



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET, SW, SUITE 23T85
ATLANTA, GEORGIA 30303-8931

January 5, 2007

Southern Nuclear Operating Company, Inc.
ATTN: Mr. T E. Tynan
Site Vice President
7821 River Road
Waynesboro, GA 30830

SUBJECT: NOTIFICATION OF VOGTLE NUCLEAR POWER PLANT- COMPONENT
DESIGN BASES INSPECTION - NRC INSPECTION REPORT
05000424/2007006, 05000425/2007006

Dear Mr. Tynan:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a component design bases inspection at your Vogtle Nuclear Power Plant during the weeks of April 23-27, 2007, May 7-11, 2007, and May 21-25, 2007. The inspection team will be led by Mike Cain, a Senior Reactor Inspector from the NRC's Region II Office. This inspection will be conducted in accordance with the baseline inspection procedure, Procedure 71111.21, Component Design Bases Inspection, issued June 22, 2006.

The inspection will evaluate the capability of risk significant / low margin components to function as designed and support proper system operation. The inspection will also include a review of selected operator actions, operating experience, and modifications.

During a telephone conversation on January 03, 2007, Mr. Cain confirmed with Mr. M. Sharma of your staff, arrangements for an information gathering site visit and the three-week onsite inspection. The schedule is as follows:

- Information gathering visit: Week of April 9-13, 2007.
- Onsite weeks: April 23-27, 2007, May 7-11, 2007, and May 21-25, 2007.

The purpose of the information gathering visit is to meet with members of your staff to identify risk-significant components and operator actions. Information and documentation needed to support the inspection will also be identified. Mr. Walt Rogers, a Region II Senior Reactor Analyst, may accompany Mr. Cain during the information gathering visit to review probabilistic risk assessment data and identify risk significant components which will be examined during the inspection.

The enclosure lists documents that will be needed prior to the information gathering visit. Please contact Mr. Cain prior to preparing copies of the materials listed in the enclosure. The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for the inspection preparation.

During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space, supplemental documents requested to be made available to the team in the Region II office prior to the inspection preparation week of April 16, 2007; arrangements for site access; and the availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Cain at (404) 562-4876 or me at (404) 562-4601.

Sincerely,

/RA/

Charles R. Ogle, Acting Deputy Director
Division of Reactor Safety

Docket Nos: 50-424, 50-425
License Nos: NPF-68, NPF-81

Enclosure: Information Request for Component Design Basis Inspection
cc w/encl: (See page 3)

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During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space, supplemental documents requested to be made available to the team in the Region II office prior to the inspection preparation week of April 16, 2007; arrangements for site access; and the availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Cain at (404) 562-4876 or me at (404) 562-4605.

Sincerely,

/RA/

Charles R. Ogle, Acting Deputy Director
Division of Reactor Safety

Docket Nos: 50-424, 50-425
License Nos: NPF-68, NPF-81

Enclosure: Information Request for Component Design Basis Inspection
cc w/encl: (See page 3)

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ADAMS: XYes ACCESSION NUMBER: _____

OFFICE	RII:DRP	RII:DRS					
SIGNATURE	/RA/	/RA/					
NAME	S. Shaeffer	L.Cain					
DATE	01/05/2007	01/05/2007	1/ /2007	1/ /2007	1/ /2007	1/ /2007	1/ /2007
E-MAIL COPY?	NO	YES	YES NO	YES NO	YES NO	YES NO	YES NO

INFORMATION REQUEST FOR VOGTLE NUCLEAR POWER PLANT COMPONENT DESIGN BASES INSPECTION

Please provide the information electronically in “.pdf” files, Excel, or other searchable format on CDROM. The CDROM should be indexed and hyperlinked to facilitate ease of use. Lists should contain enough information to be easily understood to someone who has a knowledge of pressurized water reactor technology.

1. From your most-recent probabilistic safety analysis (PSA) excluding external events and fires:
 - a. Two risk rankings of components from your site-specific probabilistic safety analysis (PSA): one sorted by Risk Achievement Worth (RAW), and the other sorted by Birnbaum Importance.
 - b. A list of the top 500 cutsets.
 - c. A risk ranking of all operator actions, sorted by RAW.
2. From your most-recent probabilistic safety analysis (PSA) including external events and fires:
 - a. Two risk rankings of components from your site-specific probabilistic safety analysis (PSA): one sorted by Risk Achievement Worth (RAW), and the other sorted by Birnbaum Importance.
 - b. A list of the top 500 cutsets.
 - c. A risk ranking of all operator actions, sorted by RAW.
3. Human reliability worksheets for the operator actions included in your PSA model.
4. Any pre-existing evaluation or list of components and calculations with low design margins (i.e., pumps closest to the design limit for flow or pressure, diesel generator close to design required output, heat exchangers close to rated design heat removal, MOV risk-margin rankings, etc.).
5. A list of operating experience evaluations completed within the last two years, sorted by associated component and/or system.
6. A list of design modifications implemented within the last two years, sorted by affected system.
7. A list of corrective-action evaluations that involved plant equipment and were completed within the last two years, sorted by involved component or system.
8. A list of common-cause failure of components that have occurred at Vogtle and have been identified within the last five years.

Enclosure

9. A list of operability evaluations completed within the last two years, sorted by associated component or system.
10. Contact information for a person to discuss PRA information prior to information gathering trip: name, title, phone number, and e-mail address.
11. List of equipment currently on the site's Station Equipment Reliability Issues List, including a description of the reason(s) why each component is on that list and summaries (if available) of your plans to address those reasons.
12. List of equipment currently in RIS-05-020 (Formerly GL 91-18) status.
13. List of equipment currently in MR (a)(1) status.