

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of)	
)	
CONSOLIDATED EDISON COMPANY)	Docket No. 50-003
OF NEW YORK INC.)	
)	
(Indian Point Unit No. 1))	

ORDER APPROVING DECOMMISSIONING PLAN AND
AUTHORIZING DECOMMISSIONING OF FACILITY

By application dated October 17, 1980, as revised October 13, 1981; July 31, 1986; March 28, 1988; August 10 1989; March 28 and July 17, 1990; February 5, April 2, July 31, September 20, and October 12, 1993; May 13 and August 11, 1994; and July 19, 1995; Consolidated Edison Company of New York, Inc. (the licensee) asked the NRC to approve its proposed Decommissioning Plan for Indian Point Unit No. 1 (IP-1) and an amendment to Provisional Operating License No. DPR-5 and the associated Technical Specifications (TSs) to make them consistent with the Decommissioning Plan. The Decommissioning Plan proposes long-term safe storage (SAFSTOR) of IP-1 spent fuel and residual radioactivity until the adjacent Indian Point Unit No. 2 (IP-2) has been permanently shut down.

A Notice of Consideration of Issuance of Amendment and Opportunity for Prior Hearing was published in the FEDERAL REGISTER on December 31, 1985, (50 FR 53407). No request for a hearing or petition for leave to intervene was filed following notice of the proposed action.

c/h

The U.S. Nuclear Regulatory Commission (the Commission) has reviewed the application with respect to the provisions of the Commission rules and regulations and has found that decommissioning as stated in the licensee Decommissioning Plan is consistent with the regulations in 10 CFR Chapter I and will not be inimical to the common defense and security or to the health and safety of the public. The basis for these findings is given in the concurrently issued Safety Evaluation by the Office of Nuclear Reactor Regulation.

The Decommissioning Plan supplements the IP-1 Safety Analysis Report. Accordingly, a license condition has been added allowing the licensee to make changes to the Decommissioning Plan and Safety Analysis Report after performing a review based upon criteria similar to the criteria of 10 CFR 50.59 to ensure that such changes do not involve an unreviewed safety question.

The Commission has prepared an Environmental Assessment for the proposed action. On the basis of that assessment, the Commission has determined that the proposed action will not result in any significant environmental impact and that an environmental impact statement need not be prepared. The Notice of Issuance of Environmental Assessment was published in the FEDERAL REGISTER on

Accordingly, pursuant to Sections 103, 161b, 161i, and 161o of the Atomic Energy Act of 1954, as amended, and 10 CFR 50.82, the proposed Decommissioning Plan, dated October 17, 1980, as revised, is approved, and

decommissioning of the IP-1 facility is authorized in accordance with the Decommissioning Plan and the Commission rules and regulations, subject to the following conditions:

- (a)(1) The approved Decommissioning Plan supplements the Final Safety Analysis Report (FSAR) and the licensee may (i) make changes in the facility or procedures as described in the FSAR and/or the Decommissioning Plan and (ii) conduct tests, or experiments not described in the FSAR or Decommissioning Plan, without prior Commission approval, unless the proposed changes, tests or experiments involve (a) a change in the Technical Specifications (TSs) incorporated in the license or (b) an unreviewed safety question, or (c) activities which would or could
 - (i) foreclose the release of the site for possible unrestricted use, or
 - (ii) significantly increase decommissioning costs, or
 - (iii) cause any significant environmental impact not previously evaluated, or
 - (iv) violate the terms of the possession-only license.
- (2) A proposed change, test, or experiment shall be deemed to involve an unreviewed safety question (i) if the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the FSAR and/or the Decommissioning Plan may be increased or (ii) if the

possibility of an accident or malfunction of a different type than evaluated previously in the FSAR and/or the Decommissioning Plan may be created; or (iii) if the margin of safety as defined in the basis for any TS is reduced.

- (b)(1) The licensee shall maintain records of changes in the facility and of changes in procedures made pursuant to this section if these changes constitute changes in the facility or procedures as described in the FSAR and/or Decommissioning Plan. The licensee shall also maintain records of tests and experiments carried out pursuant to paragraph (a) of this section. These records must include a written safety evaluation that provides the basis for determining that the changes, tests, or experiments do not involve an unreviewed safety question or a significant environmental impact not previously evaluated.
- (2) The licensee shall submit, as specified in 10 CFR 50.4, a report containing a brief description of any changes, tests, and experiments, including summaries of the safety and environmental evaluation of each. The report must be submitted annually.
- (3) The records of changes in the facility shall be maintained until the date of termination of the license and records of changes in procedures and records of tests and experiments shall be maintained for 3 years.

- (c) If the licensee desires to (1) change the TSs or (2) make a change in the facility or in the procedures described in the FSAR or Decommissioning Plan or conduct tests or experiments that are not described in the FSAR or Decommissioning Plan, and that involve an unreviewed safety question, change the TSs, or result in any of the four conditions described in (A)(1)(c) above, the licensee shall submit an application to amend its license pursuant to 10 CFR 50.90 or request approval of a revision to the FSAR and/or Decommissioning Plan.

For further details with respect to this action, see (1) the licensee's application for authorization to decommission the facility, dated October 17, 1980, as revised; (2) Amendment No. to License No. DPR-5; (3) the related NRC Safety Evaluation; and (4) the NRC Environmental Assessment and Finding of No Significant Impact. These documents are available for public inspection at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC, and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York.

FOR THE NUCLEAR REGULATORY COMMISSION

William T. Russell, Director
Office of Nuclear Reactor Regulation

Dated at Rockville, Maryland,
this

(c) If the licensee desires to (1) change the TSs or (2) make a change in the facility or in the procedures described in the FSAR or Decommissioning Plan or conduct tests or experiments that are not described in the FSAR or Decommissioning Plan, and that involve an unreviewed safety question, change the TSs, or result in any of the four conditions described in (A)(1)(c) above, the licensee shall submit an application to amend its license pursuant to 10 CFR 50.90 or request approval of a revision to the FSAR and/or Decommissioning Plan.

For further details with respect to this action, see (1) the licensee's application for authorization to decommission the facility, dated October 17, 1980, as revised; (2) Amendment No. to License No. DPR-5; (3) the related NRC Safety Evaluation; and (4) the NRC Environmental Assessment and Finding of No Significant Impact. These documents are available for public inspection at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York.

FOR THE NUCLEAR REGULATORY COMMISSION

William T. Russell, Director
Office of Nuclear Reactor Regulation

Dated at Rockville, Maryland
this

See previous concurrence

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PErickson/SWeiss
/ /95

OGC
6/19/95

DRPM:D
DCrutchfield
/ /95

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WRussell
/ /95

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CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT UNIT NO. 1

DOCKET NO. 50-3

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No.
License No. DPR-5

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to Provisional Operating License No. DPR-5 filed by Consolidated Edison Company of New York, Inc. (the licensee), dated October 17, 1980, as revised October 13, 1981; July 31, 1986; March 28, 1988; August 10, 1989; March 28 and July 17, 1990; February 5, April 2, July 31, September 20, and October 12, 1993; May 13 and August 11, 1994; and July 19, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will be maintained in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance that: (i) the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) such activities will be conducted in compliance with the Commission regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission regulations and all applicable requirements have been satisfied.

2. Accordingly, Provisional Operating License No. DPR-5 is amended by revising the indicated paragraphs as follows:
 - 2.A. Pursuant to Section 104b. of the Act and Title 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess but not operate the facility at the designated location in Westchester County, New York, in accordance with the procedures and limitations described in the application and this license;
 - 2.B. Pursuant to the Act and 10 CFR Part 70, to receive and possess up to 1918 kilograms of contained uranium-235 previously received for reactor operation;
 - 2.E. Pursuant to the Act and 10 CFR Parts 30 and 70, to receive and possess, but not to separate, such byproduct and special materials as were produced by the prior operation of the facility;
- 3.A. Maximum Power Level

The licensee is prohibited from taking the reactor to criticality, and the facility shall not be operated at any power level.
- 3.B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall maintain the facility in accordance with the Technical Specifications.
6. This amended license is effective as of its date of issuance and shall expire at midnight, September 28, 2013.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis M. Crutchfield, Director
Division of Reactor Program Management
Office of Nuclear Reactor Regulation

Attachment: Appendix A Technical
Specification Changes

Date of Issuance:

Mr. Stephen B. Bram
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

SUBJECT: ORDER TO AUTHORIZE DECOMMISSIONING AND AMENDMENT NO. TO
LICENSE NO. DPR-5 FOR INDIAN POINT UNIT NO. 1 (TAC NO. M59664)

Dear Mr. Bram:

The Commission has issued the enclosed order to authorize decommissioning of Indian Point Unit No. 1. Also enclosed is Amendment No. to License No. DPR-5, which revises the license to possession-only status, revises the Technical Specifications (TSs) and renews the license until September 28, 2013, to make the expiration date consistent with that of the IP-2 license. The order and amendment respond to your application of October 17, 1980, as revised October 13, 1981; July 31, 1986; March 28, 1988; August 10, 1989; March 28 and July 17, 1990; February 5, April 2, July 31, September 20, and October 12, 1993; May 13 and August 11, 1994 and July 19, 1995. The order has been forwarded to the Office of the Federal Register for publication.

A Notice of Consideration of Issuance of Amendment to License and Opportunity for Prior Hearing related to the requested action was published in the FEDERAL REGISTER on December 31, 1985, (50 FR 53407). No comments or requests for hearing were received.

Copies of the related Safety Evaluation, the Environmental Assessment supporting the order, and Amendment No. are enclosed. Also enclosed is a copy of the Notice of Issuance of Environmental Assessment and Finding of No Significant Impact, which was published in the FEDERAL REGISTER on

Sincerely,

Peter B. Erickson, Senior Project Manager
Non-Power Reactors and Decommissioning
Project Directorate
Division of Reactor Program Management
Office of Nuclear Reactor Regulation

Docket No. 50-3

Enclosures: 1. Order Authorizing Decommissioning
2. Amendment No. to License No. DPR-5
3. Safety Evaluation
4. Environmental Assessment (EA)
5. Notice of Issuance of EA

cc w/enclosures: See next page

Mr. Stephen B. Bram
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

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Sincerely,

Peter B. Erickson, Senior Project Manager
Non-Power Reactors and Decommissioning
Project Directorate
Division of Reactor Program Management
Office of Nuclear Reactor Regulation

Docket No. 50-3

Enclosures: 1. Order Authorizing Decommissioning
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3. Safety Evaluation
4. Environmental Assessment (EA)
5. Notice of Issuance of EA

cc w/enclosures: See next page

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See previous concurrence

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OGC
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DRPM:D
DCrutchfield
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WRussell
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MFWebber
GBagchi

A. Alan Blind
Vice President

Consolidated Edison Company of New York, Inc.
Indian Point Station
Broadway & Beakley Avenue
Buchanan, NY 10511
Telephone (914) 734-5340
Fax: (914) 734-5718
blindaa@coned.com

February 14, 2000

Re: Indian Point Unit No. 1
Docket No. 50-3

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555-0001

Subject: Proposed Technical Specification Amendment Consisting of
Administrative Changes

Transmitted herewith is an "Application for Amendment to the Operating License," sworn on February 14, 2000. This application requests an amendment to the Consolidated Edison Company of New York, Inc. (Con Edison), Indian Point Unit No. 1 Technical Specifications. In accordance with 10 CFR 50.91, a copy of this application and the associated attachments are being submitted to the designated New York State official.

The proposed administrative changes consist of the following:

- a) Changes to Sections 3.1.2, 3.2.1, and 4.1.8.1.b to reflect organizational title changes, and
- b) Change to Section 4.1.8.1 to reference the current sections of 10 CFR 20.

Attachment I to this letter provides the proposed changed pages, Attachment II provides the proposed changes as markups on the existing Technical Specification pages, and Attachment III provides the Safety Assessments. It has been determined that the administrative changes set forth herein do not represent a significant hazards consideration as defined by 10 CFR 50.92(c).

02/14/00

ACC1

Should you or your staff have any questions regarding this submittal, please contact Mr. John F. McCann, Manager, Nuclear Safety and Licensing.

Very truly yours,

Robert S. Kane
for A. Blind

Attachments

cc: Mr. Hubert J. Miller
Regional Administrator-Region I
US Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. John L. Minns, Project Manager
Division of Reactor Program Management
US Nuclear Regulatory Commission
Mail Stop 10D-4
Washington, DC 20555

Senior Resident Inspector
US Nuclear Regulatory Commission
PO Box 38
Buchanan, NY 10511

Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, NY 10511

Mr. Paul Eddy
NYS Department of Public Service
3 Empire Plaza
Albany, NY 12223

Mr. William F. Valentino, President
NYS ERDA
Corporate Plaza West
286 Washington Ave. Extension
Albany, NY 12223-6399

Mr. Jack P. Spath, Program Director
NYS ERDA
Corporate Plaza West
286 Washington Ave. Extension
Albany, NY 12223-6399

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of)
CONSOLIDATED EDISON COMPANY) Docket No. 50-3
OF NEW YORK, INC.)
(Indian Point Station, Unit No. 1))

APPLICATION FOR AMENDMENT
TO OPERATING LICENSE

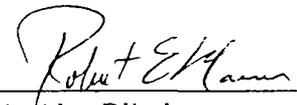
Pursuant to Section 50.90 of the Regulations of the Nuclear Regulatory Commission ("NRC"), Consolidated Edison Company of New York, Inc. ("Con Edison"), as holder of Facility Operating License No. DPR-5, hereby applies for amendment of the Technical Specifications contained in Appendix A of this license.

This Application for amendment to the Indian Point 1 Technical Specifications seeks to propose administrative changes to the following:

- a) Changes to Sections 3.1.2, 3.2.1, and 4.1.8.1.b to reflect organizational title changes, and
- b) Change to Section 4.1.8.1 to reference the current sections of 10 CFR 20.

The specific proposed Technical Specification Revisions are set forth in Attachment I to this Application. A mark-up of the existing Technical Specifications are provided in Attachment II. Safety Assessments of the proposed changes are set forth in Attachment III to this Application. These assessments demonstrate that the proposed changes do not represent a significant hazards consideration as defined in 10 CFR 50.92(c).

As required by 10 CFR 50.91(b)(1), a copy of this Application and our analysis concluding that the proposed changes do not constitute a significant hazards consideration have been provided to the appropriate New York State official designated to receive such amendments.

BY: 
for A. Alan Blind
Vice President - Nuclear Power

Subscribed and sworn to
before me this 14th day
February, 2000.


Notary Public

ELIZABETH A. MELANSON
Notary Public, State of New York
No. 01ME4878094
Qualified in Orange County
Commission Expires Feb. 9, 2001

ATTACHMENT I

PROPOSED TECHNICAL SPECIFICATION CHANGES

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT UNIT NO. 1
DOCKET NO. 50-3
FEBRUARY 2000

2.10.2.5 If a spent fuel pool contains spent fuel, the spent fuel cask shall not be moved over that pool or within a distance of that pool such that the cask could strike the pool if it fell or tipped.

2.10.2.6 A dead-load test shall be successfully performed on the fuel handling building crane before fuel movement begins. The load assumed by the crane for this test must be equal to or greater than the maximum load to be assumed by the crane during the fuel handling operation. A thorough visual inspection of the crane shall be made after the dead-load test and prior to fuel handling.

* Licensed Operator for IP-2

2.11 Fire Protection

Overall site fire protection is provided by a fire protection system which is common to both Unit No. 1 and Unit No. 2. Operation, maintenance and testing are controlled by station procedures.

Fire protection and detection systems provided for the protection of Indian Point Unit No. 2 safe shutdown systems are addressed in Appendix A to the Indian Point Unit No. 2 Facility Operating License No. DPR-26.

3.0 ADMINISTRATIVE AND PROCEDURAL SAFEGUARDS

3.1 Responsibility

3.1.1 The Vice President-Nuclear Power shall be responsible for overall facility activities and shall delegate in writing the succession to this responsibility during his absence.

3.1.2 The Plant Manager shall be responsible for facility operations and shall delegate in writing the succession to this responsibility during his absence.

3.2 Organization

3.2.1 Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Updated FSAR for Indian Point Unit No. 2.
- b. The Plant Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President-Nuclear Power shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. Individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager, however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.
- e. The operation of the facility, the operating organization, the procedures for operation, and modifications to the facility shall be subject to review by the Station Nuclear Safety Committee. The committee shall report to the Vice President-Nuclear Power.

- f. The Nuclear Facilities Safety Committee shall function to provide independent review and audit of designated activities in areas of nuclear engineering, chemistry, radiochemistry, metallurgy and non-destructive testing, instrumentation and control, radiological safety, mechanical and electrical engineering, administrative controls and quality assurance activities, and radiological environmental effects.
- g. All fuel handling shall be under the direct supervision of a licensed operator.*
- h. The Shift Manager is responsible for operations at the Unit No. 1 facility.
- i. The qualification requirements of the Operations Manager and the Assistant Operations Manager are provided in Sections 6.2.2 and 6.3 of Appendix A to the Indian Point Unit No. 2 Facility Operating License No. DPR-26.

* Licensed operator for IP-2

3.3 Operating Instructions and Procedures

- 3.3.1 No fuel will be loaded into the reactor core or moved into the reactor containment building without prior review and authorization by the Nuclear Regulatory Commission.
- 3.3.2 Detailed written instruction setting forth procedures used in connection with the operation and maintenance of the nuclear power plant shall conform to the Technical Specifications.
- 3.3.3 Operation and maintenance of equipment related to safety when there is no fuel in the reactor shall be in accordance with written instructions.

4.1.7 Radiation Protection Program

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

4.1.8 High Radiation Area

4.1.8.1 As an acceptable alternative to the "control device" or "alarm signal" required by 10 CFR 20.1601(a) and 10 CFR 20.1601(b):

- a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 4.1.8.1(a) above, and, in addition, locked doors shall be provided to prevent unauthorized entry to such areas and the keys shall be maintained under the administrative control of the Radiation Protection Manager and/or the Shift Manager on duty.

4.1.9 Spent Fuel Storage and Handling

4.1.9.1 All irradiated fuel shall be stored in the racks provided in the Fuel Handling Building Storage pools, with sufficient shielding that ensures that the radiation level on the operating deck is ≤ 15 mr/hr. Should the radiation level be found to be above 15 mr/hr, corrective action shall be initiated to restore the level to ≤ 15 mr/hr.

ATTACHMENT II

PROPOSED TECHNICAL SPECIFICATION MARKED-UP PAGES

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT UNIT NO. 1
DOCKET NO. 50-3
FEBRUARY 2000

On these marked-up pages from the current Tech Specs:

Additions are shown by ***bold italic***,

and

Deletions are shown by ~~double-strikethrough~~.

2.10.2.5 If a spent fuel pool contains spent fuel, the spent fuel cask shall not be moved over that pool or within a distance of that pool such that the cask could strike the pool if it fell or tipped.

2.10.2.6 A dead-load test shall be successfully performed on the fuel handling building crane before fuel movement begins. The load assumed by the crane for this test must be equal to or greater than the maximum load to be assumed by the crane during the fuel handling operation. A thorough visual inspection of the crane shall be made after the dead-load test and prior to fuel handling.

* **Licensed Operator for IP-2**

2.11 Fire Protection

Overall site fire protection is provided by a fire protection system which is common to both Unit No. 1 and Unit No. 2. Operation, maintenance and testing are controlled by station procedures.

Fire protection and detection systems provided for the protection of Indian Point Unit No. 2 safe shutdown systems are addressed in Appendix A to the Indian Point Unit No. 2 Facility Operating License No. DPR-26.

~~* Licensed Operator for IP-2~~

3.0 ADMINISTRATIVE AND PROCEDURAL SAFEGUARDS

3.1 Responsibility

3.1.1 The Vice President-Nuclear Power shall be responsible for overall facility activities and shall delegate in writing the succession to this responsibility during his absence.

3.1.2 The ~~General Manager Nuclear Power Generation~~ **Plant Manager** shall be responsible for facility operations and shall delegate in writing the succession to this responsibility during his absence.

3.2 Organization

3.2.1 Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Updated FSAR for Indian Point Unit No. 2.
- b. The ~~General Manager Nuclear Power Generation~~ **Plant Manager** shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President-Nuclear Power shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. Individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager, however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.
- e. The operation of the facility, the operating organization, the procedures for operation, and modifications to the facility shall be subject to review by the Station Nuclear Safety Committee. The committee shall report to the Vice President-Nuclear Power.

- f. The Nuclear Facilities Safety Committee shall function to provide independent review and audit of designated activities in areas of nuclear engineering, chemistry, radiochemistry, metallurgy and non-destructive testing, instrumentation and control, radiological safety, mechanical and electrical engineering, administrative controls and quality assurance activities, and radiological environmental effects.
- g. All fuel handling shall be under the direct supervision of a licensed operator.*
- h. The ~~Senior Watch Supervisor~~ **Shift Manager** is responsible for operations at the Unit No. 1 facility.
- i. The qualification requirements of the Operations Manager and the Assistant Operations Manager are provided in Sections 6.2.2 and 6.3 of Appendix A to the Indian Point Unit No. 2 Facility Operating License No. DPR-26.

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- 3.3.1 No fuel will be loaded into the reactor core or moved into the reactor containment building without prior review and authorization by the Nuclear Regulatory Commission.
- 3.3.2 Detailed written instruction setting forth procedures used in connection with the operation and maintenance of the nuclear power plant shall conform to the Technical Specifications.
- 3.3.3 Operation and maintenance of equipment related to safety when there is no fuel in the reactor shall be in accordance with written instructions.

4.1.7 Radiation Protection Program

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

4.1.8 High Radiation Area

4.1.8.1 As an acceptable alternative to the "control device" or "alarm signal" required by ~~paragraph 20.203(c)(2) of 10 CFR 20:~~
10 CFR 20.1601(a) and 10 CFR 20.1601(b):

- a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 4.1.8.1(a) above, and, in addition, locked doors shall be provided to prevent unauthorized entry to such areas and the keys shall be maintained under the administrative control of the Radiation Protection Manager and/or the ~~Senior Watch Supervisor~~ **Shift Manager** on duty.

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4.1.9.1 All irradiated fuel shall be stored in the racks provided in the Fuel Handling Building Storage pools, with sufficient shielding that ensures that the radiation level on the operating deck is ≤ 15 mr/hr. Should the radiation level be found to be above 15 mr/hr, corrective action shall be initiated to restore the level to ≤ 15 mr/hr.

ATTACHMENT III

SAFETY ASSESSMENTS

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT UNIT NO. 1
DOCKET NO. 50-3
FEBRUARY 2000

Introduction

In this Attachment, separate safety assessments are provided, one for each of the following proposed administrative changes:

- a) Changes to Sections 3.1.2, 3.2.1, and 4.1.8.1.b to reflect organizational title changes, and
- b) Change to Section 4.1.8.1 to reference the current sections of 10 CFR 20.

a) Changes To Sections 3.1.2, 3.2.1, and 4.1.8.1.b To Reflect Organizational Title Changes

SECTION I - Description of Change

In Sections 3.1.2 and 3.2.1.b change “General Manager-Nuclear Power Generation” to “Plant Manager.” In Sections 3.2.1.h and 4.1.8.b change “Senior Watch Supervisor” to “Shift Manager.” In addition, the footnote, “* Licensed Operator for IP-2” was moved to just after subsection 2.10.2.6.

SECTION II - Evaluation of Change

These are changes in title only. There is no change in responsibilities or functions performed by these individuals. These are administrative changes that affects only the “management” aspect of the plant. The movement of the footnote was to reflect that the footnote pertains to the text in Subsection 2.10.2 (specifically Subsection 2.10.2.4). These changes do not affect any equipment or physical plant attributes.

In the Unit No. 2 Technical Specification Amendment 198, issued August 1998, the title of “General Manager-Nuclear Power Generation” was changed to “Plant Manager.” At the time of this Unit 2 amendment, Unit No. 1 Technical Specification Sections 3.1.2 and 3.2.1.b should have been changed, but were overlooked.

a) Changes To Sections 3.1.2, 3.2.1, and 4.1.8.1.b To Reflect Organizational Title Changes

SECTION III - No Significant Hazards Evaluation

The proposed changes do not involve a significant hazards consideration because:

- 1) Does the proposed license amendment involve a significant increase in the probability or in the consequences of an accident previously evaluated?

No. The proposed change is administrative in nature. The changes involve updating Sections 3.2.1.h and 4.1.8.b to use the title "Shift Manager" instead of "Senior Watch Supervisor" and updating Sections 3.1.2 and 3.2.1.b to use the title "Plant Manager" instead of "General Manager-Nuclear Power Generation" and movement of the footnote, "* Licensed Operator for IP-2." These changes do not affect possible initiating events for accidents previously evaluated or alter the configuration or operation of the facility. The Limiting Safety System Settings and Safety Limits specified in the current Technical Specifications remain unchanged. Therefore, the proposed changes would not involve a significant increase in the probability or in the consequences of an accident previously evaluated.

- 2) Does the proposed amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

No. The proposed changes are administrative in nature. The safety analysis of the facility remains complete and accurate. There are no physical changes to the facility and the plant conditions for which the design basis accidents have been evaluated are still valid. The operating procedures and emergency procedures are unaffected. Consequently no new failure modes are introduced as a result of the proposed change. Therefore, the proposed changes would not create the possibility of a new or different kind of accident from any accident previously evaluated.

- 3) Does the proposed amendment involve a significant reduction in a margin of safety?

No. The proposed changes are administrative in nature. Since there are no changes to the operation of the facility or the physical design, the Updated Final Safety Analysis Report (UFSAR) design basis, accident assumptions, or Technical Specification Bases are not affected. Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

a) Changes To Sections 3.1.2, 3.2.1, and 4.1.8.1.b To Reflect Organizational Title Changes

SECTION IV - Impact Of Changes

This change will not adversely impact the following:

- ALARA Program
- Security and Fire Protection Programs
- Emergency Plan
- UFSAR or SER Conclusions
- Overall Plant Operations and the Environment

The changes involve updating Sections 3.2.1.h and 4.1.8.b to use the title "Shift Manager" instead of "Senior Watch Supervisor" and updating Sections 3.1.2 and 3.2.1.b to use the title "Plant Manager" instead of "General Manager-Nuclear Power Generation" and movement of the footnote, "* Licensed Operator for IP-2." For the title changes, the UFSAR is in the process of being updated. For the footnote, this level of detail is not listed or implied in the UFSAR, and thus, there is no UFSAR impact. There are no new failure modes introduced by this change. There are no physical changes to the facility and the plant conditions for which the design basis accidents have been evaluated are still valid. The operating procedures and emergency procedures are unaffected.

SECTION V - Conclusion

Therefore, the proposed changes to the Technical Specifications do not involve a significant hazards consideration. In addition, the proposed change to the Technical Specifications has been reviewed by both the Station Nuclear Safety Committee (SNSC) and the Con Edison Nuclear Facility Safety Committee (NFSC). Both Committees concur that the proposed changes do not represent a significant hazards consideration.

b) **Change To Section 4.1.8.1 To Reference The Current Sections Of 10 CFR 20**

SECTION I - Description of Change

In Section 4.1.8.1 change "As an acceptable alternative to the 'control device' or 'alarm signal' required by paragraph 20.203(c)(2) of 10 CFR 20" to "As an acceptable alternative to the 'control device' or 'alarm signal' required by 10 CFR 20.1601(a) and 10 CFR 20.1601(b)."

SECTION II - Evaluation of Change

The NRC had completely revised 10 CFR 20 in 1991, and 10 CFR 20.203(c)(2) is now defunct. The appropriate references for Section 4.1.8.1 are 10 CFR 20.1601(a) and 10 CFR 20.1601(b).

In 10 CFR 20.1601 it states:

- "(a) The licensee shall ensure that each entrance or access point to a high radiation area has one or more of the following features --
- (1) A control device that, upon entry into the area, causes the level of radiation to be reduced below that level at which an individual might receive a deep-dose equivalent of 0.1 rem (1 mSv) in 1 hour at 30 centimeters from the radiation source or from any surface that the radiation penetrates;
 - (2) A control device that energizes a conspicuous visible or audible alarm signal so that the individual entering the high radiation area and the supervisor of the activity are made aware of the entry; or
 - (3) Entryways that are locked, except during periods when access to the areas is required, with positive control over each individual entry.
- (b) In place of the controls required by paragraph (a) of this section for a high radiation area, the licensee may substitute continuous direct or electronic surveillance that is capable of preventing unauthorized entry.
- (c) A licensee may apply to the Commission for approval of alternative methods for controlling access to high radiation areas."

By law, Indian Point 1 is required to comply and does comply with this statement. This change is to remove any ambiguity that may have existed in Section 4.1.8 by referring to a defunct 10 CFR 20 Section.

This change is considered administrative since there is no change in the function, operation or physical configuration of the plant.

b) Change To Section 4.1.8.1 To Reference The Current Sections Of 10 CFR 20

SECTION III - No Significant Hazards Evaluation

The proposed change does not involve a significant hazards consideration because:

- 1) Does the proposed license amendment involve a significant increase in the probability or in the consequences of an accident previously evaluated?

[to Section 4.1.8.1]

No. The proposed change is administrative in nature. The change involves updating Section 4.1.8.1 to referenc^e 10 CFR 20.1601(a) and 10 CFR 20.1601(b). This change does not affect possible initiating events for accidents previously evaluated or alter the configuration or operation of the facility. The Limiting Safety System Settings and Safety Limits specified in the current Technical Specifications remain unchanged. Therefore, the proposed change would not involve a significant increase in the probability or in the consequences of an accident previously evaluated.

- 2) Does the proposed amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

No. The proposed change is administrative in nature. The safety analysis of the facility remains complete and accurate. There are no physical changes to the facility and the plant conditions for which the design basis accidents have been evaluated are still valid. The operating procedures and emergency procedures are unaffected. Consequently no new failure modes are introduced as a result of the proposed change. Therefore, the proposed change would not create the possibility of a new or different kind of accident from any accident previously evaluated.

- 3) Does the proposed amendment involve a significant reduction in a margin of safety?

No. The proposed change is administrative in nature. Since there are no changes to the operation of the facility or the physical design, the Updated Final Safety Analysis Report (UFSAR) design basis, accident assumptions, or Technical Specification Bases are not affected. Therefore, the proposed change does not involve a significant reduction in a margin of safety.

b) Change To Section 4.1.8.1 To Reference The Current Sections Of 10 CFR 20

SECTION IV - Impact Of Changes

This change will not adversely impact the following:

- ALARA Program
- Security and Fire Protection Programs
- Emergency Plan
- UFSAR or SER Conclusions
- Overall Plant Operations and the Environment

The change involves updating Section 4.1.8.1 to reference 10 CFR 20.1601(a) and 10 CFR 20.1601(b). This level of detail is not listed or implied in the UFSAR. Therefore, there is no UFSAR impact. There are no new failure modes introduced by this change. There are no physical changes to the facility and the plant conditions for which the design basis accidents have been evaluated are still valid. The operating procedures and emergency procedures are unaffected.

SECTION V - Conclusion

Therefore, the proposed change to the Technical Specifications does not involve a significant hazards consideration. In addition, the proposed change to the Technical Specifications has been reviewed by both the Station Nuclear Safety Committee (SNSC) and the Con Edison Nuclear Facility Safety Committee (NFSC). Both Committees concur that the proposed change does not represent a significant hazards consideration.

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