

January 24, 2007

Mr. David W. Turner
Vallecitos Nuclear Center
General Electric Company
6705 Vallecitos Road
Sunol, CA 94586

SUBJECT: NRC INSPECTION REPORT NO. 50-73/2007-201

Dear Mr. Turner:

This letter refers to the inspection conducted on January 9-11, 2007, at the General Electric Nuclear Test Reactor facility. The inspection included a review of activities authorized for your facility. The enclosed report presents the results of that inspection.

Areas examined during the inspection are identified in the report. Within these areas, the inspection consisted of selective examinations of procedures and representative records, interviews with personnel, and observation of activities in progress. Based on the results of this inspection, no safety concerns or noncompliance with U.S. Nuclear Regulatory Commission (NRC) requirements were identified. No response to this letter is required.

In accordance with Section 2.390, "Public inspections, exemptions, requests for withholding," of Title 10 of the *Code of Federal Regulations*, a copy of this letter and its enclosure will be made available electronically for public inspection in the NRC Public Document Room or from the NRC's Agencywide Documents Access and Management System (ADAMS), accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html>.

Should you have any questions concerning this inspection, please contact Mr. Kevin M. Witt at (301) 415-4075.

Sincerely,

/RA/

Johnny Eads, Branch Chief
Research and Test Reactors Branch B
Division of Policy and Rulemaking
Office of Nuclear Reactor Regulation

Docket No. 50-73
License No. R-33

Enclosure: NRC Inspection Report No. 50-73/2007-201
cc w/encl: Please see next page

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ACCESSION NO.: ML070220431

TEMPLATE #: NRR-106

OFFICE	PRTB	PRTB:LA	PRTB:BC
NAME	KWitt:cah	EHylton	JEads
DATE	01/23/2007	01 /23/2007	01/24/2007

General Electric Company (NTR)

Docket No. 50-73

cc w/encl:

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Test, Research, and Training
Reactor Newsletter
University of Florida
202 Nuclear Sciences Center
Gainesville, FL 32611

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF NUCLEAR REACTOR REGULATION

Docket No: 50-73

License No: R-33

Report No: 50-73/2007-201

Licensee: General Electric Company

Facility: Nuclear Test Reactor (NTR)

Location: Sunol, CA

Dates: January 9-11, 2007

Inspector: Kevin M. Witt

Accompanied by: John Buckley, NRC Project Manager
Robert Evans, NRC Senior Health Physicist
Jason Razo, NRC Health Physicist

Approved by: Johnny Eads, Branch Chief
Research and Test Reactors Branch B
Division of Policy and Rulemaking
Office of Nuclear Reactor Regulation

EXECUTIVE SUMMARY

General Electric Company
Nuclear Test Reactor (NTR)
Inspection Report No.: 50-73/2007-201

The primary focus of this routine, announced inspection was the on-site review of selected aspects and activities conducted since the last NRC inspection of the licensee's Class II non-power reactor safety programs including: organization and staffing, procedures, experiments, radiation protection program, design changes, committees, audits and reviews, and follow-up on previous open items.

Organization and Staffing

- The organizational structure and staffing were consistent with Technical Specification requirements for current operations.

Procedures

- Procedural control and implementation programs satisfied Technical Specification requirements.

Experiments

- The approval and control of experiments met Technical Specification and applicable regulatory requirements.

Radiation Protection Program

- Surveys were being completed and documented acceptably to permit evaluation of the radiation hazards present.
- Postings met the regulatory requirements specified in 10 CFR Parts 19 and 20.
- Personnel dosimetry was being worn as required, and doses were well within the licensee's procedural action levels and NRC's regulatory limits.
- Radiation monitoring equipment was being maintained and calibrated as required.
- The radiation protection program being implemented by the licensee satisfied regulatory requirements.
- Effluent monitoring satisfied license and regulatory requirements, and releases were within the specified regulatory and Technical Specification limits.

Design Changes

- Based on the records reviewed, the inspector determined that the licensee's design change program was being implemented as required.

Committees, Audits and Reviews

- Review and oversight functions required by the Technical Specification were acceptably completed by the Vallecitos Technological Safety Council and the office of regulatory compliance.

Follow-up on Previous Open Items

- The issue regarding the failure to conduct tests of the High-Level Conference Circuit system as required by the Emergency Plan was closed.

REPORT DETAILS

Summary of Plant Status

The licensee's TRIGA Mark I research reactor, licensed to operate at a maximum steady-state thermal power of one hundred kilowatts (100 kW), continues to be operated in support of operator training, surveillance, and utilization involving neutron radiography. During the inspection the reactor was being operated at full power to conduct neutron radiography. The licensee indicated that transportation of radioactive materials has not been conducted since the previous inspection. During the inspection, the NRC also conducted an inspection of the three shutdown reactors on site (Docket Nos. 50-18, 50-70, and 50-183), which will be documented in a separate inspection report.

1. Organization and Staffing

a. Inspection Scope (Inspection Procedure [IP] 69001)

The inspector reviewed the following regarding the licensee's organization and staffing to ensure that the requirements of Sections 6.1 of Technical Specifications (TS), Amendment No. 22, dated October 16, 2001, were being met:

- General Electric (GE) Nuclear Test Reactor (NTR) organizational structure and staffing
- management responsibilities and staff qualifications
- staffing requirements for the safe operation of the facility
- Organization Chart for the GE Vallecitos Nuclear Center (VNC), dated January 1, 2007
- Letters to NRC subject: R-33 NTR Organization Changes, dated June 26 and December 20, 2006
- Standard Operating Procedure 6.1 "Staffing Requirements" Revision 941, dated September 8, 2004

b. Observations and Findings

The NTR organizational structure and the responsibilities of the reactor management and staff had not changed since the last inspection (see NRC Inspection Report No. 50-73/2005-201). The licensee designated a new site manager, a new manager of regulatory compliance (RC) and a new NTR facility manager (FM). The inspector verified that the training and qualifications satisfied the regulatory requirements as well as the guidance contained in American National Standard for Selection and Training of Personnel for Research Reactors, ANSI/ANS-15.4-1988, Section 4-6. Current licensed staff consisted of the FM and three other facility staff members, of which three are qualified Senior Reactor Operators (SROs) and the other a qualified Reactor Operator (RO). One of the SROs was retired, but worked on an infrequent basis to supplement the full time staff. The former FM was still working at the facility and will soon be retiring but is in the process of transferring facility knowledge to the current FM. There was also a Control and Instrumentation Technician on staff at the VNC. The licensee employed contract workers to assist with neutron radiography operations.

The reactor operations staff's qualifications satisfied the training and experience requirements stipulated in the TS. The operations log and associated records confirmed that shift staffing met the minimum requirements for duty personnel. Review of records verified that management responsibilities were administered as required by TS and applicable procedures.

c. Conclusions

The organizational structure and staffing were consistent with TS requirements for current operations.

2. Procedures

a. Inspection Scope (IP 69001)

To verify compliance with TS Section 6.4, the inspector reviewed selected portions of the following:

- administrative controls
- records of changes to procedures
- procedural implementation
- procedure change log
- Administrative Procedure 9.2 "Standard Operating Procedures" Revision 979, dated December 8, 2006
- Administrative Procedure 9.3 "Engineering Release" Revision 819, dated February 24, 1994
- Administrative Procedure 9.11 "Reportable Incidents" Revision 980, dated December 15, 2006
- Completed NEO 887 "Engineering Release" forms, dated from January 2006 to present
- Nuclear Safety Procedure 3550 "Building 105/NTR Work Routines" Revision 9, dated November 2006

b. Observations and Findings

Administrative policies and controls had been developed for changing and reviewing procedures. Written changes were reviewed and approved by the NTR Manager and the RC manager as required. Oversight and review of procedure implementation was provided by licensee management. NTR staff members conducted TS activities in accordance with applicable procedures. Records showed that procedures for TS required items were available. The inspector confirmed that procedure changes are reviewed by licensed operators immediately after the procedure change is authorized. The inspector reviewed several of the procedure changes that were initiated during the inspection period and determined that the changes were appropriately analyzed.

c. Conclusions

Procedural control and implementation programs satisfied TS requirements.

3. Experiments

a. Inspection Scope (IP 69001)

The inspector reviewed selected aspects of the following to verify compliance with TS Sections 3.7 and 6.8:

- experimental program requirements
- experimental administrative controls and precautions
- Administrative Procedure 9.3 "Engineering Release" Revision 819, dated February 24, 1994
- Experiment Operations Procedure 10.1 "Experiment Type Approval" Revision 918, dated September 30, 1998
- Experiment Operations Procedure 10.2 "Individual Experiment Review and Approval" Revision 775, dated June 9, 1992
- Experiment Operations Procedure 10.4 "Explosive Handling" Revision 948, dated December 15, 2004
- Experiment Operations Procedure 10.6 "Cable-Held Retractable Irradiation System" Revision 976, dated December 7, 2006
- Experiment Operations Procedure 10.7 "Neutrography of Nonradioactive Material" Revision 669, dated May 17, 1989
- Experiment Operations Procedure 10.9 "Neutrography of Radioactive Material - North Room" Revision 673, dated April 27, 1989
- Completed EOP 10.6 forms, dated from January 2006 to present
- Completed EOP 10.9 forms, dated from January 2006 to present
- Completed NEO 887 "Engineering Release" forms, dated from January 2006 to present

b. Observations and Findings

There was one type of experiment frequently conducted at the NTR, which was the routine irradiation of various materials. The two experimental facilities that can be used for sample irradiations are the north and south beam ports and the cable-held retractable irradiation system (CHRIS). Samples can be loaded and unloaded from the beam ports and the CHRIS while the reactor is at power, and interlocks are in place to ensure that personnel are not exposed to open neutron beam ports. Samples that have been irradiated at the NTR include explosives, spent fuel and classified materials. The licensee has thoroughly analyzed all of the TS requirements for the irradiation of explosives in the exposure rooms and has specified quantity limits for irradiations. Several operations have occurred recently that has approached the TS limit for storage of explosives at the NTR facility. The inspector verified that proper controls were implemented to ensure that all of the limits were met. The experiment authorization forms for each experiment that had been completed for irradiating samples during the inspection period contained the appropriate information, hazards

analyses as applicable, and had been reviewed and approved as required by TS and procedure.

No new experiments had been initiated, reviewed, or approved since the previous inspection at the facility. If any experiments were to be initiated, they would be reviewed and approved by the NTR manager, or an SRO, and the RC manager. All new experiments would be completed under the supervision of the NTR manager or SRO and in accordance with TS requirements (e.g., reactivity limitations, corrosion resistance, etc.).

c. Conclusions

The approval and control of experiments met TS and applicable regulatory requirements.

4. Radiation Protection Program

a. Inspection Scope (IP 69001)

The inspector reviewed the following to verify compliance with 10 CFR Part 20 and TS Section 6.3, as well as procedural requirements:

- Personnel quarterly dosimetry results for 2005 to present
- Environmental dosimetry results for 2005 to present
- Radiation Protection Refresher Training, dated January 23, 2006
- Memorandum to NTR staff from E. Ehrlich, "NTR ALARA Goals for 2006," dated 2006
- VNC ALARA Goals for 2006," dated March 31, 2006
- Radiological Safety Procedure 7.1 "Radiation Protection Program" Revision 820, dated March 18, 1994
- Radiological Safety Procedure 7.3 "Radiation and Contamination Survey" Revision 904, dated November 19, 1996
- Nuclear Safety Procedure 3550 "Building 105/NTR Work Routines" Revision 9, dated November 2006
- Nuclear Safety Procedure 5100 "Inventory, Inspection, and Calibration of Instruments Used for Radiation Protection of Personnel," Revision 9, dated December 2005
- Vallecitos Safety Standards (VSS) 8.1 "Radiological Training for VNC Employees," Revision 5, dated August 1988
- Completed NSP 3550 forms, dated from January 2006 to present
- Completed NSP 5100 forms, dated from January 2006 to present
- Completed NEO 289 "Nuclear Safety Survey Record" forms, dated January 2006 to present
- Completed PM# 30 "Stack Particulate Monitor" form, dated April 24, 2006
- General Electric Nuclear Test Reactor Annual Report No. 46 for the Year 2005, dated March 31, 2006
- Summary of 2005 Safety Performance and Activities for General Electric VNC, dated March 31, 2006

- Annual Report 2005 for Effluent Monitoring and Environmental Surveillance Programs, dated March 31, 2006
- Vallecitos Technological Safety Council (VTSC) meeting minutes for August 8 and November 8, 2005; and February 2, May 4, September 22, and October 26, 2006

b. Observations and Findings

(1) Surveys

The inspector reviewed daily, weekly, and monthly radiation and contamination surveys of the licensee's controlled areas as well as radiation wipe surveys completed by personnel with the RC / Environmental Health and Safety (EHS) office. The surveys had been completed in accordance with the applicable procedure. The results were documented on the appropriate forms, evaluated as required, and corrective actions taken when readings or results exceeded set action levels. No abnormal elevated readings were discovered during the inspection period. The survey also included a checklist of items to be verified such as the adequacy of warning signs and postings in the area. The number and location of survey points were adequate to characterize the radiological conditions.

Daily surveys checked commonly used areas for contamination. Weekly surveys of general radiation exposure levels by the RC/EHS staff were conducted in accordance with Nuclear Safety Procedure 3550 and logged on the appropriate forms. A further survey is completed on a monthly basis and includes areas external to the NTR facility. Any readings that were unexpected required further evaluation from RC/EHS. During the inspection, the inspector observed a daily survey of the facility, which confirmed that there was no contamination in the commonly used areas of the facility.

Primary coolant water samples are evaluated annually while secondary coolant water samples are evaluated before release to the environment. Monitoring of the reactor water did not indicate abnormal readings. The samples that were taken indicate that the reactor integrity has not been compromised and shows no trend of breakdown, release, or degradation.

(2) Postings and Notices

The inspector reviewed the postings at the entrances to various controlled areas including the Reactor Bay, and radioactive material storage areas. The postings were acceptable and indicated the radiation and contamination hazards present. The facility's radioactive material storage areas were noted to be properly posted. No unmarked radioactive material was found in the facility. A copy of current notices to workers required by 10 CFR Part 19 was posted at the entrance to the Reactor Cell.

(3) Dosimetry

The licensee used a National Voluntary Laboratory Accreditation Program-accredited vendor to process personnel dosimetry. Through direct observation, the inspector determined that dosimetry was acceptably used by facility personnel. For visitors to the facility, an Optically Stimulated Luminescence Dosimeter (OSLD) is issued to each person entering a posted radiation area. Except for escorted tours, whole body counts are conducted for all visitors before and after the visit to the NTR to document any contamination that the individual may have received. Records indicate that no abnormal readings were obtained.

An examination of the records for the inspection period showed that all exposures were well within NRC limits and within licensee action levels. There are currently six people at the NTR facility that are being monitored, in addition to the RC/EHS personnel that perform duties less than full-time at the facility. Extremity monitoring, accomplished through the use of finger rings, also showed relatively low doses to the hands of staff members. Current exposure records for 2006 indicate no increased levels in exposures from 2005.

(4) Radiation Monitoring Equipment

The calibration of portable survey meters and friskers was typically completed by instrumentation personnel at the calibration lab, while fixed radiation detectors and air monitoring instruments were calibrated at the facility using a portable source. The calibration records of portable survey meters, friskers, fixed radiation detectors, and air monitoring equipment in use at the facility were reviewed. Calibration frequency met the requirements established in the applicable procedures while records were being maintained as required.

During the inspection, the inspector visited the calibration facility located in the GE Facilities Maintenance building. An electronics technician who conducts calibrations for portable radiation detectors described the equipment in the facility for the inspector. The calibration records reviewed were thorough and were completed using the appropriate techniques and according to procedure. The inspector observed that proper precautions are always used to maintain doses as low as reasonably achievable (ALARA).

(5) Radiation Protection Program

The inspector verified that the GE radiation protection program was being reviewed annually as required. Part of the annual safety performance and activities summary report showed that the radiation protection program at the NTR facility and the VNC was being conducted as required by the applicable procedures. No issues related to the radiation protection program at the VNC were identified in the audit of the program. The NTR manager sets annual goals for keeping exposures ALARA, and a subsequent analysis indicated that the ALARA goal for the summation of all NTR personnel exposures was not met for 2005. The licensee has attributed this increase in exposure levels to an increase in work load and has indicated that current exposure levels for 2006 will most

likely meet the ALARA goal. In the next year, the licensee intends to keep a closer watch on the exposure levels to ensure that the goals are met.

The licensee's radiation protection program is established in the VSS, which is a group of procedures that lay out effective methods of ensuring safe radiation practices. Procedure VSS 8.1 requires that all personnel who have unescorted access to the NTR (a radiation area) should receive training in radiation protection, policies, procedures, requirements, and facilities prior to entry. The training covered the topics required to be taught in 10 CFR Part 19 and the results of an examination indicated that the staff understood what was presented. The inspector also noted that annual refresher training is required for all radiation workers.

(6) Facility Tours

The inspector toured the Reactor Cell and the accompanying utilization facilities. Control of radioactive material and control of access to radiation and high radiation areas were acceptable. The postings and signs for these areas were appropriate. The inspector also determined that there were no measurable releases of gaseous or liquid radioactive material from the research reactor facility.

(7) Environmental Monitoring

The licensee maintains an environmental monitoring program in order to comply with the NRC regulations and other state and locally enforced regulations. Several OSLDs are placed in strategic locations around the VNC site, which is approximately 1600 acres. Records show that annual doses were generally minimal and always showed exposures to be less than 100 mrem.

Liquid releases from the facility are generally limited to the effluent from the secondary side of the reactor cooling system, which is a once through system. All water that passes through the heat exchanger is collected in a basin on the VNC site and is monitored before release through spray nozzles onto an open field. Review of measurements indicate that there was no measurable amount of radiation in the water released to the environment.

All gaseous releases from the facility are measured with a gaseous effluent monitor. As seen in the annual reports issued by the licensee, the release of gaseous effluents from the facility for the previous two years was less than the applicable limits specified in 10 CFR Part 20. The licensee uses the Environmental Protection Agency computational code "COMPLY," to demonstrate compliance with 10 CFR 20.1301(a)(1).

c. Conclusions

The inspector determined that : (1) surveys were being completed and documented acceptably to permit evaluation of the radiation hazards present, (2) postings met the regulatory requirements specified in 10 CFR Parts 19 and 20, (3) personnel dosimetry

was being worn as required, and doses were well within the licensee's procedural action levels and NRC's regulatory limits, (4) radiation monitoring equipment was being maintained and calibrated as required, (5) the radiation protection program being implemented by the licensee satisfied regulatory requirements, and (6) effluent monitoring satisfied license and regulatory requirements, and releases were within the specified regulatory and TS limits.

5. Design Changes

a. Inspection Scope (IP 69001)

In order to verify that any modifications to the facility were consistent with 10 CFR 50.59 and were reviewed as stipulated in TS Sections 6.2 & 6.3, the inspector reviewed selected aspects of:

- facility design changes and records for the past year
- facility configuration and associated records
- minor and substantive procedural changes and the associated approval
- General Electric Nuclear Test Reactor Annual Report No. 46 for the Year 2005, dated March 31, 2006
- Administrative Procedure 9.3 "Engineering Release" Revision 819, dated February 24, 1994
- Administrative Procedure 9.4 "Change Authorization" Revision 561, dated February 7, 1986
- Completed NEO 887 "Engineering Release" forms, dated from January 2006 to present
- Completed VNC 3080 "Change Authorization" forms, dated from January 2006 to present

b. Observations and Findings

Through review of applicable records and interviews with licensee personnel, the inspector determined that no significant changes had been initiated and/or completed at the facility since the last inspection. The procedure for change authorizations at the NTR was required to be completed for all changes at the facility. The inspector verified that administrative controls were in place that required the appropriate review and approval of all changes prior to implementation. The RC manager determines whether change authorizations need to be reviewed by the VTSC based on the complexity of the project and the relation to the safety of the reactor and the staff supporting operations. Engineering release forms are also completed to inform operations personnel of operating information and to document NTR activities which are not recorded in the operating log book. The engineering releases are normally approved by an SRO or the NTR manager and the RC manager.

c. Conclusions

Based on the records reviewed, the inspector determined that the licensee's design change program was being implemented as required.

6. Committees, Audits and Reviews

a. Inspection Scope (IP 69001)

In order to verify that the licensee had established and conducted reviews and audits as required in TS Section 6.2, the inspector reviewed selected aspects of:

- VTSC meeting minutes for August 8 and November 8, 2005; and February 2, May 4, September 22, and October 26, 2006
- Audit Report "Regulatory Compliance Review of Technical Specifications 6.4 and 6.7" dated November 29, 2006
- Audit Report "Regulatory Compliance Review of Technical Specifications 6.1, 6.5, and 6.6" dated November 20, 2006
- Audit Report "Regulatory Compliance Review of License Conditions" dated June 14, 2006
- Audit Report "Regulatory Compliance Review of Technical Specifications 3.2 and 4.2" dated March 16, 2006
- Audit Report "Regulatory Compliance Review of Technical Specifications 3.5 and 4.5" dated December 2, 2005
- VSS 1.1, "Charter - Vallecitos Technological Safety Council," Revision 7, dated August 1994, PCN dated January 26, 2004
- VSS 1.1.1, "VTSC Membership Listing," Revision 17, dated June 2006

b. Observations and Findings

The licensee has established the VTSC for all activities at the VNC. A charter is established for the VTSC and the inspector verified that the council is following all aspects of the charter. The VTSC membership satisfied the council's procedural rules. The VTSC had quarterly meetings and a quorum was always present as required. Review of the minutes indicated the VTSC provided guidance, direction and oversight, and ensured suitable use of the reactor. The minutes provided an acceptable record of VTSC review functions and of VTSC safety oversight of reactor operations.

Operations audits were performed by an RC staff member and met the TS requirements. The audits appeared to be acceptable. The inspector noted that the safety reviews and audits, and the associated findings, were acceptably detailed and that the NTR staff were supportive of the audits. During review of the audits, the inspector noted that the licensee immediately corrected any minor issues. The audits did not identify any issues related to the safe operation of the NTR facility. The inspector also reviewed the licensee's audit tracking system (ATS), which contained items that needed to be corrected. All of the items listed in the ATS assigned to the NTR were for non-license related activities or minor updates to procedures.

c. Conclusions

Review and oversight functions required by the TS were acceptably completed by the VTSC and the office of regulatory compliance.

7. Follow-up on Previous Open Items

a. Inspection Scope (IP 69001)

The inspector reviewed the actions taken by the licensee following identification of a Inspection Follow-up Item (IFI) during a previous inspection.

b. Observations and Findings

- (1) IFI 50-73/2001-201-01 - Follow-up on the licensee's efforts to ensure the completion of routine monthly tests of the High-Level Conference Circuit (HICON) at the site.

NRC Inspection Report No. 50-73/2001-201, dated June 22, 2001, outlined the situation. During that inspection, the inspector noted that Appendix B of the VNC Emergency Plan (E-Plan) requires that the High-Level Conference Circuit (HICON) be tested monthly and that the Central Alarm Station (CAS) operator document the status of the circuit and completion of the test. The inspector reviewed the records documenting the testing of the HICON from the beginning of 2000 to the present. It was noted that there was apparently no test of the HICON during July 2000 because the CAS operator had not documented the completion of the test as required.

While reviewing this issue, the inspector noted that E-Plan Section 10.3 states, "Equipment which is not used routinely such as the HICON phone is tested on a regular basis." During this inspection, the inspector noted that the licensee was conducting a test of this system on a monthly basis. The inspector noted that the testing of the HICON system indicate that there are no problems with this system. This issue is considered closed.

c. Conclusions

The issue regarding the failure to conduct tests of the HICON system as required by the E-Plan was closed.

8. Exit Interview

The inspection scope and results were summarized on January 11, 2007, with members of licensee management. A follow-up discussion of several issues was held via teleconference on January 19, 2007. The inspector described the areas inspected and discussed in detail the inspection findings. No dissenting comments were received from the licensee. Although proprietary information was reviewed during the inspection, no such material is included in this report.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

J. Ayala, Specialist, Radiation Monitoring
C. Bassett, Manager, Facilities and Quality Assurance
D. Boorn, Process Instrument Technician
E. Ehrlich, Senior Reactor Operator
L. Martin, Manager, Regulatory Compliance and EHS
R. Pearson, Radiation Monitoring Technician - Facilities Protection
T. Peterson, Reactor Operator
H. Stuart, Specialist, Radiological Engineering
D. Thomas, Manager, Nuclear Test Reactor
D. Turner, Manager, Vallecitos Nuclear Center

INSPECTION PROCEDURE USED

IP 69001: Class II Non-Power Reactors

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None

Closed

50-73/2001-201-01	IFI	Follow-up on the licensee's efforts to ensure the completion of routine monthly tests of the High-Level Conference Circuit (HICON) at the site.
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LIST OF ACRONYMS USED

ADAMS	Agencywide Documents Access and Management System
ALARA	As Low As Reasonably Achievable
ATS	Audit Tracking System
CFR	Code of Federal Regulations
CHRIS	Cable-Held Retractable Irradiation System
E-Plan	Emergency Plan
EHS	Environmental Health and Safety
FM	Facility Manager
GE	General Electric
HICON	High-Level Conference Circuit
IFI	Inspection Follow-up Item
IP	Inspection Procedure

kW	kilowatt
MREM	Millirem
NRC	Nuclear Regulatory Commission
NTR	Nuclear Test Reactor
OSLD	Optically Stimulated Luminescence Dosimeter
RC	Regulatory Compliance
RO	Reactor Operator
SRO	Senior Reactor Operator
TS	Technical Specification
VNC	Vallecitos Nuclear Center
VSS	Vallecitos Safety Standards
VTSC	Vallecitos Technological Safety Council