

Exelon Nuclear Peach Bottom Atomic Power Station 1848 Lay Road Delta, PA 17314-9032

10CFR 50.73

January 12, 2007

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U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

> Peach Bottom Atomic Power Station (PBAPS) Units 2 and 3 Facility Operating License No. DPR-44 <u>NRC Docket Nos. 50-277 and 50-278</u>

Subject: Licensee Event Report (LER) 2-06-04

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This LER reports a condition involving an Unanalyzed Condition as a result of a 1985 plant modification that inadvertently created an Emergency Diesel Generator building Carbon Dioxide Suppression room flooding vulnerability. In accordance with NEI 99-04, the regulatory commitment contained in this correspondence is to restore compliance with the regulations. The specific methods that are planned to restore and maintain compliance are discussed in the LER. If you have any questions or require additional information, please do not hesitate to contact us.

Sincerely,

) PZ FOR J.P. GRIMES

Joseph P. Grimes Plant Manager Peach Bottom Atomic Power Station

JPG/csb/IR 554800

Attachment

CC:

PSE&G, Financial Controls and Co-owner Affairs
R. R. Janati, Commonwealth of Pennsylvania
INPO Records Center
S. Collins, US NRC, Administrator, Region I
R. I. McLean, State of Maryland
US NRC, Senior Resident Inspector

CCN 06-14101



U.S. NUCLEAR REGULATORY COMMISSION (6-2004) LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)								APPROVED BY OMB: NO. 3150-0104 EXPIRES: 06/30/2007 Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information								
1. FACILITY NAME Peach Bottom Atomic Power Station (PBAPS) Unit 2								2. DOCKI 05	et numbi 000 277	ER	3. PAGE 1 OF 4					
4. TITLE Plant	t Modi:	fication	Create	d Diesel C	enerat	or Build	ing Cart	oon Di	oxide Sı	appressio	on Room F	looding V	ulnerabil	ity		
5. E	VENT D	ATE	6. LER NUMBER 7. REPORT DAT					ATE	8. OTHER FACILITIES INVOL					.VED		
MONTH	DAY	YEAR	YEAR	SEQUENTIA NUMBER	L REV NO.	MONTH	DAY	YEAR		'BAPS Unit 3		0500027		10MBER 1278		
11	17	2006	06	- 04	• 0	01	12	2007	FACILITY	ACILITY NAME			DOCKET N 050	DOCKET NUMBER		
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10. POW	1 /ER LEV	EL	20.2201(b) 20.2203(a)(3)(i) 20.2201(d) 20.2203(a)(3)(ii) 20.2203(a)(1) 20.2203(a)(3)(ii) 20.2203(a)(2)(ii) 50.36(c)(1)(i)(A) 20.2203(a)(2)(ii) 50.36(c)(1)(ii)(A) 20.2203(a)(2)(iii) 50.36(c)(2) 20.2203(a)(2)(iv) 50.46(a)(3)(ii)					(3)(i) (3)(ii) (4) (i)(A) (ii)(A) (ii)	$ \begin{bmatrix} 50.73(a)(2)(i)(C) \\ 50.73(a)(2)(ii)(A) \\ 50.73(a)(2)(ii)(A) \\ 50.73(a)(2)(ii)(B) \\ 50.73(a)(2)(iii)(B) \\ 50.73(a)(2)(iii) \\ 50.73(a)(2)(iii) \\ 50.73(a)(2)(ix)(A) \\ 50.73(a)(A) \\ 5$)(A))(B) A)		
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FACILITY NAME TELEPHONE NUMBER (Include Area Code) PBAPS Units 2 and 3, James Armstrong, Regulatory Assurance Manager 717-456-3351																
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On 11/17/2006, Engineering personnel determined that a potential flood vulnerability had existed in the Emergency Diesel Generator (EDG) building Carbon Dioxide Suppression room. A plant modification performed in 1985 had installed a catch basin at the EDG building fuel oil filling station, which is located outside the EDG building. The catch basin discharge was tied in to the EDG building's oily waste separator tank, upstream of the flood protection isolation valve. In the event of a design basis flood, a potential pathway existed for floodwater to enter the building through the floor drains. It was determined that the maximum credible flow rate would have exceeded the capability of the floor sump, thereby potentially challenging the Emergency Service Water system booster pumps and return valves, as well as High Pressure Service Water system return valves. A temporary change to the applicable special event procedure for floods was previously implemented on 11/10/06 to mitigate this condition. The procedures will be permanently revised. There were no actual safety consequences associated with this event.

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NRC FORM 366A U.S. NUCLEAR REGULATORY COMMIS (1-2001) LICENSEE EVENT REPORT (LER)	SSION						
FACILITY NAME (1)	DOCKET (2)		LER NUMBER (6)	PAGE (3)			
Peach Bottom Atomic Power Station, Unit 2	05000277	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		06	- 04 -	00	2	OF	(3)

NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

Unit Conditions at the Time of Discovery

PBAPS Unit 2 and Unit 3 were in Mode 1 at 100% reactor thermal power when the event was discovered. There were no other structures, systems or components out of service that contributed to this event.

Description of the Event

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On 11/17/2006, Engineering personnel concluded that a potential flood vulnerability had existed in the Emergency Diesel Generator (EDG) building (EIIS: ME) Carbon Dioxide Suppression room. A plant modification performed in 1985 had installed a catch basin at the EDG building fuel oil filling station, which is located outside the EDG building. The catch basin discharge was tied in to the EDG building's oily waste separator tank, upstream of the flood protection isolation valve. In the event of a design basis flood, a potential pathway existed for floodwater to enter the building through the floor drains. It was determined that the maximum credible flow rate would likely have exceeded the capability of the floor sump, thereby potentially challenging the Emergency Service Water system booster pumps and return valves, as well as High Pressure Service Water system return valves.

During a walk down by Engineering and Operations personnel on 11/07/06, it was observed that the EDG building fuel oil filling station catch basin discharges to the EDG building oily waste separator tank upstream of its flood protection isolation valve. This isolation valve would be manually closed in the event of an external flood as directed by special event flood procedures. The catch basin discharge pipe inadvertently defeated this isolation valve by providing a flow path from outside the EDG building and into the Carbon Dioxide Suppression room sump through the floor drains. This created a path for external floodwater to back up into the EDG building Carbon Dioxide Suppression (EIIS: KP) room during a postulated external flood event. Analysis performed by Engineering determined that the Carbon Dioxide Suppression room sump pumps (EIIS: P) would not be able to keep up with the worst-case backflow through the drain line. It was confirmed that the four EDG diesel (EIIS: EK) bays were adequately protected from this flooding potential by check valves (EIIS: V), which had recently been verified to be operational. The equipment in the EDG building Carbon Dioxide Suppression room potentially impacted by this condition are the Emergency Service Water (ESW) system (EIIS: BI) booster pumps and return Motor-Operated Valves (MOVs), and the High Pressure Service Water (HPSW) system return MOVs.

Based on concerns identified in a review of yard area drawings and as a result of a field walk down by Engineering and Operations personnel regarding this concern, a temporary change was made to the station Special Event procedure SE-4 for external flooding on 11/10/06. This temporary change directs the plugging of the drain line in the outside sump in sufficient time to preclude any buildup of floodwater in the room.

In summary, the walkdown performed on 11/07/06 identified the catch basin interconnection, followed by a temporary change to the external flooding procedure on 11/10/06. Upon conclusion by Engineering that the sump capability was exceeded, this condition was determined to be reportable on 11/17/06.

NRC FORM 366A U.S. NUCLEAR REGULATORY COMMISSION (1-2001)LICENSEE EVENT REPORT (LER) FACILITY NAME (1) DOCKET (2) LER NUMBER (6) PAGE (3) SEQUENTIAL NUMBER REVISION YEAR NUMBER Peach Bottom Atomic Power Station, Unit 2 05000277 06 04 00 3 OF 4

NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

This report is being submitted pursuant to 10CFR 50.73(a)(2)(ii)(B) to report an unanalyzed condition that degraded plant safety. Because the PBAPS licensing basis assumes a flood elevation at the EDG building of 137.5 feet and assumes that the EDG building has flood protection up to 138 feet, this resulted in an unanalyzed condition. The flood valve isolation would not have prevented the flow from the outside catch basin into the EDG building Carbon Dioxide Suppression room sump and floor drains.

Cause of the Event

It was determined that a plant modification performed in 1985 installed the catch basin outside of the EDG building at the fuel oil fill station.

The cause of the as-found condition was the failure of the design and review organizations at the time of the modification to recognize the impact of the EDG Building oil fill station catch basin installation on the Carbon Dioxide Suppression Room flood protection requirements. The rigor applied to the engineering evaluation and review in the development of the plant modification was inadequate. The plant modification package and the associated design were reviewed to determine the evaluation / justification for the installed configuration. This included a review of the project scope documents, work instructions, PORC meeting minutes, installation drawings and training materials. The Safety Evaluation concluded that the installation of the catch basin would have had no adverse effect on the EDG building wall and seismic separation was not required. The oil spill containment system was not considered nuclear safety-related, and therefore there were no special requirements for its installation. The modification did not involve safety-related systems, equipment or materials.

Analysis of the Event

There were no actual safety consequences associated with this event.

The floods estimated to recur once every 250 years reach an elevation of 113 feet (Conowingo Datum). General grade at the site has been established at a nominal 115' foot level. The worst-case probable maximum flood (PMF) combined with a postulated failure of the upstream dam (Holtwood Dam) would require the EDG building to withstand a design basis external flood with standing water at the 132' elevation. The catch basin design would have potentially allowed floodwater to enter the drains at the 128' elevation.

Any postulated floods that may have occurred below the 128' elevation would not have resulted in water entry into the EDG building Carbon Dioxide Suppression room. In the unlikely event of an onset of a design basis PMF flood, it is expected that significant scrutiny of the flood protection features of the station structures would have been performed. These reviews would have likely identified this vulnerability prior to a PMF occurring. Past station experience with severe weather such as hurricanes confirms that sufficient notice is provided (on the order of days) to allow for detailed reviews of existing station flood prevention features, and provide adequate time to supplement them, as necessary, to preclude equipment damage. It was confirmed that the four Emergency Diesel Generator bays were protected at all times by separate back flow check valves.

LICENSEE EVENT REPORT (LER	IMISSION					
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Assessments indicate that this event	G and ECCS room would not be risk sig	cooler fund gnificant.	ctions.			
Corrective Actions						
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A review of the current PBAPS confi would prevent recurrence of this typ modification to be performed as a nu procedures would have required the d Thus, further corrective actions related	iguration control pro e of event. Current clear safety-related esigner to consider t ed to the current des	ocedures co configura design cha he impact to ign chango	oncluded that the tion control we nge. As such, to external and i e process are no	ne current o buld result the configu nternal floo ot required	design in this tration oding a	proces type o contro nalyses
An extent of condition review was per flood protected structures identified in originally designed external flood pr Control Room complex, Emergency service water pumps), and Emergen conditions were identified.	rformed to focus on a the PBAPS UFSA rotection features. Diesel Generator bu acy heat sink facilit	modificati R that coul The reviev iilding, Pu y, includir	ons performed d create new flo vs included the mp structure (p ng cooling tow	on other PI ood entry pa Reactor b ortion cont er. No ot	BAPS oths, or ouildin taining her re	externa by-pas g, Mai g critica portabl
Previous Similar Occurrences						