January 26, 2007

Mr. Scott P. Murray, Manager
Licensing & Liabilities COE
Global Nuclear Fuel - Americas, LLC
Mail Code K-84
3901 Castle Hayne Road
Wilmington, NC 28401

SUBJECT: CERTIFICATE OF COMPLIANCE NO. 9294 FOR THE MODEL NO. NPC SHIPPING PACKAGE, REQUEST FOR ADDITIONAL INFORMATION (TAC NO. L24033)

Dear Mr. Murray:

This refers to your application dated October 3, 2006, and supplemented by your letter dated December 12, 2006, requesting an amendment to Certificate of Compliance No. 9294.

In connection with our review of your request, we need the information identified in the enclosure to this letter. Additional information requested by this letter should be submitted in the form of revised pages. To assist us in scheduling staff review of your response, we request that you provide this information by February 26, 2007. If you are unable to provide a response by that date, our review may be delayed.

If you have any questions regarding this matter, we would be pleased to meet with you and your staff. I may be contacted at (301) 415-8531.

Sincerely,

/RA/

Stewart W. Brown, Senior Project Manager
Licensing Branch
Division of Spent Fuel Storage and Transportation
Office of Nuclear Material Safety
and Safeguards

Docket No. 71-9294
TAC No. L24033

Enclosure: Request for Additional Information

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Request for Additional Information
Model No. NPC
Docket No. 71-9294

By application dated October 3, 2006, and supplemented by letter dated December 12, 2006, Global Nuclear Fuel - Americas, LLC, requested an amendment to Certificate of Compliance (CoC) No. 9294 for the Model No. NPC transportation package. This request identifies additional information needed by the U.S. Nuclear Regulatory Commission staff (the staff) in connection with its review of the application. The requested information is listed by chapter number and title in the safety analysis report (SAR). NUREG-1609, "Standard Review Plan for Transportation Packages for Radioactive Material," was used for this review. This request describes information needed by the staff for it to complete its review of the application and to determine whether the application has demonstrated compliance with regulatory requirements.

Chapter 1.0 General Information

1.1 Submit a revised drawing of the NPC Powder Container. Drawing SK105E4037, sheets 1 through 3, Revision 1, appears to contain information not appropriate for inclusion within a (CoC). Also, confirm that the NPC Powder Container depicted in this revised drawing will be the "plastic receptacle" noted in CoC No. 9294, Revision 4, Condition 5.(a)(3)(b), footnote No. 4. If so, the SAR should be revised to reflect this.

A CoC is a regulatory instrument to control the design of a package used for transportation of radioactive material. *See* 10 CFR 71.4. While the bottle and lid appear to be components associated with the Model No. NPC packaging, the NPC skid base and NPC skid lid do not appear to be packaging components.

This information is being requested in accordance with the provisions of 10 CFR 71.33 which requires an application to include a description in sufficient detail to identify the package accurately and provide sufficient basis for evaluation of the package.

1.2 Submit a drawing of the NPC "metal receptacle" noted in CoC No. 9294, Revision 4, Condition 5.(a)(3)(b), footnote No. 4. This drawing should contain sufficient information inclusion in CoC No. 9294. In the alternative, revise the SAR to remove the "metal receptacle" as a packaging component.

This information is being requested in accordance with the provisions of 10 CFR 71.33 which requires an application to include a description in sufficient detail to identify the package accurately and provide sufficient basis for evaluation of the package.

1.3 Submit a drawing of the NPC "dunnage," if appropriate, based on response to question 2-1. This drawing should contain sufficient information for inclusion in CoC No. 9294.

This information is being requested in accordance with the provisions of 10 CFR 71.33 which requires an application to include a description in sufficient detail to identify the package accurately and provide sufficient basis for evaluation of the package.

ENCLOSURE

Chapter 2.0 Structural Evaluation

2-1 Confirm by test or analysis or both that for ICCAs with contents in either plastic or metal receptacles the containment barrier will not be adversely affected as a result of the hypothetical accident condition (HAC).

The internal arrangement/layout of the receptacles within the ICCA cavity is very important. The layout may influence the structural performance and containment function of the ICCA. Therefore, results of either a test or an analysis, or both, are needed to demonstrate that for a given layout, the ability of the ICCA to retain fissile material under the HAC will not be impaired as a result of an impact by the receptacles.

This information is being requested to enable the staff to determine compliance with the requirements of 10 CFR 71.43 and 71.55.

Chapter 3.0 Thermal

3-1 Provide the results of an evaluation of the effect of the HAC fire on the “plastic receptacles” within the ICCAs.

This information is being requested to enable the staff to determine compliance with the requirements of 10 CFR 71.73.