

January 30, 2007

Mr. J. A. Stall  
Senior Vice President, Nuclear and  
Chief Nuclear Officer  
Florida Power and Light Company  
P.O. Box 14000  
Juno Beach, Florida 33408-0420

SUBJECT: ST. LUCIE UNIT 1 - ISSUANCE OF AMENDMENT REGARDING STEAM  
GENERATOR TUBE INTEGRITY (TAC NO. MD1382)

Dear Mr. Stall:

The Commission has issued the enclosed Amendment No. 200 to Renewed Facility Operating License No. DPR-67 for the St. Lucie Plant, Unit No. 1. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated April 24, 2006, as supplemented by letter dated September 14, 2006.

This amendment revises the steam generator surveillance program to be consistent with TS Task Force change traveler TSTF-449, "Steam Generator Tube Integrity."

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

**/RA/**

Brenda L. Mozafari, Senior Project Manager  
Plant Licensing Branch II-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-335

Enclosures:

1. Amendment No. 200 to DPR-67
2. Safety Evaluation

cc w/enclosures: See next page

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NRR-058

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FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 200  
Renewed License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power & Light Company (the licensee), dated April 24, 2006, as supplemented by letter dated September 14, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Renewed Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 3.B to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 200, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

Stewart N. Bailey, Acting Chief  
Plant Licensing Branch II-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Operating License  
and Technical Specifications

Date of Issuance: January 30, 2007

ATTACHMENT TO LICENSE AMENDMENT NO. 200  
TO RENEWED FACILITY OPERATING LICENSE NO. DPR-67  
DOCKET NO. 50-335

Replace Page 3 of Renewed Operating License DPR-67 with the attached Page 3.

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages

Index Page V  
Index Page XV  
1-4  
1-5  
3/4 4-5  
3/4 4-6  
3/4 4-7  
3/4 4-8  
3/4 4-10  
3/4 4-11  
3/4 4-12  
3/4 4-14  
3/4 4-14a  
6-15d  
- - - -  
- - - -  
6-19c

Insert Pages

Index Page V  
Index Page XV  
1-4  
1-5  
3/4 4-5  
3/4 4-6  
3/4 4-7  
3/4 4-8  
3/4 4-10  
3/4 4-11  
3/4 4-12  
3/4 4-14  
3/4 4-14a  
6-15d  
6-15e  
6-15f  
6-19c

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 200

TO RENEWED FACILITY OPERATING LICENSE NO. DPR-67

FLORIDA POWER AND LIGHT COMPANY

ST. LUCIE PLANT, UNIT NO. 1

DOCKET NO. 50-335

## 1.0 INTRODUCTION

By application dated April 24, 2006 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML061170619), as supplemented by letter dated September 14, 2006 (ML062620138), Florida Power and Light Company (FPL, the licensee) requested changes to the Technical Specifications (TS) for the St. Lucie Nuclear Plant, Unit No. 1.

The proposed changes would revise the existing steam generator (SG) tube surveillance program. The changes are modeled after TS Task Force (TSTF) change traveler TSTF-449, Revision 4, "Steam Generator Tube Integrity," and the model safety evaluation (SE) prepared by the U.S. Nuclear Regulatory Commission (NRC) and published in the *Federal Register* on March 2, 2005 (70 FR 10298). In this regard, the scope of the application includes changes to the definition of leakage, the primary-to-secondary leakage requirements, the SG tube surveillance program (SG tube integrity), the SG reporting requirements, and associated changes to the TS Bases.

The supplement dated September 14, 2006, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination (NSHCD), as published in the *Federal Register* on July 18, 2006 (71 FR 40746).

## 2.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The licensee provides TSs in order to maintain the operational capability of structures, systems and components that are required to protect the health and safety of the public. The Commission's regulatory requirements related to the content of the TSs are contained in Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36. Pursuant to 10 CFR 50.90, licensees may request changes to their TSs.

The background, description, and applicability of the proposed changes associated with the SG tube integrity issue and the applicable regulatory requirements were included in the NRC staff's model SE published in the *Federal Register* on March 2, 2005. The "Notice of Availability of Model Application Concerning Technical Specification Improvement to Modify Requirements Regarding Steam Generator Tube Integrity Using the Consolidated Line Item Improvement Process [CLIIP]," was published in the *Federal Register* on May 6, 2005 (70 FR 24126), and made the model SE available for licensees to reference. This was subsequently modified by a simplified model SE (ML061040351), which was approved on September 8, 2006.

### 3.0 TECHNICAL EVALUATION

#### 3.1 Overview

In its April 24, 2006, application, and September 14, 2006, supplement, the licensee proposed changes to the St. Lucie Unit 1 TSs that are modeled after TSTF-449. Although the licensee's justification is based on the model SE published with the CLIIP, there were minor differences between TSTF-449 and the licensee's application. These included differences in the facility licensing basis (from that discussed in TSTF-449, which assumes the Standard TS basis) and differences in TS numbering and format, because the licensee has a different TS format than that assumed in TSTF-449.

With respect to the differences in the facility licensing basis, the differences did not invalidate the technical evaluation of TSTF-449; rather, they resulted in the licensee having to slightly deviate from some of the modifications discussed in TSTF-449. For example, in the Bases section for SG Tube Integrity, TSTF-449 indicated that the accident analysis for an SG tube rupture assumes the contaminated secondary fluid is only briefly released to the atmosphere via safety valves and the majority is discharged to the main condenser. Since the licensee has a different licensing basis than the one described in the standard TSs, the TSTF-449 wording was modified to reflect its existing licensing basis. Since these differences are minor in nature, are consistent with the plant's licensing basis (e.g., in the level of detail incorporated into the TS Bases), and are consistent with the intent of TSTF-449, the NRC staff determined they were acceptable.

Similarly, since the differences in numbering of the TSs are administrative in nature and do not affect the technical adequacy of the submittal, the NRC staff determined they were acceptable.

Due to the differences in the format of the TSs, the licensee had to reword several of the TSTF-449 requirements. Since these differences are administrative in nature and do not affect the technical adequacy of the submittal, the NRC staff determined they were acceptable.

The licensee also indicated they would be making changes to TS Bases pertaining to reactor coolant system operational leakage. These changes facilitated adopting the proposed changes in TSTF-449. Since these proposed changes are consistent with the plant's licensing basis and generally consistent with the standard TSs, the staff determined they were acceptable.

In summary, the staff determined that the differences between the proposed changes and TSTF-449 are acceptable and the model SE is applicable to this review.

Consistent with TSTF-449, the proposed TS changes include revisions to: (1) the definitions of LEAKAGE and PRESSURE BOUNDARY LEAKAGE, (2) TS 3/4.4.5, "Steam Generator Tube

Integrity," requirements, and (3) TS 3/4.4.6, "Reactor Coolant System Leakage." They also include a new TS 6.8.4.I, "Steam Generator (SG) Program," and a new TS 6.9.1.12, "Steam Generator Tube Inspection Report." Finally, the Index pages are revised to reflect the proposed changes.

### 3.2 Staff Evaluation

The proposed TS changes establish a programmatic, largely performance-based regulatory framework for ensuring SG tube integrity is maintained. The NRC staff finds that it addresses key shortcomings of the current framework by ensuring that SG programs are focused on accomplishing the overall objective of maintaining tube integrity. It incorporates performance criteria for evaluating tube integrity that the NRC staff finds consistent with the structural margins and the degree of leak tightness assumed in the current plant licensing basis. The NRC staff finds that maintaining these performance criteria provides reasonable assurance that the SGs can be operated safely without an increase in risk.

The revised TS contain limited specific details concerning how the SG Program is to achieve the required objective of maintaining tube integrity. This is intended to allow the licensee to have the flexibility to determine the specific strategy for meeting this objective. Nevertheless, the NRC staff finds that the proposed TS revisions include sufficient regulatory constraints on the establishment and implementation of the SG Program so as to provide reasonable assurance that tube integrity will be maintained.

Failure to meet the performance criteria will be reportable pursuant to the requirements in 10 CFR 50.72 and 50.73. The NRC Reactor Oversight Process provides a process by which the NRC staff can verify that the licensee has identified any SG Program deficiencies that may have contributed to a reportable occurrence and that appropriate corrective actions have been implemented.

In conclusion, the NRC staff finds that the TS changes proposed by the licensee in its April 24, 2006, application and September 14, 2006, supplement conform to the requirements of 10 CFR 50.36 and establish a TS framework that will provide reasonable assurance that SG tube integrity is maintained without undue risk to public health and safety.

The licensee's application included, for information, the revised TS Bases to be implemented with the TS changes. As previously noted, the NRC staff found them to be acceptable. The licensee will incorporate these changes in accordance with its NRC-approved TS Bases Control Program.

### 4.0 STATE CONSULTATION

Based upon a letter dated May 2, 2003, from Michael N. Stephens of the Florida Department of Health, Bureau of Radiation Control, to Brenda L. Mozafari, Senior Project Manager, U.S. Nuclear Regulatory Commission, the State of Florida does not desire notification of issuance of license amendments.

### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes



surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (71 FR 40746). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Date: January 30, 2007

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