

January 26, 2007

Mr. Thomas J. Palmisano  
Site Vice President  
Prairie Island Nuclear Generating Plant  
Nuclear Management Company, LLC  
1717 Wakonade Drive East  
Welch, MN 55089

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 & 2 - NRC  
RECEIPT OF RESPONSE TO GENERIC LETTER 2003-01 "CONTROL ROOM  
HABITABILITY" (TAC NOS. MB9843 AND MB9844)

Dear Mr. Palmisano:

The Nuclear Regulatory Commission (NRC) acknowledges the receipt of your response to Generic Letter (GL) 2003-01, "Control Room Habitability," dated December 09, 2003 (Accession No. ML033500298) and December 18, 2006 (Accession No. ML063520545). This letter provides a status of your response and describes any additional information that may be necessary to consider your response to GL 2003-01 complete.

The GL requested that you confirm that your control rooms meet their design bases (e.g. General Design Criterion (GDC) 1, 3, 4, 5, and 19, draft GDC, or principal design criteria), with special attention to: (1) determination of the most limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in your design bases for meeting control room operator dose limits from accidents (GL 2003-01, Item 1a); (2) determination that the most limiting unfiltered inleakage is incorporated into your hazardous chemical assessments (GL 2003-01, Item 1b); and (3) determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 2003-01, Item 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability, and plans to retire them (GL 2003-01, Item 2).

You reported the results of tracer gas tests you performed in 1998, and in 2004, for the Prairie Island Nuclear Generating Plant Units 1 and 2 common control room using American Society for Testing Materials Standard E741, "Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution." In the most recent tests (2004), you determined that the maximum tested values for inleakage into the control room envelope (CRE) was 151 cfm for High Radiation-Train A, and 135 cfm for Safety Injection-Train A. These measurements are less than the 165 cfm assumed in the loss-of-coolant accident analysis, the 175 cfm assumed in the main steamline break analysis, and the 410 cfm assumed in the fuel handling accident analysis. You indicated that in-leakage results are higher with Train A operating.

You provided information that adequately supported a conclusion that unfiltered inleakage into the CRE is not specifically incorporated into the hazardous chemical assessment because toxic

gases are not considered to be a threat based on hazard screening performed on chemicals stored onsite or transported nearby. You also provided information which demonstrated that reactor control capability is maintained from either the control room or the alternate shutdown panel in the event of smoke.

The GL further requested that you assess your Technical Specifications (TS) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design basis analysis for control room habitability, in light of the demonstrated inadequacy of a delta ( $\Delta$ ) P measurement to alone provide such verification (GL Item 1.c). As permitted by the GL, you provided a schedule for revising the surveillance requirement in the TS to reference an acceptable surveillance methodology. In your December 9, 2003, response, you indicated that you would submit proposed changes to your TS that will capture the intent of Technical Specification Task Force (TSTF)-448 six months following the approval of TSTF-448.

The information you provided also supported the fact that there are no compensatory measures needed to be in place to demonstrate control room habitability and that you are committed to meet the intent of the draft GDC regarding control room habitability.

Your commitment to submit a license amendment request based on TSTF-448, following our formal review and approval, is acceptable for purposes of closing out your response to GL 2003-01.

If you have any questions regarding this correspondence, please contact me.

Sincerely,

**/RA/**

Peter S. Tam, Senior Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

cc: See next page

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Peter S. Tam, Senior Project Manager  
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July 2006