# **Appendix C**

# Construction Inspector Technical Proficiency-Level Training and Qualification Journal

# Prerequisite

Do not begin the activities or complete the courses in this qualification journal until you have completed the Basic Inspector Certification Journal. You may complete the General Proficiency requirements contained in Appendix B to MC 1252 concurrent with the Technical Proficiency requirements outlined in this journal.

# **Table of Contents**

Introd	luction	C-3
Requ	ired Technical Proficiency-Level Training Courses:	C-4
	<ol> <li>New Reactor Technology (one design only is required)</li> <li>Construction Inspection Program Course</li> <li>Codes and Inspection Course (completing a Codes and Inspection Coin only one discipline is required: Civil/Structural, Mechanical, Electrical Preoperational/Startup Testing)</li> </ol>	
Post (	Qualification Construction Inspector Training Courses	C-4
Requi	red Refresher Training	C-4
Const	(ISA-CI-5) Report Writing and Construction Inspection Program Information Management System (CIPIMS)	C-10 C-12 C-14
	ruction Inspector On-the-Job Activity	C-31 C-32
<u>Techr</u>	nical Proficiency-Level Signature Card and Certification	C-34
Form	1: Construction Inspector Technical Proficiency Level Equivalency Justification	C-36
Revisi	ion History Sheet	C-37

Issue Date: 02/06/07

#### Introduction

Note: Some of the Construction Inspection Program Inspection Manual Chapters (IMCs) and other guidance may still be in draft. Use the latest draft IMCs and guidance documents while fully discussing with your supervisor

The purpose of this qualification journal is to ensure that inspectors are qualified to perform inspections in accordance with the NRC's Construction Inspection Program. The minimum qualification standards provide the technical competencies required for successfully completing the Construction Inspection Program.

The requirements of this standard are applicable to all NRC personnel conducting inspections and assessments used to verify acceptability of construction activities and ITAAC determinations. Satisfactory and documented attainment of the requirements contained in this qualification standard will ensure that personnel possess the requisite competence to fulfill functional area duties and responsibilities.

This qualification standard is intended to identify the minimum technical competency requirements in four broad functional areas: civil/structural, mechanical, electrical/l&C and construction/start-up testing. Qualification as a construction inspector may be obtained in one or all functional areas. Inspectors not qualified in a particular functional area are not to perform independent implementation of inspection procedures applicable to that functional area. Initial qualification may be obtained in only one functional area. Other functional areas may be pursued after full qualification is completed via the Individual Development Plan, with supervisor approval. The expected knowledge and/or skill that an individual must meet in order to perform inspections and assessments in each functional area must be reviewed and approved by the management responsible for implementing the construction inspection program.

Technical competency can be achieved through successful completion of the required training or through demonstration of equivalency by evidence of formal classroom training, college education or previous experience. The basis for acceptance of equivalency must be documented and requires approval from both the inspector's Branch Chief and Division Director. Any training course substituted for a required course must be reviewed and approved by the NRC Training program office.

Satisfaction of the minimum standards in this document is not intended to result in an expert level of knowledge. Additional training or education beyond the requirements of this standard would be required to achieve the specialist expertise level necessary to perform detailed disciplinary reviews or assessments within any specific area. In all cases, responsible management shall determine whether an inspector is qualified to inspect specific areas. The qualification journal must provide traceable documentation to show that minimum requirements have been met for each individual functional area.

# Construction Inspector Technical Proficiency Formal Training Activities

# Required Construction Inspector Technical Proficiency-Level Training Courses (Note 1)

#### **All Functional Areas**

- Construction Inspection Program
- Reactor Technology

# Civil/Structural Functional Area

Civil/Structural Codes and Inspection Course (with Focus on ITAAC)

#### **Mechanical Functional Area**

Mechanical Codes and Inspection Course (with Focus on ITAAC)

#### **Electrical Functional Area**

• Electrical Codes and Inspection Course (with Focus on ITAAC, Including Digital I&C and fiber optics)

# Preoperational/Start-up Testing Functional Area

• Preoperational and Startup Testing Inspection Course (with Focus on ITAAC)

# **Post Qualification Construction Inspector Training Courses**

- Reactor Technology Courses (other designs)
- Welding/NonDestructive Examination (NDE) Inspection
- Geotechnical Inspection
- Concrete Inspection
- Structural Steel Inspection

#### Required Refresher Training

- None identified
- (Note 1): Courses listed are under development. Until those courses are available, other equivalent courses may be substituted for certification. However, their equivalency and use for certification must first be reviewed and approved by the NRC Region II Deputy Regional Administrator for Construction. Any course submitted for equivalency review should contain elements covering technical fundamentals, regulatory and code requirements, and techniques for inspection applicable to the subject matter being taught.

Construction Inspector Technical Proficiency Individual Study Activities

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**TOPIC:** (ISA-CI-1) Construction QA and Defect Reporting Requirements

**PURPOSE:** The purpose of this activity is to familiarize inspectors with the regulatory

provisions that require licensees to have a quality assurance program and to report deficiencies. In addition to inspections that support the determination that ITAAC have been met, additional inspections will be needed to provide assurance that the licensee's processes for oversight of construction activities are effective. The successful implementation of a comprehensive QA program by the licensee will be an important indicator of the licensee's ability to manage the various activities associated with a large construction project. This activity will highlight the relationship between Part 52 and the requirements in Part 50 that relate

to quality assurance and defect reporting.

**COMPETENCY** 

AREAS: INSPECTION

LEVEL OF

**EFFORT:** 24 hours

**REFERENCES:** 1. 10 CFR Part 50, Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants."

2. Inspection Procedure 35100, "Review of QA Manual."

- 3. 10 CFR 50.55, "Conditions of Construction Permits." (note: the applicable provisions of 10 CFR 50 have been incorporated into Part 52 by reference and not all the requirements of part 50.55 pertaining to construction permits will apply to Part 52)
- 4. 10 CFR Part 21, "Reporting of Defects and Noncompliance."
- 5. Regulatory Guide 1.28, "Quality Assurance Program Requirements (Design and Construction)."
- 6. ASME NQA-1, "Quality Assurance Requirements for Nuclear Facility Application."
- 7. NUREG 1055, "Improving Quality and the Assurance of Quality in the Design and Construction of Nuclear Power Plants, (A Report to Congress), March, 1987." (a hard copy of this report is available in each region)

# EVALUATION CRITERIA:

At the completion of this activity, and as determined by the supervisor, inspectors should be able to:

- 1. Explain how 10 CFR Parts 50 and 21 requirements are invoked by 10 CFR Part 52.
- 2. Identify the Appendix B criteria and describe how an inspection would verify that they are correctly implemented during construction activities.
- 3. Discuss the basis for the applicability Part 21, "Reporting of Defects and Noncompliance" to vendors and suppliers. Define a defect in the context of Part 21.
- 4. Describe the notification requirements of § 50.55(e) and explain the relationship of this part to 10 CFR Part 50 and Part 21.
- 5. Discuss the American National Standards (ANSI) and American Society of Engineers (ASME) guidance that is endorsed by Reg Guide 1.28 and explain the exceptions to the guidance (contained in Reg Guide 1.28) which are required to be met in order for a licensee or vendor to use the endorsement.
- 6. Describe the applicability of ASME NQA-1 and explain the content of the various parts.

#### TASKS:

- 1. Read § 52.83.
- Review Appendix B and Inspection Procedure 35100, "Review of QA Manual." Compare the criteria with the inspection attributes in the IP and pay particular attention to the IP guidance.
- 3. Review Part 21 with particular focus on the § 21.21 requirements for evaluating defects and for reporting them.
- 4. Read 50.55(e) and determine what actions must be completed by licensees to satisfy these requirements.
- 5. Review Reg Guide 1.28 and focus on the additions and modifications to ANSI/ASME NQA-1 identified in Section C, "Regulatory Position."
- 6. Review ASME NQA-1.
- 7. Read NUREG 1055 and note the root causes for past construction problems.
- 8. Meet with your supervisor, mentor, or a qualified construction

1252, Appendix C C-8 Issue Date: 02/06/07

inspector to discuss any questions you may have as a result of this activity. Discuss the answers to the questions listed under the Evaluation Criteria section of this study guide with your supervisor.

**DOCUMENTATION:** Qualification Signature Card, Item ISA-CI-1.

**TOPIC:** (ISA-CI-2) 10 CFR Part 52

**PURPOSE:** 

The purpose of this activity is to familiarize inspectors with the documents which form the bases and guidance required to support construction inspection activities. The Code of Federal Regulations (CFR) sets out the requirements for applying for a license to build and operate a nuclear power plant. In the past, the requirements, contained in 10 CFR Part 50, supported a two-phased licensing process. Under Part 50, the applicant first applied for and received a construction permit. Then, when plant construction was nearing completion, an application was submitted for a license to operate the plant. Under 10 CFR Part 52, which was first adopted in 1989, the licensing process was modified to one step, where an applicant can be granted a combined construction permit and operating license or a combined license (COL). This activity will provide you with knowledge of Part 52 and the licensing processes which will be used in the licensing of new reactors.

**COMPETENCY** 

**AREAS:** REGULATORY FRAMEWORK

LEVEL OF

**EFFORT:** 16 hours

#### **REFERENCES:**

- 1. 10 CFR Part 52, "Early Site Permits; Standard Design Certifications; And Combined Licenses For Nuclear Power Plants."
- 2. NUREG/BR-0298, "Nuclear Power Plant Licensing Process," Rev.2

# **EVALUATION**

CRITERIA:

At the completion of this activity, and as determined by the supervisor, inspectors should be able to:

- 1. Explain the basis and criteria for the licensing of nuclear plants.
- 2. Recognize and locate specific topics presented in 10 CFR Part 52.
- 3. Discuss the definitions of following terms used in 10 CFR Part 52:
  - a. early site permit
  - b. standard design
  - c. standard design certification
  - d. combined license

4. Discuss the importance of the inspections, tests, and analyses discussed in 10 CFR 52.47(b)(2) to the overall licensing of a new nuclear power plant. Explain why there are no inspections associated with standard design certification.

TASKS:

NOTE: 10 CFR Part 52 is undergoing a significant revision where some topics are being moved to different sections. The revisions to the rule will not be finalized and become effective until early 2007, and will likely undergo some additional changes in response to public comment. However, the relocation of topics is not likely to be affected. As a result, this activity uses the proposed rule for study rather than the current version of Part 52 to allow inspectors to become familiar with the new location of information. The proposed rule can be found in ADAMS at Accession Number ML061320560. If the rule has been finalized, the current information from the CFR should be used.

- 1. Read NUREG/BR-0298, "Nuclear Power Plant Licensing Process," Rev.2. This document is available on the NRC's external website and may be available in hard copy from each region's Office of Public Affairs. Compare the topics covered in the NUREG to the topics covered in Part 52.
- 2. Locate the section in Part 52 where Early Site Permits are discussed. Read all paragraphs related to Early Site Permits including § 52.17 which addresses the technical information that must be submitted by the applicant and is reviewed by the NRC. Then, review Inspection Manual Chapter 2501, Construction Inspection Program: Early Site Permits. Compare the inspections conducted under IMC-2501 to the ESP topics being evaluated by the NRC.
- 3. Review Subpart B on standard design certification. Pay particular attention to §52.47(b)(2), which discusses the inspections, tests, analyses and acceptance criteria of the design.
- 4. Review Subpart C on combined licenses. Pay particular attention to the title of § 52.79. Scan the technical information that must be in the application for a COL.
- 5. Meet with your supervisor, mentor, or a qualified construction inspector to discuss any questions you may have as a result of this activity. Discuss the answers to the questions listed under the Evaluation Criteria section of this study guide with your supervisor.

**DOCUMENTATION:** Qualification Signature Card, Item ISA-CI-2.

**TOPIC:** (ISA-CI-3) Construction Inspection Program

**PURPOSE:** The purpose of this activity is to introduce the requirements that form the

basis of the Construction Inspection Program. This activity will provide you with knowledge of the Manual Chapter requirements that will be

implemented to inspect all phases of new reactor construction.

**COMPETENCY** 

**AREAS:** REGULATORY FRAMEWORK

LEVEL OF

**EFFORT:** 24 hours

REFERENCES:

1. MC 2501, "Early Site Permits"

2. MC 2502, "COL Application"

3. MC 2503, "Construction Inspection Program - ITAAC"

4. MC 2504, "Construction Inspection Program - Non ITAAC

5. IP 65001, "Inspections, Tests, Analyses and Acceptance Criteria (ITAAC) Matrix Inspections"

# EVALUATION CRITERIA:

At the completion of this activity, and as determined by the supervisor, inspectors should be able to:

- 1. Discuss the program requirements contained in the construction inspection program Manual Chapters and in IP 65001.
- Relate the requirements of the CFR to the inspection objectives of the construction inspection program by comparing and contrasting the various aspects of the licensing process to the Inspection Manual Chapters 2501, 2502, 2503 and 2504 of the construction inspection program.
- Discuss how inspection information will be used to support the Commission decision that the licensee can load fuel and operate the plant.

**TASKS:** 1. Review MC 2501, 2502, 2503 and 2504 and IP 65001.

2. Review § 52.97 on issuance of a combined license. Compare the inspections in IMC-2504 to the topical areas in § 52.97.

- 3. Read § 52.99 on inspections during construction. Pay particular attention to the requirements of § 52.99(2)(e).
- 4. Read § 52.103 on operation under a combined license. Pay particular attention to § 52.103(g).
- 5. Meet with your supervisor, mentor, or a qualified construction inspector to discuss any questions you may have as a result of this activity. Discuss the answers to the questions listed under the Evaluation Criteria section of this study guide with your supervisor.

**DOCUMENTATION:** Qualification Signature Card, Item ISA-CI-3.

**TOPIC:** (ISA-CI-4) Design Control Document

**PURPOSE:** 

The purpose of this activity is for you to learn about a key document related to the licensing and construction of a new plant under 10 CFR Part 52. A design control document (DCD) is a repository of information about a standard design certified under 10 CFR Part 52. Under Part 52, a reactor design can be submitted to the NRC for review and approval, even if there are no applicants to build the plant. The vendor submits sufficient design information such that the staff can make an overall determination of the design adequacy as it relates to safety and risk. Typically, the information needed to certify a design accounts includes the principal plant systems and components and their corresponding inspections, tests, analysis, and acceptance criteria (ITAACS). addition to the detailed design information, the vendor also provides information about the site parameters (the postulated physical, environmental, and demographic features of an assumed site) for a plant and the other system interfaces that will be needed to support the safe operation of the reactor. Part 52 also requires the vendor to provide other types of information for use in the future when an applicant references an already certified design.

This ISA focuses on the AP1000 and ABWR DCDs with the intent of highlighting the differences in these documents. This approach will show how these and future DCDs may vary and should sensitize the inspector to the differences that can be encountered from various certified designs.

**COMPETENCY** 

**AREAS**: REGULATORY FRAMEWORK

LEVEL OF

**EFFORT:** 8 Hours per design

REFERENCES: 1. 10 CFR Part 52

2. Design Control Documents

#### **EVALUATION**

**CRITERIA:** 

At the conclusion of this activity, and as determined by the supervisor, the inspector should be able to:

- 1. Define the following terms:
  - a. Design Control Document (DCD)
  - b. Tier 1
  - c. Tier 2
  - d. Tier 2\*

- e. ITAAC
- f. Design Acceptance Criteria (DAC)
- 2. Explain the relationship between Tier 1 and Tier 2 information.
- 3. Describe the kind of information required to be provided by a COL applicant as described in the DCD.
- 4. Compare and contrast the methods required to make changes to the information contained in Tiers 1, 2 and 2\*.
- 5. Locate the ITAAC information and tables in the DCD.
- 6. Identify the kind of information available in the Tier 2 section of the DCD.
- Discuss in detail why DAC were used as part of design certification and demonstrate an understanding of the practical application of DAC.
- 8. Discuss in detail the different definitions of "as built" in each design. Describe how these differences can affect inspection planning.

#### TASKS:

- Locate the DCD for the AP1000 and the ABWR on the Construction Inspection Program webpage. This site can be accessed via ROP Digital City and the NRR websites.
- 2. Look at the organization of both DCDs.
- 3. Read the Introduction to the DCD section for each design.
- 4. Review Chapter 1 of Tier 1 for each design. Within the General Provisions sections take particular note of:
  - Treatment of individual items
  - Implementation of ITAAC
  - Verifications for Basic Configuration for Systems for ABWR

Consider how the differences between these sections may affect inspection planning.

- 5. In both DCDs review the content, format and presentation of the ITAAC .
- 6. Read the definitions for the terms listed in evaluation criterion 2 a-e, in both the AP1000 and the ABWR.
- 7. Review the content and format of Tier 2 for AP1000 and ABWR.

- 8. Go to NUREG-0800, the Standard Review Plan. Locate chapter 14.3. Read the description of Design Acceptance Criteria (DAC).
- 9. Review the ITAAC for Human Factors Engineering for both designs as related to DAC.
- 10. Meet with your supervisor, mentor, or a qualified construction inspector to discuss any questions you may have as a result of this activity. Discuss the answers to the questions listed under the Evaluation Criteria section of this study guide with your supervisor.

**DOCUMENTATION:** Qualification Signature Card, Item ISA-CI-4.

**TOPIC:** (ISA-CI-5) Report Writing and Construction Inspection Program

Information Management System (CIPIMS)

**PURPOSE:** The purpose of this activity is to familiarize inspectors with the program

requirements required to document construction inspection activities.

**COMPETENCY** 

**AREAS:** INSPECTION

LEVEL OF

**EFFORT:** 16 hours

REFERENCES: 1. Manual Chapter 0613, "Construction Inspection Reports."

2. CIPIMS User's Guide.

# **EVALUATION**

**CRITERIA:** At the completion of this activity, and as determined by the supervisor, inspectors should be able to:

1. Describe how inspection results will be documented and what a construction inspection report will look like.

- 2. Explain how inspection findings are characterized what the different types of findings are.
- Explain how CIPIMS is used in documenting inspection findings and in developing inspection reports. Describe the specific inputs into CIPIMS.

TASKS:

- 1. Read MC 0613 and understand the requirements for characterizing and documenting inspection findings.
- 2. Review the CIPIMS User's Guide and understand the requirements for documenting results in CIPIMS.
- 3. Meet with your supervisor, mentor, or a qualified construction inspector to discuss any questions you may have as a result of this activity. Discuss the answers to the questions listed under the Evaluation Criteria section of this study guide with your supervisor.

**DOCUMENTATION:** Qualification Signature Card, Item ISA-CI-5.

**TOPIC:** (ISA-CI-6) Industry Codes & Standards - General

**PURPOSE:** The purpose of this activity is to introduce you to some of the basic

Codes, NRC Regulatory Guides (RG) and associated industry standards commonly used during construction activities. The list is not inclusive but it does contain the major documents that will be utilized during

construction and fabrication.

**COMPETENCY** 

**AREAS:** INSPECTION

LEVEL OF

**EFFORT:** 40 hours

**REFERENCES:** See the attached list of References for ISA-CI-6.

# **EVALUATION**

CRITERIA:

At the completion of this activity, and as determined by the supervisor, inspectors should be able to :

- 1. Identify the general codes commonly used by construction inspectors and discuss the topics included in these codes.
- 2. Discuss the relationship between RG (guidelines) and industry standards (accepted methodologies).
- 3. Identify the RG and associated industry standards that address the 10CFR50, Appendix B, QA Criteria.
- 4. Discuss the topics included in the RG and industry standards associated with each

TASKS:

- 1. For the appropriate technical area, review each of the documents listed in the attached list of references.
- 2. Meet with your supervisor, mentor, or a qualified construction inspector to discuss any questions you may have as a result of this activity. Discuss the answers to the questions listed under the Evaluation Criteria section of this study guide with your supervisor.

**DOCUMENTATION:** Qualification Signature Card, Item ISA-CI-6.

### **REFERENCES for ISA-CI-6**

#### General

10 CFR Part 50 Appendix A, General Design Criteria for Nuclear Power Plants
 10 CFR 50.46 Acceptance Criteria for Emergency Core Cooling Systems for Light-Water Nuclear Power Reactors
 10 CFR 50.49 Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants

10 CFR 50.55a Codes and Standards

The American Society for Mechanical Engineers (ASME)

Section II - Materials Section III - Nuclear

Section V - Nondestructive Examination

Section VIII - Pressure Vessels

Section IX - Welding and Brazing Qualifications

Section XI - Inservice inspection

#### Civil/Structural

### **American Concrete Institute (ACI)**

ACI 117	Tolerances for Concrete Construction and Materials
ACI 214	Recommended Practice for Evaluation of Strength Test Results of Concrete
ACI 301	Specifications for Structural Concrete
ACI 304	Measuring, Mixing, Transporting and Placing Concrete
ACI 305	Hot Weather Concreting
ACI 306	Cold Weather Concreting
ACI 304	Guide for Measuring, Mixing, Transporting and Placing Concrete
ACI 308	Curing Concrete
ACI 309	Consolidation of Concrete
ACI 311	Recommended Practice for Concrete Inspection
ACI 318	Building Code Requirements for Reinforced Concrete
ACI 349-0	01 Code Requirements for Nuclear Safety Related Concrete Structures
ACLSP-2	Manual of Concrete Inspection

# **American Institute of Steel Construction (AISC)**

M011	Manual of Steel Construction
S326	Specification for the Design, Fabrication, and Erection of Structural Steel for Buildings

# **American National Standards Institute (ANSI)**

N45.2.5 Supplemental QA Requirements for Installation, Inspection, and Testing of Structural Concrete and Structural Steel

# American Society for Testing and Materials (ASTM)

A 36 A 325 A 490	Specification for Structural Steel Specification for High-Strength Bolts for Structural Steel Joints Specification for Heat Treated, Steel Structural Bolts, 150 ksi Tensile
	Strength
C 29	Unit Weight and Voids in Aggregates
C 94	Specification for Ready-Mixed Concrete
C 172	Method of Sampling Freshly Mixed Concrete
C 1077	Practice for Laboratories Testing concrete and Concrete Aggregates for Use in Construction and Criteria for Laboratory Evaluation
D 422	Method for Particle-Size Analysis of Soils
D 1556	Test Method for Density of Soil in Place by the Sand-Cone Method
D 2167	Test Method for Density and Unit Weight of Soil In-Place by the Rubber Balloon Method
D 2922	Test Methods for Density of Soil and Soil-Aggregate in Place by Nuclear Methods
D 3017	Moisture Content of Soil and soil Aggregate in Place by Nuclear Methods
D 3740	Practice for Evaluation of Agencies Engaged in Testing and/or Inspection of Soil and Rock as Used in Engineering Design and Construction
E 329	Evaluation of Testing and Inspection Agencies

# **American Welding Society (AWS)**

AWS DI.1	Structural Welding Code - Steel
<b>AWS B2.1</b>	Specification for Welding Procedure and Performance Qualification

# The International Organization for Standardization (ISO)

ISO 14731 - Welding Coordination - Tasks and Responsibilities

# **Nuclear Regulatory Commission Regulatory Guides (RG)**

RG 1.107	Qualifications for Cement Grouting for Prestressing Tendons in Containment
	Structures
RG 1.127	Inspection of Water-Control Structures Associated with Nuclear Power Plants
RG 1.132	Site Investigations for Foundations of Nuclear Power Plants

RG 1.136	Materials, Construction, and Testing of Concrete Containments (Articles
	CC-1000, -2000, and -4000 through -6000 of the Code for Concrete Reactor
	Vessels and Containments)
RG 1.138	Laboratory Investigations of Soils for Engineering Analysis and Design of
	Nuclear Power Plants
RG 1.142	Safety Related Concrete Structures for Nuclear Power Plants

# **Concrete Reinforcing Steel Institute (CRSI)**

MSP-1 Manual of Standard Practice

# **Portland Cement Association (PCA)**

EB001 Design and Control of Concrete Mixtures

### Mechanical

# **American Society of Mechanical Engineers (ASME)**

**B&PV Code** 

Section III, Division 1 Nuclear Power Plant Components

B&PV Code

Section III, Division 2 Concrete Reactor Vessels and Containments

# **Nuclear Regulatory Commission Regulatory Guides (RG)**

RG 1.27 RG 1.31 RG 1.43 RG 1.44 RG 1.50 RG 1.54	Ultimate Heat Sink for Nuclear Power Plants Control of Ferrite Content in Stainless Steel Weld Metal Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components Control of the Use of Sensitized Stainless Steel Control of Preheat Temperature for Welding of Low-Alloy Steel Service Level I, II, and III Protective Coatings Applied to Nuclear Power
110 1.04	Plants
RG 1.71	Welder Qualification for Areas of Limited Accessibility
RG 1.84	ASME Code Case Applicability
RG 1.87	Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors (Supplement to ASME Section III Code Cases 1592, 1593, 1594, 1595, and 1596)
RG 1.96	Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water Reactor Nuclear Power Plants
RG 1.100	Seismic Qualification of Electrical and Mechanical Equipment for Nuclear Power Plants
RG 1.116	QA Requirements for Installation, Inspection and Testing of Mechanical Equipment and Systems

 RG 1.150 Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations

# **Electrical**

# Institute of Electrical and Electronics Engineers

315	Graphic Symbols for Electrical and Electronics Diagrams
336	Installation, Inspection, and Testing Requirements for Power,
	Instrumentation, and Control Equipment at Nuclear Facilities
338	IEEE Standard Criteria for Periodic Testing of Nuclear Power Generating
	Station Class 1E Power and Protection Systems
1050	Guide for Instrument and Control Equipment Grounding in Generating
	Stations

# **Nuclear Regulatory Commission Regulatory Guides (RG)**

RG 1.6	Independence Between Redundant Standby (Onsite) Power Sources and Between Their Distribution Systems
RG 1.11	Instrument Lines Penetrating Primary Containment
RG 1.30	Quality Assurance Requirements for the Installation, Inspection, and Testing of Instrumentation and Electric Equipment (ANSI N45.2.4/IEEE 336)
RG 1.32	Criteria for Safety-Related Electric Power Systems for Nuclear Power Plants (IEEE 308)
RG 1.40	Qualification Tests of Continuous-Duty Motors Installed Inside the Containment of Water-Cooled Nuclear Power Plants (IEEE 334)
RG 1.47	Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems
RG 1.53	Application of the Single-Failure Criterion to Nuclear Power Plant Protection Systems (IEEE 279 and IEEE 379)
RG 1.63	Electric Penetration Assemblies in Containment Structures for Light-Water-Cooled Nuclear Power Plants (IEEE 317)
RG 1.73	Qualification Tests of Electric Valve Operators Installed Inside the Containment of Nuclear Power Plants
RG 1.75	Physical Independence of Electric Systems (IEEE 384)
RG 1.81	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power Plants
RG 1.89	Qualification of Class 1E Equipment for Nuclear Power Plants (IEEE 323)
RG 1.97	Instrumentation for Light -Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident
RG 1.100	Seismic Qualification of Electrical and Mechanical Equipment for Nuclear Power Plants
RG 1.105	Instrument Set points (ISA S67.04)
RG 1.106	Thermal Overload Protection for Electric Motors on Motor-Operated Valves
RG 1.128	Installation Design and Installation of Large Lead Storage Batteries for Nuclear Power Plants (IEEE 484)

RG 1.129	Maintenance, Testing, and Replacement of Large Lead Storage Batteries for
	Nuclear Power Plants (IEEE 450)
RG 1.131	Qualification Tests of Electric Cables, Field Splices, and Connections for
	Light-Water-Cooled Nuclear Power Plants (IEEE 383)
RG 1.151	Instrument Sensing Lines (ISA S67.02)
RG 1.180	Guidelines for Evaluating Electromagnetic and Radio-Frequency Interference
	in Safety-Related Instrumentation and Control Systems

# <u>Testing</u>

# **Nuclear Regulatory Commission Regulatory Guides (RG)**

RG 1.68	Initial Test Programs for Water-Cooled Nuclear Power Plants
RG 1.79	Preoperational Testing of Emergency Core Cooling Systems for Pressurized
	Water Reactors

**TOPIC:** (ISA-ASME-1) Regulatory Basis for the Requirements Specified in

the American Society of Mechanical Engineers (ASME) Boiler &

Pressure Vessel Code (Code).

**PURPOSE:** The purpose of this activity is to familiarize you with the federal

regulations which incorporate by reference the requirements of the ASME Code for design, fabrication, erection, testing, and inspection of the reactor coolant system and associated auxiliary, control, and

protection systems.

**COMPETENCY** 

**AREA:** REGULATORY FRAMEWORK

LEVEL OF EFFORT:

40 Hours

REFERENCES:

1. 10 CFR 50.55a

2. 10 CFR 50, Appendix A, General Design Criteria (GDC)

3. Proposed Rule for 10 CFR Part 52 (71 FR 12885)

# EVALUATION CRITERIA:

At the completion of this activity the inspector should be able to:

- 1. Identify the sections in the Code of Federal Regulations (CFR) which mandate the use of the ASME Code for construction of the reactor coolant pressure boundary and associated auxiliary, control, and protection systems.
- Identify the Code editions and addenda that have been endorsed by the NRC staff relative to the following ASME Code Sections:
  - a. ASME III, Rules for Construction of Nuclear Facility Components
  - b. ASME XI, Rules for In-Service Inspection of Nuclear Plant Components
  - c. ASME OM Code, ASME Code for Operation and Maintenance of Nuclear Power Plants
- 3. Identify the locations of the listing of limitations and modifications which the NRC has imposed on the use of the Code editions currently endorsed by the staff.
- 4. Describe the requirements a licensee must follow before using proposed alternatives to the requirements of the ASME Code editions currently endorsed by the NRC.
- 5. Identify the ASME Code Class requirements specified in the

CFR for construction of components classified as NRC Quality Groups A, B, and C.

6. Explain how the ASME Code requirements are incorporated in the regulations for combined license applications and new reactor construction in accordance with 10 CFR Part 52.

TASKS:

- 1. Read the 10 CFR 50 Appendix A GDC, especially GDC 1, 14, 15, 30, 31, and 32, to become familiar with the minimum requirements established for the principal design criteria for the reactor coolant system for water-cooled nuclear power plants.
- 2. Read 10 CFR 50.55a(b) to understand how the ASME Code has become approved for incorporation by reference into the CFR.
- 3. Read 10 CFR 50.55a(b)(1), b(2), and b(3) to understand which editions and addenda of the ASME Code have been reviewed and endorsed by the NRC staff, including identification of any limitations imposed by the staff on their usage.
- 4. Read 10 CFR 50.55a(c), (d), and (e) to recognize the Code design requirements established for components categorized by the NRC as Quality Groups A, B, and C components.
- 5. Become familiar with the proposed rule for 10 CFR Part 52 published in the Federal Register (71 FR 12885). Specifically, read the proposed new rule in 10 CFR 52.79(a)(11), and note the incorporation of 10 CFR 50.55a into the Part 52 regulations for new construction.
- Meet with your supervisor, mentor, or a qualified construction inspector to discuss any questions you may have as a result of this activity. Discuss the answers to the questions listed under the Evaluation Criteria section of this study guide with your supervisor.

**DOCUMENTATION:** 

Qualification Signature Card, Item ISA-ASME-1

**TOPIC:** (ISA-ASME-2) American Society of Mechanical Engineers Boiler &

Pressure Vessel Code, Section III (ASME Code, or Section III)

Overview

**PURPOSE:** The ASME Code, Section III has been incorporated by reference into

the Code of Federal Regulations (CFR). 10 CFR 50.55a effectively mandates use of the ASME Code, Section III for design, construction, nondestructive examination, testing, code certification stamping, and overpressure protection of new components used in nuclear power plants. For this reason, it is necessary that inspectors have a general understanding of the scope of Section III, and the applicability and limitations of its rules. This general understanding will provide the foundation for more detailed understanding of selected Articles of Section III directly related to the inspector's duties. These insights will also facilitate the inspection of certain Part 52 COL ITAAC which requires some knowledge of ASME Code reports and documentation.

COMPETENCY

AREA: TECHNICAL AREA EXPERTISE REGULATORY FRAMEWORK

LEVEL OF EFFORT:

40 Hours

**REFERENCES:** 

- 1. ASME Code, Section III, Subsection NCA, General Requirements for Division 1 and Division 2
- 2. "Companion Guide to the ASME Boiler & Pressure Vessel Code; Criteria and Commentary on Select Aspects of the Boiler & Pressure Vessel and Piping Codes," Volumes 1, 2, and 3, edited by K.R. Rao, ASME Press, New York, 2006.
- 3. 10 CFR 50.55a, Codes and standards

# EVALUATION CRITERIA:

At the completion of this activity the inspector should be able to:

- 1. Identify and explain the differences in the various Divisions of the ASME Code, Section III.
- 2. Identify and discuss the limits of the ASME Code, Section III rules, including use of Code editions, Addenda, and Code Cases.
- 3. Identify the location of the ASME Code Glossary which defines selected terms commonly used throughout ASME III.

- 4. Identify the principal ASME Code-specified documents requiring certification, by either a Registered Professional Engineer or by the Authorized Nuclear Inspector, that a component has been constructed in accordance with Code requirements. Discuss the importance of these documents to inspections and closure of ASME Code-related ITAAC.
- Identify the ASME Code-specified document which establishes the design basis for a component, including loadings and load combinations, and defines appropriate Design, Service, and Test Limits for construction of ASME classified components.
- 6. Compare and contrast the ASME Code Service Limit A, B, C, and D provisions for damage allowed to a component while still satisfying the Code Service Limits A through D.
- 7. Identify the general duties of the Authorized Nuclear Inspector (ANI), and discuss some of the principal functions of the ANI in the certification process for Code component construction.
- 8. Explain how to access the electronic version of the ASME Boiler & Pressure Vessel Code from the NRC website.

#### TASKS:

- Access the ASME Boiler & Pressure Vessel Code on the NRC website by the following steps:
  - a. From the NRC home page select 'Information Resources'
  - b. Scroll down to the Codes & Standards section and select 'IHS Codes & Standards'
  - c. Follow the access directions by again selecting 'IHS Codes & Standards'
  - d. From the IHS menu select 'Specs & Standards'
  - e. From the 'Quick Searches' pull-down menu select 'Table of Contents'
  - f. Select the ASME Code edition desired
  - g. Select the ASME Code Section desired, e.g., ASME III
- Read ASME III, Articles NCA-1110 and NCA-1130 to understand the differences in types of items addressed by the rules of the various Divisions of the ASME Code, and to become aware of the basic limitations on applicability of these ASME Code rules.
- 3. Read ASME III, Article NCA-9000 to become aware of specific definitions of terms commonly used within the ASME III Code.
- 4. Read ASME III, Article NCA-1140 to understand the basis for

- use of Code editions, Addenda, and Code Cases, and to understand the impact that 10 CFR 50.55a has on their usage.
- 5. Read the following ASME III Articles to identify the ASME Code reports, and their basic provisions, required for certification that components have been constructed in accordance with Code requirements:
  - a. NCA-3250, Provision of Design Specifications
  - b. NCA-3260, Review of Design Report
  - c. NCA-3555. Certification of Design Report
  - d. NCA-3270, Overpressure Protection Report
  - e. NCA-3280, Owner's Data Report and Filing
  - f. NCA-3340, Design Drawings and Construction Specification Division 2)
  - g. NCA-3350, Design Report (Division 2)
  - h. NCA-3360, Certification of the Construction Specification, Design Drawings, and Design Report (Division 2)
  - i. NCA-3380, Certification of Construction Report (Division 2)
  - j. NCA-3454, Contents of the Construction Report (Division 2)
  - k. NCA-5290, Data Reports and Construction Reports (Divisions 1 and 2)
- 6. Read ASME III, Article NCA-2140 to identify the principal document, and its major provisions, for establishment of the design basis for ASME components.
- 7. Read ASME III, Article NCA-2142.4 to understand the differences between the various ASME Service Limits, and to be able to discuss the service-related component damage implications of each Service Limit.
- 8. Read ASME III, Article NCA-5200 to understand the requirements for ASME Code authorized inspections implemented by the Authorized Nuclear Inspector.
- 9. Read Chapter 5 of Reference 2 for a concise introduction to the general requirements of ASME Section III, Divisions 1 and 2. Other chapters of Reference 2 provide outline summaries for all other provisions of the ASME Code, and offer a usable source of Code-related information which can be used as a roadmap to the more detailed requirements contained in the various ASME Code Sections.

10. Meet with your supervisor, mentor, or a qualified construction inspector to discuss any questions you may have as a result of this activity. Discuss the answers to the questions listed under the Evaluation Criteria section of this study guide with your supervisor.

**DOCUMENTATION:** Qualification Signature Card, Item ISA-ASME-2

Construction Inspector Technical Proficiency
On-the-Job Activities

# **Construction Inspector On-the-Job Activity**

**TOPIC:** (OJT-CI-1) Inspection Accompaniment

**PURPOSE:** The purpose of this activity is to familiarize an inspector with a typical

construction inspection for a nuclear facility.

**COMPETENCY** 

AREA: INSPECTION

**LEVEL** 

**OF EFFORT:** 24 hours

**REFERENCES**: 1. Applicable Final Safety Analysis Report Sections

2. Applicable Inspection Procedures and Manual Chapters

3. ITAAC (as applicable)

# EVALUATION CRITERIA:

Upon completion of the tasks, the inspector should be able to:

- 1. Generally describe the construction requirements in the area being inspected.
- 2. Discuss the results of the inspection both in technical and regulatory contexts.

TASKS:

- 1. Review the references listed above to develop an understanding of the construction requirements and discuss them with the inspector to be accompanied.
- 2. Accompany a qualified inspector during implementation of the inspection procedures and witness the conduct of inspection activities in progress.
- 3. Meet with your supervisor or a qualified inspector designated by your supervisor and discuss the result of your activities.

**DOCUMENTATION:** 

Construction Inspection Proficiency Level Qualification Signature Card Item OJT-Cl-1

# **Construction Inspector On-the-Job Activity**

**TOPIC:** (OJT-Cl-2) Inspection Entrance/Exit Meetings

**PURPOSE:** The purpose of this activity is to familiarize an inspector with entrance

and exit meetings for a typical construction inspection.

**COMPETENCY** 

AREA: INSPECTION

**LEVEL** 

**OF EFFORT:** 8 hours

**REFERENCES**: 1. IP 30703, Entrance and Exit Meetings (previously deleted)

2. Applicable Inspection Procedures

EVALUATION CRITERIA:

Upon completion of the tasks, the inspector should be able to:

1. Demonstrate proficiency in conducting entrance and exit

meetings.

**TASKS:** 1. Under the supervision of a qualified construction inspector or

supervisor, conduct an entrance meeting.

2. Under the supervision of a qualified construction inspector or

supervisor, conduct an exit meeting.

**DOCUMENTATION**: Construction Inspection Proficiency Level Qualification

Signature Card Item OJT-CI-2

# **Construction Inspector On-the-Job Activity**

**TOPIC:** (OJT-Cl-3) Inspection Documentation

**PURPOSE:** The purpose of this activity is to familiarize an inspector with the

requirements for construction inspection report documentation.

**COMPETENCY** 

AREA: INSPECTION

**LEVEL** 

**OF EFFORT:** 24 hours

**REFERENCES**: 1. Manual Chapter 0613, Construction Inspection Reports

2. CIPIMS User's Guide

3. Applicable Inspection Procedures

4. NRC Enforcement Manual Section for Construction Inspection

EVALUATION CRITERIA:

Upon completion of the tasks, the inspector should be able to:

1. Generally describe the requirements for documenting construction inspection reports.

2. Demonstrate the ability to properly document inspection findings in accordance with the applicable NRC Enforcement Manual sections.

TASKS:

- 1. Review the references listed above to develop an understanding of the documentation requirements for construction inspections.
- 2. Under the guidance of a qualified construction inspector, draft an inspection report input and develop CIPIMS inputs.
- 3. Organize and incorporate draft inspection report inputs from other inspectors, as applicable.

**DOCUMENTATION**: Construction Inspection Proficiency Level Qualification

Signature Card Item OJT-CI-3

# Construction Inspector Technical Proficiency Level Signature Card and Certification

Inspector Name:	Inspector Initials/Date	Supervisor's Signature/Date				
Required Training Courses						
All Functional Areas						
New Reactor Technology (one design only is required)						
Construction Inspection Program						
Completing a Codes and Inspection Course in only one discip	oline is required:					
Civil/Structural Functional Area	_					
Codes and Inspection Course (Civil/Structural)						
Mechanical Functional Area						
Codes and Inspection Course (Mechanical)						
Electrical Functional Area						
Codes and Inspection Course (Electrical)						
Construction/Start-up Testing Functional Area						
Preoperational and Startup Testing Inspection						
Individual Study Activities						
(ISA-CI-1) Construction Quality Assurance (QA) and Defect Reporting Requirements						
(ISA-CI-2) 10 CFR Part 52						
(ISA-CI-3) Construction Inspection Program						
(ISA-CI-4) Design Control Document						
(ISA-CI-5) Construction Inspection Program Information Management System (CIPIMS)						
(ISA-CI-6) Industry Codes and Standards - General						
(ISA-ASME-1) Regulatory Basis for the Requirements Specified in the American Society of Mechanical Engineers (ASME) Boiler & Pressure Vessel Code - (Mechanical Area Inspectors Only)						
(ISA-ASME-2) American Society of Mechanical Engineers Boiler & Pressure Vessel Code, Section III (ASME Code, or Section III) Overview - (Mechanical Area Inspectors Only)						
On-the-Job Activities						
(OJT-CI-1) Inspection Accompaniment						
(OJT-CI-2) Inspection Entrance/Exit Meetings						

Inspector Name:	Inspector Initials/Date	Supervisor's Signature/Date			
(OJT-CI-3) Inspection Documentation					
Area for Certification:					
<ul> <li>□ Civil/Structural</li> <li>□ Mechanical</li> <li>□ Electrical</li> <li>□ Preoperational/StartupTesting</li> </ul>					
Branch Chief's signature indicates successful completion of all required courses and activities listed in this journal for designated inspection area and readiness to appear before the Oral Board.					
Branch Chief's Signature:	Date:				

# Form 1

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Construction Inspector Technical Proficiency Level Equivalency Justification				
Inspector Name:		Identify equivalent training and experience for which the inspector is to be given credit	s	
Required Training Courses				
Individual Study Activities				
On-the-Job Activities				
Branch Chief's Recommendation —	Signati	ure / Date		
Division Director's Approval	Signati	ure / Date		
Copies to: Inspector HR Office Branch Chief				

This form must accompany the Signature Card and Certification, if applicable.

# Revision History Page

Commitment Tracking Number	Issue Date	Description of Change	Training needed	Training Completion Date	Comment Resolution Accession Number
	02/06/07 CN 07-005	Initial issuance of document	NO		

C-37