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U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Subject: Duke Power Company LLC
Oconee Nuclear Station, Unit 1, 2, and 3
Docket No. 50-269,-270,-387
Fourth Ten-year Inservice Inspection Interval
Request for Alternate (Relief Request 2004-ON-001)

By letter dated September 22, 2004, Duke Energy Corporation), now Duke Power Company LLC d/b/a Duke Energy Carolinas, LLC, (Duke) submitted Relief Request 2004-ON-001, a Request for Alternate per 10 CFR 50.55a(a)(3)(i) for the Oconee fourth 10-year Inservice Inspection Intervals which began on January 1, 2004 for Unit 1, September 9, 2004 for Unit 2, and December 16, 2004 for Unit 3.

As described in that request, the applicable code for this interval is ASME Boiler and Pressure Vessel Code, Section XI, 1998 Edition with 2000 Addenda. This Code Edition and Addenda lists two pressure test requirements for Class 3 systems.

Duke specifically sought relief to use leakage tests in lieu of hydrostatic tests on Class 3 pressure retaining components.

Duke understands that NRC review of Relief Request 2004-ON-001 is not complete. At this time, Duke wishes to withdraw Relief Request 2004-ON-001.

Instead, pursuant to 10 CFR 50.55a(g)(4)(iv) and in accordance with the guidance provided in NRC Regulatory Issue Summary (RIS) 2004-12, dated July 28, 2004, Duke is requesting NRC approval to use a portion of a later version of the ASME Boiler and Pressure Vessel Code. Specifically Duke wishes to use the ASME Boiler and Pressure Vessel Code, Section XI, 2001 Edition, Table IWD-2500-1, Category D-B, where there is only one periodic leakage test required. Duke plans to comply with the pressure and temperature hold times cited in 10 CFR 50.55a(b)(2)(xx) which requires a ten-minute holding time after attaining test pressure for Class 2 and Class 3 components that do not normally operate during operation, and no holding time is required for the remaining Class 2 and Class 3 components provided that the system has been in operation for at least four hours for insulated components or ten minutes for uninsulated components.

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This request is similar to a request submitted by Duke's Catawba Nuclear Site dated January 31, 2006.

As noted in RIS 2004-12, licensees seeking to use later editions and addenda of the ASME Code, Section XI, pursuant to 10 CFR 50.55a(f)(4)(iv) or 10 CFR 50.55a(g)(4)(iv), are required to obtain NRC approval prior to implementation, but are not required to seek relief pursuant to 10 CFR 50.55a(a)(3), 10 CFR 50.55a(f)(5)(iv), or 10 CFR 50.55a(g)(5)(iv).

There are no NRC commitments contained in this letter.

Please direct any questions to R. P. Todd at (864) 885-3418.

Very truly yours,

R. M. Glover / for

Bruce Hamilton,
Site Vice President,
Oconee Nuclear Station