

January 9, 2007

Mr. Bruce H. Hamilton
Vice President, Oconee Site
Duke Power Company LLC
7800 Rochester Highway
Seneca, SC 29672

SUBJECT: NRC RECEIPT OF OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3
(OCONEE), RESPONSE TO GENERIC LETTER 2003-01, "CONTROL ROOM
HABITABILITY" (TAC NOS. MB9829, MB9830, AND MB9831)

Dear Mr. Hamilton:

The Nuclear Regulatory Commission (NRC) acknowledges the receipt of your response to Generic Letter 2003-01, "Control Room Habitability," dated December 9, 2003 (Agencywide Documents Access and Management System [ADAMS] Accession No. ML033510109). This letter provides a status of your response and describes any actions that may be necessary to consider your response to GL 2003-01 complete.

The GL requested that you confirm that your control rooms meet their design bases (e.g., General Design Criteria [GDC] 1, 3, 4, 5, and 19, draft GDC, or principal design criteria), with special attention to: (1) Determination of the most-limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in your design bases for meeting control room operator dose limits from accidents (GL 2003-01, Item 1a), (2) Determination that the most-limiting unfiltered inleakage is incorporated into your hazardous chemical assessments (GL 2003-01, Item 1b) and, (3) Determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 2003-01, Item 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability, and plans to retire them (GL 2003-01, Item 2).

You reported the results of ASTM E741 (American Society for Testing Materials, Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution) tracer gas tests for the Oconee control rooms. The control room for Units 1 and 2 are common while the control room for Unit 3 is separate. All control rooms are pressurized for accident mitigation.

For Units 1 and 2 you determined that the maximum tested value for unfiltered inleakage into the control room envelope (CRE) was 869 (+/- 31) actual cubic feet per minute (acfm) for normal alignment (pre-booster pump actuation) and 0 (+/- 30) standard cubic feet per minute (scfm) for emergency alignment (post-booster fan actuation). These values are less than the unfiltered inleakage values of 1150 cfm (pre-booster pump actuation) and 40 cfm (post-booster pump actuation) assumed in your design basis radiological dose analyses for control room habitability (CRH).

For Unit 3 you determined that the maximum tested value for unfiltered inleakage into the CRE was 467 (+/- 16) acfm for normal alignment (pre-booster pump actuation) and 0 (+/- 42) scfm for emergency alignment (post-booster pump actuation). These values are less than the unfiltered inleakage values of 1150 cfm (pre-booster pump actuation) and 40 cfm (post-booster pump actuation) assumed in your design basis radiological dose analyses for CRH.

You indicated that your hazardous chemical assessment accounts for the most-limiting unfiltered inleakage into the Oconee CRE, which occurs during normal operation with the booster fans OFF. This value is 1065 cfm for the Units 1 and 2 control rooms and 534 cfm for the Unit 3 control room from the 1998 tracer gas test. Since your 2001 tracer gas test results are lower (869 +/- 31 and 467 +/- 16, respectively) the information you provided adequately supported the conclusion that the most-limiting unfiltered inleakage into the CRE is incorporated into the hazardous chemical assessments. You also indicated that reactor control capability is maintained from either the control room or the Standby Shutdown Facility in the event of smoke.

The GL further requested that you assess your Technical Specifications (TSs) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design-basis analysis for CRH, and in light of the demonstrated inadequacy of a delta (Δ) P measurement to alone provide such verification (GL 2003-01, Item 1.c). As permitted by the GL, you provided a schedule for revising the surveillance requirement in the TSs to reference an acceptable surveillance methodology. In your December 9, 2003, response, you stated your plans to implement Technical Specification Task Force (TSTF)-448 once it is made available for use.

The information you provided also supported the fact that there are no compensatory measures needed to be in place to demonstrate CRH.

B. Hamilton

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The information you provided also supported the conclusion that you are committed to meet the intent of the draft GDC regarding CRH which is documented in your Updated Final Safety Evaluation Report (UFSAR).

Based on the information provided and your plans to implement TSTF-448, the NRC staff finds your responses to GL 2003-01 complete.

If you have any questions regarding this correspondence, please contact me.

Sincerely,

/RA/

Leonard N. Olshan, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270 and 50-287

cc: See next page

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Oconee Nuclear Station, Units 1, 2, and 3

cc:

Mr. Bruce H. Hamilton
Vice President, Oconee Site
Duke Power Company LLC
7800 Rochester Highway
Seneca, SC 29672

Ms. Lisa F. Vaughn
Associate General Counsel and Managing
Attorney
Duke Energy Carolinas, LLC
526 South Church Street - EC07H
Charlotte, North Carolina 28202

Manager, LIS
NUS Corporation
2650 McCormick Dr., 3rd Floor
Clearwater, FL 34619-1035

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
7812B Rochester Highway
Seneca, SC 29672

Mr. Henry Porter, Director
Division of Radioactive Waste Management
Bureau of Land and Waste Management
Dept. of Health and Env. Control
2600 Bull St.
Columbia, SC 29201-1708

Mr. Michael A. Schoppman
Framatome ANP
1911 North Ft. Myer Dr.
Suite 705
Rosslyn, VA 22209

Mr. B. G. Davenport
Regulatory Compliance Manager
Oconee Nuclear Site
Duke Energy Corporation
ON03RC
7800 Rochester Highway
Seneca, SC 29672

Mr. Leonard G. Green
Assistant Attorney General
NC Department of Justice
P.O. Box 629
Raleigh, NC 27602

Mr. R. L. Gill, Jr.
Manager - Nuclear Regulatory
Issues and Industry Affairs
Duke Power Company LLC
526 S. Church St.
Mail Stop EC05P
Charlotte, NC 28202

Division of Radiation Protection
NC Dept of Environment, Health, & Natural
Resources
3825 Barrett Dr.
Raleigh, NC 27609-7721

Mr. Peter R. Harden, IV
VP-Customer Relations and Sales
Westinghouse Electric Company
6000 Fairview Road
12th Floor
Charlotte, NC 28210

Mr. Henry Barron
Group Vice President, Nuclear Generation
and Chief Nuclear Officer
P.O. Box 1006-EC07H
Charlotte, NC 28201-1006

Mr. Charles Brinkman
Director, Washington Operations
Westinghouse Electric Company
12300 Twinbrook Parkway, Suite 330
Rockville, MD 20852

Ms. Kathryn B. Nolan
Senior Counsel
Duke Energy Carolinas, LLC
526 South Church Street - EC07H
Charlotte, NC 28202