10 CFR 50.59, 10 CFR 72.48

Palo Verde Nuclear Generating Station

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102-05618-SAB/TNW/CJJ
December 22, 2006

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Dear Sirs:

Subject: Palo Verde Nuclear Generating Station (PVNGS)
Units 1, 2, & 3
Docket Nos. STN 50-528/529/530
Annual 10 CFR 50.59, 10 CFR 72.48 and Commitment Change Report
(January – December 2005)

Pursuant to 10 CFR 50.59(d)(2), Arizona Public Service Company is submitting the enclosed report. This report contains a brief description of each change and the conclusion of the evaluation required by 10 CFR 50.59(d)(1) for each change. This report contains all evaluations written during 2005, regardless of the implementation status of the evaluated action.

There were no changes per 10 CFR 72.48(d)(1) required to be reported during 2005.

There was one NRC Commitment clarification made during 2005 which is also described in the enclosure.

No commitments are being made to the NRC by this letter. Should you have any questions, please contact Thomas N. Weber at (623) 393-5764.

Sincerely,

SAB/TNW/CJJ/gt

Enclosure

cc: B. S. Mallett (all w/enclosure)
M. B. Fields
G. G. Warnick

IE47

ENCLOSURE

PALO VERDE NUCLEAR GENERATING STATION

ACRONYM/ABBREVIATION LIST,

10 CFR 50.59 REPORT JANUARY – DECEMBER 2005,

AND

COMMITMENT CLARIFICATION - 2005

ACRONYMN/ABBREVIATION LIST

ALARA	-	As Low As Reasonably Achievable
ASME	-	American Society of Mechanical Engineers
Calc.	-	Calculation
COLSS	-	Core Operating Limit Supervisory System
CPC	-	Core Protection Calculator
CRDR	-	Condition Report/Disposition Request
CVCS HUT	-	Chemical Volume and Control System Hold Up Tank
DMWO	-	Design Modification Work Order
EDC	-	Engineering Document Change
EOP	-	Emergency Operating Procedure
IA	-	Instrument Air
IOSGADV	-	Inadvertent Opening of a Steam Generator Atmospheric Dump Valve
LDCR	-	Licensing Document Change Request
LOCA	-	Loss of Coolant Accident
LOF	-	Loss of Flow
MS	-	Main Steam
NC	-	Nuclear Cooling
PVNGS	-	Palo Verde Nuclear Generating Station
RAS	-	Recirculation Actuation Signal
RCS	-	Reactor Coolant System
RMI	-	Reflective Metal Insulation
RSG	-	Replacement Steam Generator
RTD	-	Resistance Temperature Detector
SAFDL	-	Specified Acceptable Fuel Design Limit
SDC	-	Shutdown Cooling
SG	-	Steam Generator
TMOD	-	Temporary Modification Work Order
TS	-	Technical Specifications
UFSAR	-	Updated Final Safety Analysis Report

10 CFR 50.59 Annual Report (January - December 2005)

Log	Doc Type	Doc Number	Description	Summary
E-05-0001	Procedure	40OP-9SI02.R49	This procedure revision altered the Safety Injection pump stand-by line-up so that the lines between the containment sump and the Safety Injection pump suction, as well as the Containment RAS sumps themselves, remain filled with borated water during normal power operation.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0002	LDCR	04-F005 & 04-F019	This evaluation addressed the application of a new approved evaluation method to PVNGS Units 1 & 3 to provide a consistent evaluation method for all PVNGS units. The methodology separated the IOSGADV event from LOF from SAFDL event, thus established a more appropriate evaluation of IOSGADV event while creating a new safety analysis for a "composite" limiting infrequent event. The method was approved for PVNGS Unit 2 by TS Amendment #149.	This change has been previously approved by the NRC TS Amendment #149.
E-05-0003	LDCR	05-F008	This modification implements the changes from DMWO 2739742 (Unit 3 only) into Units 1 and 2. The modification installed piping, isolation valves and supports to facilitate the filling, draining, venting, and testing of the post-LOCA recirculation piping located upstream of the recirculation check valves, PSIAV205 and PSIAV206. The modification also upgraded various materials on valves JSIAUV0673 and JSIBUV0675 to more corrosion resistant materials.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0004			Canceled	
E-05-0005	EDC	2005-00211	This EDC updated vendor calculation A-PV2-FE-0164 (PVNGS Subcritical Main Steam Line Break Return to Power for RSG/Uprate Scenario) to take into account errors documented in CRDR 2595572. Specifically, the initial power level and steam generator level used in the vendor calculation were not the most limiting, however crediting a higher RWT boron concentration resulted in impact to the conclusion.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0006			Canceled	
E-05-0007	DFWO	2745267	This modification applied an approved epoxy coating on inside and outside concrete surfaces of D ring and Reactor Cavity walls replaced during Steam Generator outage 2R11.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).

Log	Doc Type	Doc Number	Description	Summary
E-05-0008	DMWO	2541291	This modification covers activities performed during U1R12 [U3R13], which will facilitate steam generator replacement. These include modifications to existing platforms, addition of new platforms, platform beam connection upgrade, removal of polar crane access walkway support brackets, concrete wall cutting/restoration and the addition of a new permanent steel formwork plate.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0009	DMWO	2513158	The insulation of the reactor vessel upper head was replaced with a new design at Unit 1 under DMWO 2251358. Testing determined the insulation did not meet Regulatory Guide 1.36. This evaluation was performed to justify the potentially non-conforming insulation remaining installed for the remainder of the operating cycle.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0010	PROCEDURE	72ST-9RX03.R11	This revision to appendix B of the procedure revised the azimuthal tilt calculation with COLSS out of service.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0011	DFWO	2788561	This DFWO is a Use-As-Is disposition for missing parts potentially left in the refueling pool. The proposed activity evaluated the potential impact of 5 fastening devices found missing from the refueling machine hoisting frame. These devices consist of two 1/4" bolts, one 3/8" bolt, one 3/8" nut, and one 3/8" washer.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0012	DMWO	2541285	This modification was to the large bore piping systems and associated supports that are necessary to accommodate steam generator replacements in Units 1 and 3 during U1R12 and U3R13, respectively. A new large bore secondary piping system, recirculation, and associated valves will also be installed to facilitate use of the new recirculation feature of the replacement steam generator (RSG) design. Modifications to the Main Steam (MS) whip restraints will be made to accommodate the taller RSGs. In addition, modifications will be made due to increased loads resulting from steam generator replacement and power uprate.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0013	PROCEDURE	73TI-9CH04	This procedure will obtain plant data during specified pump suction flow path manipulations in order to determine plant configurations that cannot support the CH-536 charging pump suction path to be an operable boron path.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0014	DFWO	2788561	This DFWO is a Use-As-Is disposition for missing finger from tele-detector. The debris is ~ 2" long by ~ 1/4" wide the ~ 1 mm thick and made of a copper alloy. The debris could have fallen on top of fuel assemblies in the reactor vessel.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).

Log	Doc Type	Doc Number	Description	Summary
E-05-0015			Canceled	
E-05-0016	LDCR	05-F023	This revision to the UFSAR modified Table 9.3-3 to state that CVCS HUT mode of sampling is now Local Sampling at 2 locations equivalent to existing (old) method listed on sheet 10 of 11.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0017	DMWO	2541022	This DMWO addresses process/performance design requirements and regulatory requirements associated with only the design and fabrication of the Units 1 and 3 steam generators, their internals, supports, and cold leg elbows. Sufficient design information is included to assure that design intent, scope and approval are adequate and are in compliance with applicable design requirements and regulatory criteria.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0018			Canceled	
E-05-0019			Canceled	
E-05-0020	DMWO	2540917	This is the Master DMWO for Power Uprate. This DMWO addresses all engineering documentation and products necessary to support the process/performance, safety analyses, radiological analyses and topical issues for design bases for the power uprate.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0021	DFWO	2809648	This DFWO was an Accept-As-Is disposition of a short-term excursion above the Design Temperature, as described in the UFSAR, of 700 F for the pressurizer. It has been determined that all pressure boundary aspects of the pressurizer meet the ASME Section III code stress limits, with the use of stress intensity values as permitted in Relief Request 33.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0022	TMOD	2785294	This TMOD provided temporary instrument air (IA) compressors during the replacement of the permanent compressors under DMWO 2693912 for Unit 3.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0023			Canceled	
E-05-0024	DMWO	2358951	This modification permanently removed the Gas Stripper radiation monitor and its support equipment from service. This modification resulted in a change to the UFSAR (LDCR 05-F038). The UFSAR was updated to remove the reference to Radiation Monitor DJCHNRYSH0265.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0025			Canceled	

Log	Doc Type	Doc Number	Description	Summary
E-05-0026			Canceled	
E-05-0027	DMWO	2651111	This modification and associated UFSAR revision (LDCR 04-F031) will remove NC low flow switch JNCNFSL0613 and will relocate the associated isolation interlock with CH-UV523 to a new temperature switch JCHNTS0224-2.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0029	TMWO	2826217	This modification was for the temporary installation and use of a portable temporary laundry skid interface to PVNGS Unit 1.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0030			Canceled	
E-05-0031	ABB	V-99-060	This activity changed the fuel tube cladding dimensional tolerances and tolerances of form beginning with Palo Verde Batch 3L fuel.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0032	CALC	FS-03-C00-1996-007	This evaluation revised a Proprietary Unit 3 Nuclear Fuel Basis document. There were alignment issues between the fuel center guide tube and the lower end fitting.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0033	DMWO	2754516	This modification installed a welded attachment in the Unit 1 SDC suction line nozzle to suppress vortex generated acoustic vibration.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0035	ABB	V-99-017	This evaluation was for ABB V-99-017, Transmittal of Final PV2L Manufacturing Documentation, Enclosure D. Two batch PV2L Urainum Erbium oxide fuel pellet lots, C976JK Sublot A and C977JK Sublot B, contained pellets with high sintered density. The estimated upper confidence limits of these lots exceeded the high sintered density specification limit of 97% Theoretical Density (TD) by a small amount. This excess was 0.2% TD and was accommodated via accepted DCR 5321-98-2082-H-1.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0036	Procedure	40AO-9ZZ01	This procedure revision added a New Standard Appendix (103) for RCS Makeup and Emergency Boration for use with the EOP's based on Engineering Study for use of CHE-HV-536. This evaluation also applies to various other procedure changes also based on the same Engineering Study.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).
E-05-0037	TMOD	2849895	This temporary modification will substitute non-class RTD 2JRCNTE0111X for Class 1E RDT 2JRCDTE0112HD as the Reactor Coolant System (RCS) Loop-1 Hot-Leg temperature input to the Unit 2 Core Protection Calculator (CPC) Channel-D.	This change does not require prior NRC approval in accordance with 10CFR50.59(c)(1).

Commitment Clarification – 2005

The below commitment clarification was made on November 18, 2005:

In response to GL 89-13 letter 102-02678 dated October 1, 1993 had the following statement:

> ". . . The SP system is also monitored for total system flow and heat exchanger pressure drop on a monthly basis."

The intent of the pressure drop monitoring was only for the EW heat exchanger and not the other heat exchangers cooled by the SP system. The justification for this clarification is that at the time the letter was issued the EW heat exchanger was the only heat exchanger being monitored for pressure drop. Since the letter clearly indicates that monitoring was already in place and since only the EW heat exchanger was being monitored at that time this clarification is justified.