.2(SRO Only) – Ability to manipulate the controls as required to operate between shutdown and designated power levels.	2.4 3.5	1/1
239002 SRVs	0 8		0 1									K1.08 – Knowledge of the physical connections and/or cause-effect relationships between and ADS. K3.01 – Knowledge of the effect that a loss or malfunction of the will have on reactor pressure control.	4.0 3.9	2/0
259002 Reactor Water Level Control		0 1										Knowledge of the electrical power supplies to reactor water level control system circuits Plant Obj # 078.00.16	2.4	1/0
261000 SGTS			0 5									Knowledge of the effect that a loss or malfunction of will have on secondary containment radiation/ contamination levels.	3.2	1/0
262001 AC Electrical Distribution				0 1								Knowledge of design feature(s) and/or interlocks which provide for bus lockouts.	3.0	1/0
262002 UPS (AC/DC)						0 2						Knowledge of the effect that a loss or malfunction of D.C. power will have on	2.8	1/0
263000 DC Electrical Distribution					0 1						x	K5.01 – Knowledge of the operational implications of Hydrogen generation during battery charging as it applies to 2.4.26 – Knowledge of facility protection requirements including fire brigade and portable fire fighting equipment usage.	2.6 2.9	2/0
264000 EDGs							0 3	0 1				A1.03 Ability to predict and/or monitor changes in operating parameters. A2.01(SRO Only) – Ability to predict the impacts of parallel generator operation on the; and use procedures to correct, control, or mitigate the consequences.	2.8 3.5	1/1
300000 Instrument Air								0 1			x	A2.01 – Ability to predict the impacts of air dryer and filter malfunctions on the; and use procedures to correct, control, or mitigate the consequences. 2.4.8(SRO Only) – Knowledge of how event- based emergency/abnormal operating procedures are used in conjunction with symptom-based EOPs.	2.9 3.7	1/1
400000 Component Cooling Water	02								0			K1.02 – Knowledge of the physical connections and/or cause-effect relationships between and loads cooled. A3.01 – Ability to monitor automatic operation of the including setpoints on instrument signal levels for normal ops, warnings, and trips that are applicable to	3.2 3.0	2/0
K/A Category Point Totals:	3	3	3	2	2	2	2	2 / 3	2	2	3 / 2	Group Point Total:		26/5

ES-401			F	Plan				xami Tier				ine (RO / SRO)	Form E	S-401-1
System # / Name	K 1	K 2					A 1	A 2	A 3		G		IR	#
201001 CRD Hydraulic														
201002 RMCS														
201003 Control Rod and Drive Mechanism														
201004 RSCS												SUPPRESSED – RSCS has been abandoned in place at LaSalle.		
201005 RCIS												SUPPRESSED – LaSalle does not have RCIS.		
201006 RWM														
202001 Recirculation														
202002 Recirculation Flow Control														
204000 RWCU														
214000 RPIS					0 1							Knowledge of the operational implications of reed switches as they applies to	2.7	1/0
215001 Traversing In-core Probe			0 1									Knowledge of the effect that a loss or malfunction of the will have on Loca Power Range Monitor Calibration Plant Obj. # 046.00.18	2.3	1/0
215002 RBM				0 3								Knowledge of design feature(s) and/or interlocks which provide for initiation point (30%)	2.9	1/0
216000 Nuclear Boiler Inst.					1 0							Knowledge of the operational implications of vessel level measurement as it applies to	3.1	1/0
219000 RHR/LPCI: Torus/Pool Cooling Mode						0 8						Knowledge of the effect that a loss or malfunction of ECCS Room Cooling will have on	2.7	1/0
223001 Primary CTMT and Aux.							0 2					Ability to predict and/or monitor changes in Drywell pressure associated with operation of	3.6	1/0
226001 RHR/LPCI: CTMT Spray Mode								1 0				Ability to predict the impacts of Nuclear Boiler Instrument failures on the; and use procedures to correct,control, or mitigate the consequences.	· 3.0	1/0
230000 RHR/LPCI: Torus/Pool Spray Mode									0 1			Ability to monitor automatic operation of the including valve operation.	3.4	1/0
233000 Fuel Pool Cooling/Cleanup										1 0		Ability to manually operate and/or monitor tank levels.	2.5	1/0
234000 Fuel Handling Equipment											×	2.2.26(SRO Only) – Knowledge of refueling administrative requirements.	3.7	0/1
239001 Main and Reheat Steam								0				A2.01(SRO Only) – Ability to predict the impacts of a malfunction of reactor turbine pressure regulating system on the; and use procedures to correct,control, or mitigate the consequences.	3.9	0/1

ES-401 BWR Examination Outline						Form ES-401-								
Plant Systems - Tier 2/Group 2 (RO / SRO)														
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
239003 MSIV Leakage Control												SUPPRESSED – MSIV Leakage Control abandoned in place at LaSalle.		
241000 Reactor/Turbine Pressure Regulator											x	2.1.7 – Ability to evaluate plant performance and make operational judgements based on operating characteristics/reactor behavior/and instrument interpretation.	3.7	1/0
245000 Main Turbine Gen. / Aux.	0 6											Knowledge of the physical connections and/or cause-effect relationships between and component cooling water system.	2.6	1/0
256000 Reactor Condensate			0 2									Knowledge of the effect that a loss or malfunction of the will have on CRD Hydraulic system.	3.2	1/0
259001 Reactor Feedwater								0 9				A2.09(SRO Only) – Ability to predict the impacts of TDRFP steam inlet pressure flow on the; and use procedures to correct,control, or mitigate the consequences.	2.6	0/1
268000 Radwaste														
271000 Offgas														
272000 Radiation Monitoring														
286000 Fire Protection													_	
288000 Plant Ventilation													_	
290001 Secondary CTMT													_	
290003 Control Room HVAC													_	
290002 Reactor Vessel Internals														
K/A Category Point Totals:	1	0	2	1	2	1	1	1 / 2	1	1	1 / 1	Group Point Total:		12/3

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Facility: L	aSalle Co	ounty Station U1/U2 Date of I	Exam:	Nov	ember 2	2006
Category	K/A #	Topic	R	0	SRO-Only	
			IR	#	IR	#
	2.1.1	Conduct of operations requirements.	3.7	1	ļ	ļ
1.	2.1.17	Make accurate/clear and concise verbal reports.	3.5	1	ļ	
Conduct of Operations	2.1.11	Knowledge of less than one hour technical specification action statements for systems.			3.8	1
	2.1.34	Maintain primary and secondary plant chemistry within allowable limits.			2.9	1
·	Subtotal			2		2
	2.2.12	Surveillance procedures.	3.0	1		
	2.2.13	Tagging and clearance procedures.	3.6	1		
2.	2.2.33	Control rod programming	2.5	1		
Equipment Control	2.2.9	Knowledge of the process for determining if the proposed change / test or experiment increases the probability of occurrence of an accident during the change / test or experiment.			3.3	1
	2.2.29	SRO fuel handling responsibilities.			3.8	1
<u> </u>	Subtotal			3		2
	2.3.1	10 CFR 20 and related facility radiation controls.	2.6	1		[
3.	2.3.11	Ability to control radiation releases.	2.7	1		
Radiation Control	2.3.10	Perform procedures to reduce excessive levels of radiation and guard against personnel exposure.			3.3	1
	Subtotal			2		1
	2.4.20	Knowledge of operational implications of EOP warnings / cautions / and notes.	3.1	1		
4. Emergency Procedures / Plan	2.4.46	Verify that the alarms are consistent with plant conditions.	3.3	1		
	2.4.48	Interpret control room indications to verify the statusunderstandaffect plant and system conditions.	3.5	1		
	2.4.18	Knowledge for the specific basis for EOPs.			3.6	1
	2.4.30	Knowledge of which events related to system operations/status should be reported to outside agencies.			3.6	1
	Subtotal			3		2
Tier 3 Point Tot				10		7

Initial Conditions:       100% RTP; RCIC is OOS, day 3 of a s         Turnover:       Maintain power at 100% RTP except as required to service early next shift.         Event       Maiff.       Event         No.       Type*       D         1       I (TS)       Perform TSV Scram and Ror more valves will fail to r         2       R       Commence 200 Mwe Loar         3       C (TS)       During load decrease one valve continues to close u         4       M (TS)       Degraded off-site power s         5       I/C       Full Core Display fails (du use alternate methods to represent to the represen	Form ES	Scenario Ou		)	ppendix [
Turnover: Maintain power at 100% RTP except as required to service early next shift.         Event No.       Malf.       Event Type*       D         1       I (TS) BOP/SRO       Perform TSV Scram and B or more valves will fail to r TS evaluation.         2       R RO/SRO       Commence 200 Mwe Loar Social decrease one valve continues to close u Resultant flow mismatch valve contin	Op-Test No.: <u>2006-0</u> ors:				
Functional Test.RCIC is OOS, day 3 of a scheduled 3 da returned to service early next shiftEvent No.Malf.Event Type*1I (TS) 	cheduled 3 day outage.	P; RCIC is OOS, day	100% RTP	nditions: _	Initial Co
No.No.Type*D1I (TS) BOP/SROPerform TSV Scram and B or more valves will fail to r TS evaluation.2R RO/SROCommence 200 Mwe Loa3C (TS) RO/SRODuring load decrease one valve continues to close u Resultant flow mismatch v denerator trips. Bus 141 open, loss of power to bus4M (TS) ALLDegraded off-site power s Generator trips. Bus 141 open, loss of power to bus5I/C RO/SROFull Core Display fails (du use alternate methods to r bus 142Y.7MStation Blackout.		S, day 3 of a schedu	CIC is OOS	al Test. R	Functiona
BOP/SROor more valves will fail to r TS evaluation.2R RO/SROCommence 200 Mwe Loa Commence 200 Mwe Loa3C (TS) RO/SRODuring load decrease one valve continues to close u Resultant flow mismatch v Generator trips. Bus 141 open, loss of power to bus4M (TS) ALLDegraded off-site power s Generator trips. Bus 141 open, loss of power to bus5I/C RO/SROFull Core Display fails (du use alternate methods to bus bus 142Y.7MStation Blackout.	Event escription				
RO/SRO         3       C (TS) RO/SRO       During load decrease one valve continues to close u Resultant flow mismatch v Resultant flow mismatch v Resu	OC-RPT Functional Test. One neet acceptance criteria requirin	or more valves wi			1
RO/ŠRO       valve continues to close u         Resultant flow mismatch v         Generator trips. Bus 141         open, loss of power to bus         Ro/SRO         Full Core Display fails (du         use alternate methods to the         BOP/SRO         BOP/SRO         M         Station Blackout.	Decrease.	Commence 200 N			2
ALL       Generator trips. Bus 141 open, loss of power to bus         5       I/C         Full Core Display fails (du use alternate methods to box)         6       C         BOP/SRO       Degraded/Loss of cooling bus 142Y.         7       M		valve continues to			3
RO/SRO     use alternate methods to       6     C     Degraded/Loss of cooling bus 142Y.       7     M     Station Blackout.	upplies leading to LOOP and Ma normal feed breaker fails to 141Y.	Generator trips.			4
BOP/SRO         bus 142Y.           7         M         Station Blackout.	e to loss of power) requiring RO letermine shutdown status.				5
	to Division 2 DG results in loss				6
		Station Blackout.			7

\* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Appendix [	D		Scenario Outline	Form ES-D-1					
Facility: _ Examine			Scenario No.: 2 Op-Tes Operators:	st No.: <u>2006-301</u>					
	nditions: _ DRFP_is O		te 30% RTP, both RR Pumps in slow spee	d with FCVs full					
leaks. A	pproximate	30% RTP	progress (step E2.9 of LGP-2-1) to repair co , both RR Pumps in slow speed with FCVs n. Two TDRFPs are in service still.						
Event No.	Malf. No.	Event Type*	Event Description						
1		R RO/SRO	Insert control rods until generator output is approx. 60 MWe						
2		I (TS) RO/SRO	TCV & TSV Scram Bypass Relay energizes prematurely while greater than 25% Power.						
3		N BOP/SRO	Remove one TDRFP from service and en single element control.	sure transfer to					
4		l (TS) ro/sro	Two or more IRMs remain greater than 50 10 after IRM detectors are inserted.	0% scale on range					
5		I/C RO/SRO	Operating feedpump controller begins to manual control or shift to other TDRFP.	oscillate requiring					
6		MALL	Partial ATWS (several control rods fail to	insert).					
7		C BOP/SRO	Main Turbine will not trip from the MCR						
8		C RO/SRO	CRD pump trips following scram attempt restarted until suction filter trips are bypas replaced.						
	L								

\* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

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Facility:       LaSalle Co. Station       Scenario No.:       3       Op-Test No.:       2006-301         Examiners:       Operators:	ppendix [	)		Scenario Ou	line		Form ES-D-1		
Turnover:       Approximately 80% RTP. MSIV Scram Functional Test has just been completer         RCIC has just been returned to service after a 3 day outage, but pump operability test still         needs to be performed.       Power will be increased to 100% following completion of RCIC         surveillance.       Surveillance.         Event       Malf.       Event         No.       Type*       Description         1       C (TS)       RCIC fails pump surveillance.         2       C (TS)       Control rod drift (control rod continues to move out after single notch withdrawal).         3       C (TS)       SRO receives report from system engineer that fuel oil analysis for the common unit diesel generator indicates that fuel oil analysis for the common unit diesel generator indicates that fuel oil analysis for the common unit diesel generator indicates that fuel oil analysis for the common unit diesel generator indicates that fuel oil analysis for the common unit diesel generator indicates that fuel oil analysis for the common unit diesel generator indicates that fuel oil analysis for the common unit diesel generator indicates that fuel oil analysis for the concentration is out of specification.         4       C       TDRFP high vibration leading to removal of pump from service.         5       C       CRD flow control valve fails open causing degradation of									
RCIC has just been returned to service after a 3 day outage, but pump operability test still needs to be performed. Power will be increased to 100% following completion of RCIC surveillance.         Event       Malf.       Event Type*       Event Description         1       C (TS) BOP/SRO       RCIC fails pump surveillance.         2       C (TS) RO/SRO       Control rod drift (control rod continues to move out after single notch withdrawal).         3       C (TS) SRO       SRO receives report from system engineer that fuel oil analysis for the common unit diesel generator indicates that fuel oil particulate concentration is out of specification.         4       C BOP/SRO       TDRFP high vibration leading to removal of pump from service.	Initial Cor	nditions: _	Approxima	tely 80% RTP;					
No.No.Type*Description1C (TS) BOP/SRORCIC fails pump surveillance.2C (TS) RO/SROControl rod drift (control rod continues to move out after single notch withdrawal).3C (TS) RO/SROSRO receives report from system engineer that fuel oil analysis for the common unit diesel generator indicates that fuel oil particulate concentration is out of specification.4C BOP/SROTDRFP high vibration leading to removal of pump from service.5CCRD flow control valve fails open causing degradation of	RCIC has needs to	s just been be perforn	returned to	o service after a 3 da	ay outage, bu	it pump operabili	ty test still		
2       C (TS) RO/SRO       Control rod drift (control rod continues to move out after single notch withdrawal).         3       C (TS) SRO       SRO receives report from system engineer that fuel oil analysis for the common unit diesel generator indicates that fuel oil particulate concentration is out of specification.         4       C BOP/SRO       TDRFP high vibration leading to removal of pump from service.         5       C       CRD flow control valve fails open causing degradation of									
2       RO/SRO       single notch withdrawal).         3       C (TS) SRO       SRO receives report from system engineer that fuel oil analysis for the common unit diesel generator indicates that fuel oil particulate concentration is out of specification.         4       C BOP/SRO       TDRFP high vibration leading to removal of pump from service.         5       C       CRD flow control valve fails open causing degradation of	1								
3C (TS) SROSRO receives report from system engineer that fuel oil analysis for the common unit diesel generator indicates that fuel oil particulate concentration is out of specification.4C BOP/SROTDRFP high vibration leading to removal of pump from service.5CCRD flow control valve fails open causing degradation of	2		C (TS) RO/SRO			tinues to move o	out after		
4     BOP/SRO     service.       5     C     CRD flow control valve fails open causing degradation of	3			SRO receives repo analysis for the cor	rt from syste nmon unit die	esel generator in	dicates that		
	4			•	on leading to	removal of pum	p from		
RO/SRO RR pump seals due to thermal shock.	5		C RO/SRO		•		dation of		
6 M Both RR pumps seals sequentially fail resulting in a LOCA. ALL One loop cannot be isolated.	6				•	ally fail resulting	in a LOCA.		
7C BOPWhen HPCS pump is started a water hammer causes a pipe break down stream of the pump (inside the HPCS pump room) that cannot be immediately isolated.	7			pipe break down st	ream of the <b>j</b>	oump (inside the	HPCS		
8 MALL HPCS line break partially drains suppression pool to HPCS room. (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor			L	room.			ol to HPCS		