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December 6, 2006

U.S. Nuclear Regulatory Commission
Washington, DC 20555

ATTENTION: Document Control Desk

SUBJECT: **R.E. Ginna Nuclear Power Plant**
Docket No. 50-244

LER 2006-007, Main Steam Safety Valve Setpoint Exceedance

The attached Licensee Event Report (LER) 2006-007 is submitted in accordance with 10 CFR 50.73, Licensee Event Report System. A supplement to this LER is planned to be submitted by March 2, 2007. Should you have questions regarding the information in this submittal, please contact Mr. Robert Randall at (585) 771-3734 or robert.randall@constellation.com.

Very truly yours,

A handwritten signature in black ink, appearing to read "M.G.K." followed by "for M&K". The signature is written in a cursive, somewhat stylized script.

Mary G. Korsnick

Attachments: (1) LER 2006-007
(2) Regulatory Commitments

cc: S. J. Collins, NRC
P.D. Milano, NRC
Resident Inspector, NRC (Ginna)

1001678

IE22

Attachment 1

LER 2006-007

Main Steam Safety Valve Setpoint Exceedance

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME R.E. Ginna Nuclear Power Plant	2. DOCKET NUMBER 05000 244	3. PAGE 1 OF 4
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4. TITLE
Main Steam Safety Valve Setpoint Exceedance

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
10	07	2006	2006	- 007 -	00	12	06	2006	FACILITY NAME	DOCKET NUMBER
										05000
										05000

9. OPERATING MODE 1	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR§: (Check all that apply)									
10. POWER LEVEL 90	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)						
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)						
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)						
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)						
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)						
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)						
<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)							
<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER							
<input type="checkbox"/> 20.2203(a)(2)(vi)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A							

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME Robert Randall, Director of Licensing	TELEPHONE NUMBER (Include Area Code) (585) 771-3734
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED <input checked="" type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE) <input type="checkbox"/> NO	15. EXPECTED SUBMISSION DATE	MONTH	DAY	YEAR
		03	02	2007

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On October 7, 2006, with the plant in Mode 1, in-place testing of main steam safety valve 3508 determined that the as-found lift pressure did not meet the acceptance band of +1% / - 3% of setpoint (1085 psig), as specified by Technical Specification surveillance SR 3.7.1.1. This was the second unsatisfactory as-found lift pressure for a main steam safety valve, as in-place sequential testing had previously determined that safety valve 3515 had failed to meet the as-found acceptance band.

Technical Specification LC0 3.7.1, "Main Steam Safety Valves (MSSVs)", requires eight main steam safety valves to be operable in Modes 1, 2 and 3. Since the two unsatisfactory as-found lift pressures may have arisen over a period of time (found during sequential testing), it is assumed that at least one required main steam safety valve was not operable during past plant operation for a time greater than allowed. Therefore, this occurrence is considered reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by the plant's Technical Specifications.

The root cause evaluation of this event has not been completed; pending further evaluation of the specific test conditions. A supplement to this LER is planned.

Operation of the facility with the main steam safety valves as-found settings was within analytical bounds; therefore, this event had no impact on the health and safety of the public.

LICENSEE EVENT REPORT (LER)

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
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		2006	007	00	

17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

I. PRE-EVENT PLANT CONDITIONS:

At the time the condition was identified, the plant was in Mode 1 at approximately 90% rated thermal power.

II. DESCRIPTION OF EVENT:

A. EVENT:

On October 7, 2006, with the plant in Mode 1, in-place testing of main steam safety valve 3508 determined that the as-found lift pressure did not meet the acceptance band of +1% / - 3% of setpoint, as specified by Technical Specification surveillance SR 3.7.1.1. The initial as-found lift pressure for safety valve 3508 was 1.39% above the specified lift setting. This was the second unsatisfactory as-found lift pressure for a main steam safety valve, as in-place testing had previously determined that safety valve 3515 had failed to meet the as-found acceptance band, with an initial as-found lift pressure of 1.09% above the specified lift setting. All of the other valves were tested within range, but at an elevated lift setpoint when compared to their previous as-left value.

Technical Specification LC0 3.7.1 requires eight main steam safety valves to be operable in Modes 1, 2 and 3. Testing of main steam safety valves is performed one valve at a time, with each valve adjusted if necessary and returned to operable status before proceeding with the testing of another valve. In this manner, a maximum of one valve is known to be inoperable at any time during testing. However, since the cause of the two unsatisfactory as-found lift pressures may have arisen over a period of time, it is assumed that at least one required main steam safety valve was not operable during past plant operation for a time greater than allowed. Therefore, this occurrence is considered reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by the plant's Technical Specifications.

B. INOPERABLE STRUCTURES, COMPONENTS, OR SYSTEMS THAT CONTRIBUTED TO THE EVENT:

None

C. DATES AND APPROXIMATE TIMES OF MAJOR OCCURENCES:

- October 7, 2006, 1240 EDST: main steam safety valve 3515 removed from service for lift setpoint testing and returned to service following adjustment.
- October 7, 2006, 1445 EDST: main steam safety valve 3508 removed from service for lift setpoint testing and returned to service following adjustment.

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17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

D. OTHER SYSTEMS OR SECONDARY FUNCTIONS AFFECTED:

None

E. METHOD OF DISCOVERY:

Review of test data associated with as-found setpoint testing.

F. SAFETY SYSTEM RESPONSES:

No safety systems were actuated.

III. CAUSE OF EVENT:

The root cause evaluation of this event has not been completed; pending further evaluation by the vendor of the specific test conditions and equipment used during the testing. This evaluation is expected to include testing at the vendor facility. A supplement to this LER is planned.

IV. ASSESSMENT OF THE SAFETY CONSEQUENCES OF THE EVENT:

This event is reportable in accordance with 10 CFR 50.73, Licensee Event Report System, item (a)(2)(i)(B), which requires a report of, "Any operation or condition which was prohibited by the plant's Technical Specifications."

The operability of the main steam safety valves ensures that the secondary system pressure will be limited to within 110% of its design pressure of 1085 psig during the most severe anticipated system operational transient. The as-found condition of the main steam safety valves was compared to the current overpressure analysis prepared in support of extended power uprate and it was concluded that the analysis remained bounding. This analysis is conservative with regards to prior operation. As such, the applicable acceptance criteria for design basis events would have been met and the safety valves remained capable of performing their intended safety function.

Operation of the facility with the main steam safety valves as-found settings was within analytical bounds; therefore, this event had no impact on the health and safety of the public.

LICENSEE EVENT REPORT (LER)

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17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

V. CORRECTIVE ACTIONS:

A. ACTION TAKEN TO RETURN AFFECTED SYSTEMS TO PRE-EVENT NORMAL STATUS:

Immediate actions included adjusting main steam safety valves 3513 and 3508 to within +/-1% of their required set lift pressure.

The main steam safety valves were subsequently tested on October 31, 2006 during the plant startup from the refueling outage, and all of the valves exhibited a reduced as-found setpoint (within the +1% / - 3% allowed range).

B. ACTION TAKEN OR PLANNED TO PREVENT RECURRENCE:

The Long term corrective actions and a final cause determination will be documented in a supplemental LER for both of the subject main steam safety valves.

VI. ADDITIONAL INFORMATION:

A. FAILED COMPONENTS:

No other structures, systems, or components failed as result of this event.

B. PREVIOUS LERs ON SIMILAR EVENTS:

A similar Ginna LER event historical search was conducted which resulted in no similar events.

C. THE ENERGY INDUSTRY IDENTIFICATION SYSTEM (EII) COMPONENT FUNCTION IDENTIFIER AND SYSTEM NAME OF EACH COMPONENT OR SYSTEM REFERRED TO IN THIS LER:

COMPONENT	IEEE 803 FUNCTION NUMBER	IEEE 805 SYSTEM IDENTIFICATION
Valve, Relief	RV	SB

Attachment 2

Regulatory Commitments

REGULATORY COMMITMENT	DUE DATE
Submit supplemental LER.	March 2, 2007.