



DEC 26 2006

L-PI-06-092  
10 CFR 50.90  
10 CFR 50.46

U S Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555-0001

Prairie Island Nuclear Generating Plant Units 1 and 2  
Dockets 50-282 and 50-306  
License Nos. DPR-42 and DPR-60

Supplement to License Amendment Request (LAR) to Incorporate Large Break Loss Of Coolant Accident (LOCA) Analyses Using the Automated Statistical Treatment of Uncertainty Method (TAC Nos. MD2567 and MD2568)

- References: 1) License Amendment Request (LAR) to Incorporate Large Break Loss Of Coolant Accident (LOCA) Analyses Using the Automated Statistical Treatment of Uncertainty Method, dated July 6, 2006, Accession Number ML061880026.  
2) Supplement to License Amendment Request (LAR) to Incorporate Large Break Loss Of Coolant Accident (LOCA) Analyses Using the Automated Statistical Treatment of Uncertainty Method, dated September 15, 2006, Accession Number ML062610088.

The Nuclear Management Company (NMC) submitted an LAR, Reference 1, to incorporate new Large Break LOCA (LBLOCA) analyses using the realistic LBLOCA methodology in the NRC approved WCAP-16009-P-A, "Realistic Large Break LOCA Evaluation Methodology Using the Automated Statistical Treatment of Uncertainty Method (ASTRUM)" and also to revise Technical Specification (TS) 5.6.5.b to include reference to WCAP-16009-P-A. Reference 2 provided supplemental information to this LAR which responded to NRC requests for additional information (RAI). This letter supplements the LAR to clarify the response to NRC RAI 3 in Reference 2 and use of the term "optimized fuel assembly (OFA)" fuel in that response. NMC submits this supplement in accordance with the provisions of 10 CFR 50.90.

In response to NRC RAI 3 of Reference 2, NMC referred to the Prairie Island Nuclear Generating Plant (PINGP) fuel type that was analyzed for the realistic LBLOCA as "PINGP Optimized Fuel Assembly (OFA) fuel". Based on subsequent discussions with the NRC and Westinghouse, this characterization of the fuel type requires clarification. The fuel analyzed for the realistic LBLOCA, and the fuel that is presently in use in both PINGP units, should be referred to as 14x14, 0.400" Outside Rod Diameter Vantage+ fuel.

Westinghouse uses the terminology "Vantage+" for all Zirlo clad pressurized water reactor fuel. The NRC approved Zirlo cladding for PINGP in License Amendments 108 (Unit 1) and 101 (Unit 2), dated September 3, 1993.

The supplemental information provided in this letter and enclosure does not impact the conclusions of the Determination of No Significant Hazards Consideration and Environmental Assessment presented in the July 6, 2006 submittal as supplemented September 15, 2006.

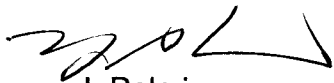
In accordance with 10 CFR 50.91, NMC is notifying the State of Minnesota of this LAR by transmitting a copy of this letter to the designated State Official.

Summary of Commitments

This letter contains no new commitments and no revisions to existing commitments.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on           DEC 26 2006



Thomas J. Palmisano  
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Nuclear Management Company, LLC

cc: Administrator, Region III, USNRC  
Project Manager, Prairie Island, USNRC  
Resident Inspector, Prairie Island, USNRC  
State of Minnesota