



U.S. NUCLEAR REGULATORY COMMISSION

STANDARD REVIEW PLAN

14.3.6 ELECTRICAL SYSTEMS - INSPECTIONS, TESTS, ANALYSES, AND ACCEPTANCE CRITERIA

REVIEW RESPONSIBILITIES

Primary - The organization responsible for electrical engineering review

Secondary - None

I. AREAS OF REVIEW

This Standard Review Plan (SRP) section addresses inspections, tests, analyses, and acceptance criteria (ITAAC) related to the station electrical systems. ITAAC information is contained in the final safety analysis report (FSAR) of a combined license (COL) application or Tier 1 information from the design control document (DCD) of a design certification (DC) application. This SRP section primarily involves the entire station electrical system, including Class 1E portions of the system, equipment qualification (EQ), major portions of the non-Class 1E system, and portions of the plant lightning protection, grounding, and lighting systems. The development of ITAAC for evolutionary plants that typically involve a significant amount of reliance on alternating current (AC) electrical systems for accomplishing safety functions may be much different from that for passive plant designs that involve much less reliance on AC electrical systems for accomplishing safety functions.

The specific areas of review are as follows:

1. The organization responsible for electrical engineering reviews the Tier 1 portion of the DCD submitted by the applicant. The organization responsible for electrical engineering

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USNRC STANDARD REVIEW PLAN

This Standard Review Plan, NUREG-0800, has been prepared to establish criteria that the U.S. Nuclear Regulatory Commission staff responsible for the review of applications to construct and operate nuclear power plants intends to use in evaluating whether an applicant/licensee meets the NRC's regulations. The Standard Review Plan is not a substitute for the NRC's regulations, and compliance with it is not required. However, an applicant is required to identify differences between the design features, analytical techniques, and procedural measures proposed for its facility and the SRP acceptance criteria and evaluate how the proposed alternatives to the SRP acceptance criteria provide an acceptable method of complying with the NRC regulations.

The standard review plan sections are numbered in accordance with corresponding sections in Regulatory Guide 1.70, "Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (LWR Edition)." Not all sections of Regulatory Guide 1.70 have a corresponding review plan section. The SRP sections applicable to a combined license application for a new light-water reactor (LWR) are based on Regulatory Guide 1.206, "Combined License Applications for Nuclear Power Plants (LWR Edition)."

These documents are made available to the public as part of the NRC's policy to inform the nuclear industry and the general public of regulatory procedures and policies. Individual sections of NUREG-0800 will be revised periodically, as appropriate, to accommodate comments and to reflect new information and experience. Comments may be submitted electronically by email to NRR_SRP@nrc.gov.

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has primary review responsibility for the station electrical systems in Tier 1. Review responsibilities may be consistent with those contained in Appendix A to SRP Section 14.3. The scope of the electrical review includes the entire Class 1E portion of the electrical system as well as a major portion of the non-Class 1E electrical system. It also includes portions of the plant lightning protection, grounding, and lighting systems.

2. The organization responsible for electrical engineering has the responsibility for the review of Tier 1 information regarding qualification of equipment to withstand harsh environments.
3. The organization responsible for electrical engineering has responsibility for the review of selected definitions, interface requirements of the standard design with the site, and site parameters for the design, that pertain to electrical issues.
4. The organization responsible for electrical engineering performs related reviews and coordination activities, as requested by other organizations, for Tier 1 systems using Class 1E power.
5. **For a DC application:**
 - A. The staff reviews the proposed ITAAC that are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, a plant that incorporates the design certification is built and will operate in accordance with the design certification, the Atomic Energy Act, and the NRC's regulations.
 - B. The staff reviews the justification that compliance with the interface requirements is verifiable through ITAAC. The staff also reviews the method that is to be used for verification of the interface requirements.
6. **For a COL application:**
 - A. The staff reviews the proposed ITAAC that are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, the facility has been constructed and will operate in conformity with the combined license, the Atomic Energy Act, and the NRC's regulations.
 - B. If the application references a standard design certification, the staff verifies that the ITAAC contained in the certified design apply to those portions of the facility design that are approved in the design certification.
7. COL Action Items and Certification Requirements and Restrictions. For a DC application, the review will also address COL action items and requirements and restrictions (e.g., interface requirements and site parameters).

For a COL application referencing a DC, a COL applicant must address COL action items (referred to as COL license information in certain DCs) included in the referenced DC. Additionally, a COL applicant must address requirements and restrictions (e.g., interface requirements and site parameters) included in the referenced DC.

Review Interfaces

Other listed SRP sections interface with this section as follows:

1. SRP Section 14.3 provides general guidance on ITAAC information and review interfaces.
2. Acceptability of ITAAC information regarding qualification of equipment for seismic environments is reviewed under SRP Section 14.3.2.

The specific acceptance criteria and review procedures are contained in the referenced SRP sections.

II. ACCEPTANCE CRITERIA

In establishing the top level requirements for the electrical design, the reviewer should use the Code of Federal Regulations (CFR) including the General Design Criteria (GDC) of Appendix A and Parts 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants," and 50.63, "Loss of All Alternating Current Power," as the main bases. In addition, the Institute of Electrical and Electronics Engineers (IEEE) nuclear standards should be used, as appropriate, to further establish top level requirements. These are discussed below. The reviewer should use the review checklists provided in Appendix C to SRP Section 14.3 as an aid for establishing consistency and comprehensiveness in the review of the systems. Also, the reviewer should consider significant lessons learned from operating experience problems and insights gained from the probabilistic risk assessment (PRA) for the standard design.

Requirements

Acceptance criteria are based on meeting the relevant requirements of the following Commission regulations:

1. 10 CFR 52.47(b)(1), which requires that a DC application contain the proposed inspections, tests, analyses, and acceptance criteria (ITAAC) that are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, a plant that incorporates the design certification is built and will operate in accordance with the design certification, the provisions of the Atomic Energy Act, and the NRC's regulations;
2. 10 CFR 52.80(a), which requires that a COL application contain the proposed inspections, tests, and analyses, including those applicable to emergency planning, that the licensee shall perform, and the acceptance criteria that are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, the facility has been constructed and will operate in conformity with the combined license, the provisions of the Atomic Energy Act, and the NRC's regulations.
3. GDC 17, in part, requires that an onsite and an offsite electric power systems be provided to permit functioning of SSCs important to safety. It further requires that the onsite electric power system have independence and redundancy and the electric power

supplied by the offsite system be supplied by two physically independent circuits. Also, GDC 17 requires that provisions be included to minimize the likelihood of losing all electric power as a result of or coincident with, loss of power generated by the nuclear power unit, from the transmission network, or the onsite electric power supplies.

4. 10 CFR 50.49 as it relates to EQ of electrical equipment important to safety for nuclear power plants. Applicants must ensure that safety-related, certain nonsafety-related, and certain post-accident monitoring equipment can perform their intended functions in various anticipated environments.
6. 10 CFR 50.63 requires that a nuclear power plant be able to withstand and recover from a station blackout event.

SRP Acceptance Criteria

Specific SRP acceptance criteria acceptable to meet the relevant requirements of the NRC's regulations identified above are as follows for the review described in this SRP section. The SRP is not a substitute for the NRC's regulations, and compliance with it is not required. However, an applicant is required to identify differences between the design features, analytical techniques, and procedural measures proposed for its facility and the SRP acceptance criteria and evaluate how the proposed alternatives to the SRP acceptance criteria provide acceptable methods of compliance with the NRC regulations.

The staff's review of the standard plant is conducted to ensure, in part, that Tier 1 contains top level design, fabrication, testing, and performance requirements for SSCs important to safety. Design Descriptions and ITAAC should be established to verify that these top level requirements (or design commitments) are met when the plant is built. IEEE nuclear standards should be used, as appropriate, to further establish top level requirements. IEEE Std. 308, "IEEE Standard Criteria for Class 1E power Systems for Nuclear Power Generating Stations," in conjunction with other related IEEE standards, establish specific design criteria for nuclear power plant electrical systems and equipment.

The standard design Class 1E electrical systems may include: (1) the Class 1E electrical power distribution system, (2) the emergency diesel generators (EDGs), (3) the Class 1E direct current power supply, and (4) the Class 1E vital ac and Class 1E instrument and control power supplies. Using the above regulations, IEEE standards, operating experience, and PRA as its bases, the applicant should establish top-level design commitments for the Class 1E electrical systems of the standard design to be included in the design descriptions and verified by ITAAC. The top-level design commitments for the Class 1E electrical systems include design aspects related to:

1. Equipment qualification for seismic and harsh environment

To ensure that the seismic design requirements of GDC 2 and the EQ requirements of 10 CFR 50.49 have been adequately addressed, a "basis configuration" standard ITAAC may be established for applicable systems to verify these design aspects of electrical equipment important to safety.

The Design Description should identify that Class 1E equipment is seismic Category 1 and equipment located in a harsh environment is qualified. The basic configuration standard ITAAC may be used to verify these areas.

EQ of safe-shutdown equipment may be verified as part of the basic configuration ITAAC for safety-related systems. EQ treatment in the ITAAC would then be discussed in the General Provisions section of Tier 1. Verification may include type tests or a combination of type tests and analyses of Class 1E electrical equipment identified in the Design Description or accompanying figures to show that the equipment can withstand the conditions associated with a design basis accident without loss of safety function for the time that the function is needed.

Qualification of systems and components for seismic and harsh environments should be verified by ITAAC. Electrical equipment located in a "mild" environment should be discussed in the applicable sections of the COL application only. An exception is made for state-of-the-art digital instrumentation and control (I&C) equipment and digital control and protection systems located in an "other than harsh" environment. Operational experience has shown these state-of-the-art equipment and systems to be sensitive to temperature. ITAAC should be included to verify the qualification of equipment whose performance may be impacted by sensitivity to particular environmental conditions not considered by regulations to be harsh.

2. Redundancy and independence

To ensure that the Class 1E electric systems meet the single failure requirements of GDC 17 (and other GDC), ITAAC may be established to verify the redundancy and independence of the Class 1E portion of the electrical design. For the electrical systems, ITAAC should verify the Class 1E divisional assignments and independence of electric power by both inspections and tests. The independence may be established by both electrical isolation and physical separation. Identification of the Class 1E divisional equipment should be included to aid in demonstrating the separation. (The detailed requirements are specified in Tier 2. For example, separation distances and identification are outlined in Tier 2). These attributes should be verified all the way to the electrically powered loads by a combination of the electrical system ITAAC and the ITAAC of the individual fluid, I&C, and heating, ventilation and air conditioning (HVAC) systems which also cover the electrical independence and divisional power supply requirements.

ITAAC should be included to verify adequate separation, required inter-ties (if any), required identification (e.g., color coding), proper routing/termination (i.e., location), separation of non-Class 1E loads from 1E buses. Post-fire safe shutdown separation of electrical circuits should be addressed in the fire protection system ITAAC.

3. Capacity and Capability

To ensure that the electrical systems have the capacity and capability to supply the safety-related electrical loads, ITAAC **should** be established to verify the adequate sizing of the electrical system equipment and its ability to respond (e.g., automatically in the times needed to support the accident analyses) to postulated events. This includes the Class 1E portion and the non-Class 1E portion to the extent that it is involved in supporting the Class 1E system.

ITAAC should be included to analyze the as-built electrical system and installed equipment (diesel generators, transformers, switchgear, batteries, etc.) to verify its ability to power the loads. In addition, the ITAAC should also include tests to

demonstrate the operation of the equipment. Testing should be included in ITAAC to verify EDG capacity and capability based on the Technical Specifications. In some cases regulatory guidance specifies the need for margin in capacity to allow for future load growth. If it is only for future load growth, ITAAC does not need to check for the additional margin.

ITAAC should be developed to verify the initiation of the Class 1E equipment necessary to mitigate postulated events for which the equipment is credited (e.g., loss of coolant accident (LOCA), loss of offsite power (LOOP), and degraded voltage conditions).

ITAAC should be included to analyze the as-built electrical power system for its response to a LOCA, LOOP, combinations of LOCA and LOOP (including LOCA with delayed LOOP and LOOP with delayed LOCA), and degraded voltage, including tests to demonstrate the actuation of the electrical equipment in response to postulated events.

Analyses to demonstrate the acceptability of a voltage drop should be included in ITAAC to verify adequacy for supporting the accomplishment of a direct safety function. The applicable section of the COL application should include a discussion of how the voltage analyses will be performed, i.e., reference to industry standards. Testing should be included in ITAAC to verify the EDG voltage and frequency response is acceptable and is the same as that specified in the Technical Specifications.

4. Electrical protection features

To ensure that the electrical power system is protected against potential electrical faults, ITAAC should be established to verify the adequacy of the electrical circuit protection included in the design. Operating experience and NRC Electrical Distribution System Functional Inspections (EDSFIs) have indicated some problems with the short circuit rating of some electrical equipment and breaker and protective device coordination. Inclusion in ITAAC should be based on the potential for preventing safety functions and the operating experience.

ITAAC should be included to analyze the as-built electrical system equipment for its ability to withstand and clear electrical faults. ITAAC should also be included to analyze the protection feature coordination to verify its ability to limit the loss of equipment due to postulated faults. Equipment short circuit capability and breaker coordination should be verified by specifying ITAAC for analyses. The description of the analyses should be included in the applicable section of the application. Similarly, diesel generator protective trips (and bypasses if applicable) should be considered.

5. Displays/controls/alarms

To help ensure that the electrical power system is available when required, ITAAC should be included to verify the existence of monitoring and controls for the electrical equipment. The minimum set of displays, alarms, and controls is based on the emergency procedure guidelines. In some cases, additional displays, alarms, and controls may be specified based on special considerations in the design and/or operating experience.

ITAAC should be included to inspect for the ability to retrieve the information (displays and alarms), and to control the electrical power system in the main control room and/or

at locations provided for remote shutdown. Detection of undervoltage conditions along with the starting and loading of EDG should be included in ITAAC. This is a direct safety function in response to design basis event of loss of power. Problems with relay settings should be considered in this requirement.

Other Electrical Equipment Important to Safety

In addition to the Class 1E systems addressed above, other aspects of the electrical design that are deemed to be important to safety and the top-level design commitments are included in Tier 1.

1. Offsite Power

To ensure that the requirements of GDC 17 for the adequacy and independence of the preferred offsite power sources within the standard design scope were met, ITAAC should verify the capacity and capability of the offsite sources to feed the Class 1E divisions, and the independence of those sources.

ITAAC should be included to inspect the direct connection of the offsite sources to the Class 1E divisions and to inspect for the independence/separation of the offsite sources. ITAAC should be developed to inspect for appropriate lightning protection and grounding features.

In addition, the Design Description includes "interface" requirements for the portions of the offsite power outside of the standard design scope; however, no ITAAC are included for the interfaces. The interfaces define the requirements that the offsite portion of the design (that is out-of-scope) must meet to support and not degrade the in-scope design (See also Appendix A to SRP Section 14.3).

2. Containment Electrical Penetrations

To ensure the containment electrical penetrations (both those containing Class 1E circuits and those containing Non Class 1E circuits) do not fail due to electrical faults and potentially breach the containment, ITAAC should verify that all electrical containment penetrations are protected against postulated currents greater than their continuous current rating.

3. Alternate AC Power Source (if applicable)

To ensure the availability of the alternate AC (AAC) power source for station blackout events, ITAAC should be developed to verify, through inspection and testing, the AAC power source (combustion gas turbines, diesel generators, or hydro units) and its auxiliaries along with its independence from other AC sources.

4. Lighting

ITAAC should be included to verify the continuity of power sources for plant lighting systems to ensure that portions of the plant lighting remain available during accident scenarios and power failures. The basis for inclusion may be more related to defense-in-depth, support function, operating experience, or PRA rather than "accomplishing a direct safety function."

5. Electrical Power For Non-Safety Plant Systems

To ensure that electrical power is provided to support the non-safety plant systems, Design Descriptions cover portions of the non-Class 1E electrical systems. **ITAAC should be included to verify the functional arrangement of electrical power systems provided to support non-safety plant systems to the extent that those systems perform a significant safety function.**

Technical Rationale

The technical rationale for application of these requirements and/or SRP acceptance criteria to areas of review addressed by this SRP section is discussed in the following paragraphs:

1. Compliance with GDC 17, in part, requires that an onsite and an offsite electric power system be provided to permit functioning of structures, systems and components important to safety. It further requires that the onsite electric power system have independence and redundancy and the electric power supplied by the offsite system be supplied by two physically independent circuits. This provides a reasonable assurance that the facility will function reliably in the event of a fault in an area of the electrical design.
2. Compliance with 10 CFR 50.49 requires that certain electrical equipment be qualified for accident (referred to as harsh) environments. This provides a reasonable assurance that the equipment needed in the event of an accident will perform its intended function.
3. Compliance with 10 CFR 50.63 requires that a nuclear power plant be able to withstand and recover from a station blackout event. This ensures that the plant can withstand and recover from this event safely.
4. IEEE Std. 308, "IEEE Standard Criteria for Class 1E power Systems for Nuclear Power Generating Stations," in conjunction with other related IEEE standards, establishes specific design criteria for nuclear power plant electrical systems and equipment. This provides a reasonable assurance that the electrical systems will perform their intended function in the anticipated operational environment.

III. REVIEW PROCEDURES

The reviewer will select and emphasize material from the procedures described below, as may be appropriate for a particular case.

For each area of review specified in subsection I of this SRP section, the review procedure is identified below. These review procedures are based on the identified SRP acceptance criteria. For deviations from these specific acceptance criteria, the staff should review the applicant's evaluation of how the proposed alternatives to the SRP criteria provide an acceptable method of complying with the relevant NRC requirements identified in subsection II.

1. Follow the general procedures for review of Tier 1 contained in the Review Procedures section of SRP Section 14.3. Ensure that the DCD is consistent with Appendix A to SRP Section 14.3. Review responsibilities may be consistent with those in Appendix B to SRP Section 14.3.

2. Ensure that all Tier 1 information is consistent with Tier 2 information. Figures and diagrams should be reviewed to ensure that they accurately depict the functional arrangement and requirements of the systems. Reviewers should use the Review Checklists in Appendix C to SRP Section 14.3 as an aid in establishing consistent and comprehensive treatment of issues.
3. Ensure that the electrical systems are clearly described in Tier 1, including the key performance characteristics and safety functions of SSCs based on their safety significance.
4. The reviewer should ensure that appropriate guidance is provided to other branches such that electrical issues in Tier 1 are treated in a consistent manner among branches.
5. Ensure that the standard ITAAC entries related to electrical systems are included in the appropriate systems of the design. In particular, the reviewer should review the **basic configuration ITAAC for verification of EQ**. The reviewer should coordinate with **the organization responsible for the review of SRP Section 3.2.1** for the review of seismic qualification of electrical components in the basic configuration ITAAC. The reviewer should ensure consistent application and treatment of the standard ITAAC entries for divisional power supply, physical separation, and independence for electrical and I&C systems in Tier 1.
6. Ensure that design features from the resolutions of selected technical and policy issues are adequately addressed in Tier 1, based on safety significance. Ensure that the appropriate Commission guidance, requirements, bases and resolutions for these items are documented clearly in the SER.
7. Ensure that definitions, legends, interface requirements, and site parameters that pertain to electrical issues are treated consistently and appropriately in Tier 1.
9. For review of a DC application, the reviewer should follow the above procedures to verify that the design, including requirements and restrictions (e.g., interface requirements and site parameters), set forth in the final safety analysis report (FSAR) meets the acceptance criteria. DCs have referred to the FSAR as the design control document (DCD). The reviewer should also consider the appropriateness of identified COL action items. The reviewer may identify additional COL action items; however, to ensure these COL action items are addressed during a COL application, they should be added to the DC FSAR.

For review of a COL application, the scope of the review is dependent on whether the COL applicant references a DC, an early site permit (ESP) or other NRC approvals (e.g., manufacturing license, site suitability report or topical report).

10. **Implementation of ITAAC will be inspected in accordance with NRC Inspection Manual Chapter IMC-2503, "Construction Inspection Program - ITAAC Inspections."**

IV. EVALUATION FINDINGS

The reviewer verifies that the applicant has provided sufficient information and that the review and calculations (if applicable) support conclusions of the following type to be included in the staff's safety evaluation report. The reviewer also states the bases for those conclusions.

The reviewer verifies that sufficient information has been provided to satisfy SRP Section 14.3 and this SRP section, and concludes that the ITAAC is acceptable. A finding similar to that in the Evaluation Findings section of SRP Section 14.3 should be provided in a separate section of the SER.

For DC and COL reviews, the findings will also summarize the staff's evaluation of requirements and restrictions (e.g., interface requirements and site parameters) and COL action items relevant to this SRP section.

In addition, to the extent that the review is not discussed in other SER sections, the findings will summarize the staff's evaluation of the ITAAC, including design acceptance criteria, as applicable.

V. IMPLEMENTATION

The staff will use this SRP section in performing safety evaluations of DC applications and license applications submitted by applicants pursuant to 10 CFR Part 50 or 10 CFR Part 52. Except when the applicant proposes an acceptable alternative method for complying with specified portions of the Commission's regulations, the staff will use the method described herein to evaluate conformance with Commission regulations. The provisions of this SRP section apply to reviews of applications docketed six months or more after the date of issuance of this SRP section, unless superceded by a later revision.

VI. REFERENCES

1. 10 CFR 52.47, "Contents of Applications."
2. 10 CFR 52.80, "Contents of Applications."
3. 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants."
4. 10 CFR 50.63, "Loss of All Alternating Current Power."
5. 10 CFR Part 50, Appendix A, General Design Criterion 17, "Electric Power Systems."
6. NUREG-1503, "Final Safety Evaluation Report Related to the Certification of the Advanced Boiling Water Reactor," Volumes 1 and 2, July 1994.
7. NUREG-1462, "Final Safety Evaluation Report Related to the Certification of the System 80+ Design," Volumes 1 and 2, August 1994.
8. IEEE Std. 308-2001, "Standard Criteria for Class 1E Power Systems for Nuclear Power Generating Stations."
9. NRC Draft Regulatory Guide DG-1145, "Combined License Applications for Nuclear Power Plants (LWR Edition)," June 30, 2006.
10. SECY-06-0220, Final Rule to Update 10 CFR Part 52, "Licenses, Certifications, and Approvals for Nuclear Power Plants," October 31, 2006.

11. SECY-02-0077, Proposed Rule to Update 10 CFR Part 52, "Early Site Permits, Standard Design Certifications, and Combined Licenses for Nuclear Power Plants," May 8, 2002.
12. SECY-00-77, Certification of Two Evolutionary Designs, April 15, 1996.
13. NRC Inspection Manual Chapter IMC-2503, "Construction Inspection Program - ITAAC Inspections," issued April 26, 2006.

PAPERWORK REDUCTION ACT STATEMENT

The information collections contained in the Standard Review Plan are covered by the requirements of 10 CFR Part 50 and 10 CFR Part 52, and were approved by the Office of Management and Budget, approval number 3150-0011 and 3150-0151.

PUBLIC PROTECTION NOTIFICATION

The NRC may not conduct or sponsor, and a person is not required to respond to, a request for information or an information collection requirement unless the requesting document displays a currently valid OMB control number.

SRP Section 14.3.6
Description of Changes

This SRP section affirms the technical accuracy and adequacy of the guidance previously provided in (Draft) Revision 0, dated April 1996 of this SRP. See ADAMS accession number ML052070664.

In addition, this SRP section was administratively updated in accordance with NRR Office Instruction, LIC-200, Revision 1, "Standard Review Plan (SRP) Process." The revision also adds standard paragraphs to extend application of the updated SRP section to prospective submittals by applicants pursuant to 10 CFR Part 52.

The technical changes are incorporated in Revision 1, dated 2007:

Review Responsibilities - Reflects changes in review branches resulting from reorganization and branch consolidation. Change is reflected throughout the SRP.

I. AREAS OF REVIEW

1. Revised introductory paragraph for consistency with DG1145 Section C.II.2 § 14.3.6
2. Revised to reflect changes in review branches resulting from reorganization and branch consolidation. This change is reflected throughout the SRP.

II. ACCEPTANCE CRITERIA

1. Modified to include ITAAC for COL applications, as described in DG-1145 Section 14.3 and SECY 06-0220.
2. SRP Acceptance Criteria, Item 1, Equipment Qualification for Seismic and Harsh Environment. Added a paragraph describing the qualification of systems and components to achieve consistency with DG-1145 C.II.2, Appendix A.
3. SRP Acceptance Criteria, Item 2, "Redundancy and Independence." Added a paragraph to achieve consistency with DG-1145 C.II.2.
4. SRP Acceptance Criteria, Item 3, Capacity and Capability - In first sentence, the word "may" was changed to "should" to achieve consistency with DG-1145.
5. SRP Acceptance Criteria, Item 3, Capacity and Capability, modified second paragraph to achieve consistency with DG-1145 Appendix A.
6. SRP Acceptance Criteria, Item 3, Capacity and Capability, modified the third paragraph to achieve consistency with DG-1145.
7. SRP Acceptance Criteria, Item 3, Capacity and Capability, modified the fourth paragraph to achieve consistency with DG-1145.
8. SRP Acceptance Criteria, Item 3, Capacity and Capability, added a paragraph to include acceptability of voltage drop and EDG voltage and frequency response for consistency with DG-1145 C.II.2, Appendix A.

9. SRP Acceptance Criteria, Item 4, Electrical Protection Features, modified the first paragraph to achieve consistency with DG-1145, C.II.2, Appendix A.
10. SRP Acceptance Criteria, Item 4, Electrical Protection Features, modified the second paragraph to achieve consistency with DG-1145, C.II.2, Appendix A.
11. SRP Acceptance Criteria, Item 5, Displays / Controls / Alarms; modified the second paragraph to achieve consistency with DG-1145, C.II.2, Appendix A.
12. "Other Electrical Equipment Important to Safety," Item 1, Offsite Power; Revised second paragraph to achieve consistency with DG-1145.
13. "Other Electrical Equipment Important to Safety," Item 3, Combustion Turbine Generator; modified to achieve consistency with DG-1145.
14. "Other Electrical Equipment Important to Safety," Item 4, Lighting; modified to achieve consistency with DG-1145 C.II.2 and Appendix A to C.II.2.
15. "Other Electrical Equipment Important to Safety," Item 5, Electrical Power For Non-Safety Plant Systems; modified to achieve consistency with DG-1145, C.II.2.

III. REVIEW PROCEDURES

1. Revised Item 5, to reflect changes in review branches resulting from reorganization and branch consolidation.

IV. EVALUATION FINDINGS

None.

V. IMPLEMENTATION

None.

VI. REFERENCES

1. Revised Reference 8, to include date of IEEE Std. 308.
2. Added DG-1145, "Combined License Applications for Nuclear Power Plants (LWR Edition)," June 30, 2006.
3. Added SECY-06-0220, Final Rule to Update 10 CFR Part 52, "Licenses, Certifications, and Approvals for Nuclear Power Plants," October 31, 2006.
4. Added SECY-02-0077, Proposed Rule to Update 10 CFR Part 52, "Early Site Permits, Standard Design Certifications, and Combined Licenses for Nuclear Power Plants," May 8, 2002.
5. Added SECY-00-77, Certification of Two Evolutionary Designs, April 15, 1996.

