



U.S. NUCLEAR REGULATORY COMMISSION
STANDARD REVIEW PLAN

9.4.4 TURBINE AREA VENTILATION SYSTEM

REVIEW RESPONSIBILITIES

Primary - Organization responsible for the review of ventilation and air filtration

I. AREAS OF REVIEW

The function of the turbine area ventilation system (TAVS) is to maintain ventilation, permit personnel access, and control the concentration of airborne radioactive material in the turbine area during normal operation, anticipated operational occurrences, and after any accident that results in a release of radioactive material.

The staff reviews the TAVS from the air intake to the point of discharge to ensure compliance with the requirements of General Design Criteria (GDCs) 2, 5, and 60. The review includes components such as air intakes, ducts, air-conditioning units, blowers, isolation dampers, filters, and exhaust fans. The review of the TAVS includes systems contained in the turbine building and their relationship, if any, to safety-related equipment areas.

The specific areas of review are as follows:

1. The functional performance requirements and the methods and equipment provided for air treatment equipment for the TAVS to determine whether the ventilation system or portions of the system have been designed or need to be designed as a safety-related system. In making this determination, systems provided for heating, ventilating, and air

Rev. 3 - [Month] 2007

USNRC STANDARD REVIEW PLAN

This Standard Review Plan, NUREG-0800, has been prepared to establish criteria that the U.S. Nuclear Regulatory Commission staff responsible for the review of applications to construct and operate nuclear power plants intends to use in evaluating whether an applicant/licensee meets the NRC's regulations. The Standard Review Plan is not a substitute for the NRC's regulations, and compliance with it is not required. However, an applicant is required to identify differences between the design features, analytical techniques, and procedural measures proposed for its facility and the SRP acceptance criteria and evaluate how the proposed alternatives to the SRP acceptance criteria provide an acceptable method of complying with the NRC regulations.

The standard review plan sections are numbered in accordance with corresponding sections in the Regulatory Guide 1.70, "Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (LWR Edition)." Not all sections of the standard format have a corresponding review plan section. The SRP sections applicable to a combined license application for a new light-water reactor (LWR) will be based on Regulatory Guide 1.206, "Combined License Applications for Nuclear Power Plants (LWR Edition)," until the SRP itself is updated.

These documents are made available to the public as part of the NRC's policy to inform the nuclear industry and the general public of regulatory procedures and policies. Individual sections of NUREG-0800 will be revised periodically, as appropriate, to accommodate comments and to reflect new information and experience. Comments may be submitted electronically by email to NRR_SRP@nrc.gov.

Requests for single copies of draft or active SRP sections (which may be reproduced) should be made to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Reproduction and Distribution Services Section, or by fax to (301) 415-2289; or by email to DISTRIBUTION@nrc.gov. Electronic copies of this section are available through the NRC's public Web site at <http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr0800/>, or in the NRC's Agencywide Documents Access and Management System (ADAMS), at <http://www.nrc.gov/reading-rm/adams.html>, under Accession # [MLxxxxxxx](#).

conditioning of the turbine area, designed to normal industrial standards, and those systems that provide for control and filtration of small quantities of radioactive gas leakage in the turbine area during normal plant operation, are not considered safety-related for the purpose of this Standard Review Plan (SRP) section.

Based on this determination, any safety-related portions of the system are reviewed with respect to functional performance requirements during adverse environmental occurrences, normal operation, anticipated operational occurrences, and subsequent to postulated accidents, including the loss of offsite power. Safety-related portions of the system are reviewed to ensure:

- A. A single, active failure can not result in loss of system functional performance capability.
 - B. Failures of nonseismic Category I equipment or components will not affect the TAVS.
2. Safety-related portions of the TAVS are also reviewed with respect to the following:
- A. The capability to direct ventilation air from areas of low radioactivity to areas of higher radioactivity.
 - B. The capability to detect the need for isolation and to isolate portions of the system in the event of failures or malfunctions, and the capability of the system to function under such conditions.
 - C. The capability to actuate components not normally operating that are required to operate during accident conditions and to provide necessary isolation.
3. Inspection, Test, Analysis, and Acceptance Criteria (ITAAC). For design certification (DC) and combined license (COL) reviews, the applicant's proposed information on the ITAAC associated with the SSCs related to this SRP section is reviewed in accordance with SRP Section 14.3, "Inspections, Tests, Analyses, and Acceptance Criteria - Design Certification." The staff recognizes that the review of ITAAC is performed after review of the rest of this portion of the application against acceptance criteria contained in this SRP section. Furthermore, the ITAAC are reviewed to assure that all SSCs in this area of review are identified and addressed as appropriate in accordance with SRP Section 14.3.
4. COL Action Items and Certification Requirements and Restrictions. COL action items may be identified in the NRC staff's final safety evaluation report (FSER) for each certified design to identify information that COL applicants must address in the application. Additionally, DCs contain requirements and restrictions (e.g., interface requirements) that COL applicants must address in the application. For COL applications referencing a DC, the review performed under this SRP section includes information provided in response to COL action items and certification requirements and restrictions pertaining to this SRP section, as identified in the FSER for the referenced certified design.

Review Interfaces

Other SRP sections interface with this section as follows:

1. Sections 3.2.1 and 3.2.2: determination of the acceptability of the seismic and quality group classifications for system components.
2. Sections 3.3.1, 3.3.2, 3.5.3, 3.7.1 through 3.7.4, 3.8.4, and 3.8.5: determination of the acceptability of the design analyses, procedures, and criteria that establish the ability of seismic Category I structures housing the system and supporting systems to withstand the effects of natural phenomena like the safe shutdown earthquake, the probable maximum flood, and tornado missiles.
3. Sections 3.9.1 through 3.9.3: determination that components, piping, and structures are designed in accordance with applicable codes and standards.
4. Section 3.9.6: review of the adequacy of the inservice testing program of pumps and valves.
5. Section 3.10: review of the seismic qualification of Category I instrumentation and electrical equipment.
6. Section 6.6: verification that inservice inspection requirements are met for system components.
7. Sections 7.7 and 8.3.1: determination of the adequacy of the design, installation, inspection, and testing of all essential electrical components (sensing, control, and power) required for proper operation.
8. Section 11.5: evaluation of the capability of the system to detect and control leakage of radioactive contamination.
9. Section 12.3-4: evaluation of the capability of the system to meet radiation protection criteria.
10. Section 16.0: review of proposed technical specifications.
11. Chapter 17: review of the quality assurance program.

The specific acceptance criteria and review procedures are contained in the referenced SRP sections.

II. ACCEPTANCE CRITERIA

Requirements

Acceptance criteria are based on meeting the relevant requirements of the following Commission regulations:

1. GDC 2, "Design Bases for Protection Against Natural Phenomena," as related to the system being capable of withstanding the effects of earthquakes.

2. GDC 5, "Sharing of Structures, Systems, and Components," as related to shared systems and components important to safety.
3. GDC 60, "Control of Release of Radioactive Materials to the Environment," as related to the system's capability to suitably control release of gaseous radioactive effluents to the environment.
4. 10 CFR 52.47(a)(1)(vi), as it relates to ITAAC (for design certification) sufficient to assure that the SSCs in this area of review will operate in accordance with the certification.
5. 10 CFR 52.97(b)(1), as it relates to ITAAC (for combined licenses) sufficient to assure that the SSCs in this area of review have been constructed and will be operated in conformity with the license and the Commission's regulations.

SRP Acceptance Criteria

Specific SRP acceptance criteria acceptable to meet the relevant requirements of the NRC's regulations identified above are as follows for review described in Subsection I of this SRP section. The SRP is not a substitute for the NRC's regulations, and compliance with it is not required. However, an applicant is required to identify differences between the design features, analytical techniques, and procedural measures proposed for its facility and the SRP acceptance criteria and evaluate how the proposed alternatives to the SRP acceptance criteria provide acceptable methods of compliance with the NRC regulations.

1. For GDC 2, acceptance is based on the guidance of RG 1.29, Position C.1 for safety-related portions and Position C.2 for nonsafety-related portions.
2. For GDC 5, acceptance is based on the determination that the use of the TAVS in multiple-unit plants during an accident in one unit does not significantly affect the capability to conduct a safe and orderly shutdown and cool-down in the remaining unit(s).
3. For GDC 60, acceptance is based on guidance of RGs 1.52 and 1.140 as related to design, inspection, testing, and maintenance criteria for post-accident and normal atmosphere cleanup systems, ventilation exhaust systems, air filtration, and adsorption units of light-water-cooled nuclear power plants. For RG 1.52 rev 2, the applicable regulatory position is C.2. For RG 1.52 rev 3, the applicable regulatory position is C.3. For RG 1.140 rev 1, the applicable regulatory positions are C.1 and C.2. For RG 1.140 rev 2, the applicable regulatory positions are C.2 and C.3.

Technical Rationale

The technical rationale for application of these requirements and/or SRP acceptance criteria to the areas of review addressed by this SRP section is discussed in the following paragraphs:

1. GDC 2, as related to the system being capable of withstanding the effects of earthquakes, requires that SSCs important to safety be designed to withstand the effects of a design-basis earthquake without loss of capability to perform their safety functions.

The function of the TAVS is to maintain ventilation, to permit personnel access, and to control airborne radioactivity in the turbine area during normal operation and anticipated operational occurrences and during and after postulated accidents, including loss of offsite power. This requirement ensures that, during and after a design-basis earthquake, essential portions of the TAVS will remain functional and that the failure of nonessential portions of the system or of other systems not designed to seismic Category I standards will not result in offsite doses in excess of 5 mSv (0.5 rem) to the whole body or an equivalent dose to any part of the body.

Meeting the GDC 2 requirements ensures that the TAVS will operate as designed, thus providing protection against release of radioactivity exceeding regulatory limits.

2. GDC 5 requires that SSCs important to safety shall not be shared among nuclear power units unless it can be shown that such sharing will not significantly impair their ability to perform safety functions, including, in the event of an accident in one unit, an orderly shutdown and cooldown of the remaining units.

For the TAVS, GDC 5 requires that the component parts of the TAVS be essentially independent to ensure that an accident in one unit of a multiple-unit facility will not propagate to other units. Therefore, the TAVS for each unit should be designed to accommodate the loads resulting from accident conditions. At the same time, the operating environment of equipment associated with unaffected units must be maintained within specified limits.

Meeting the GDC 5 requirements adds assurance that a failure or accident in one unit will not affect additional units of a multiple-unit site.

3. GDC 60 requires provisions to be included in the nuclear power unit design to ensure suitable controls on the release of radioactive materials in gaseous effluents during normal reactor operation, including anticipated operational occurrences.

GDC 60 requirements apply to the design of the TAVS because its function is to control the quantities of radioactive materials in gaseous effluents released to the environment from normal ventilation systems. RGs 1.140 and 1.52 offer design, testing, and maintenance criteria acceptable to the staff for air filtration and adsorption units of normal ventilation exhaust systems and for engineered safety-feature atmospheric cleanup systems in light-water-cooled nuclear power plants.

Meeting the GDC 60 requirements adds assurance that release of radioactive materials entrained in gaseous effluents will not exceed the limits specified in 10 CFR Part 20 for normal operation and anticipated operational occurrences.

III. REVIEW PROCEDURES

The reviewer will select and emphasize material from the procedures described below, as may be appropriate for a particular case.

For each area of review specified in subsection I of this SRP section, the review procedure is identified below. These review procedures are based on the identified SRP acceptance criteria.

For deviations from these specific acceptance criteria, the staff should review the applicant's evaluation of how the proposed alternatives to the SRP criteria provide an acceptable method of complying with the relevant NRC requirements identified in subsection II.

The procedures are used during the construction permit or standard review to determine that the design criteria and bases and the preliminary design, as set forth in the preliminary SAR, meet the acceptance criteria of subsection II of this SRP section.

For the review of operating license (OL) or COL applications, the procedures verify that the initial design criteria and bases have been implemented appropriately in the final design as set forth in the final SAR.

The procedures for OL or COL reviews include a determination that the proposed technical specifications agree with the requirements for testing, minimum performance, and surveillance developed by the staff.

The primary reviewer coordinates this review with other reviewers for their particular areas of responsibility. The primary reviewer uses such inputs as required to complete this review procedure.

As a result of various TAVS designs proposed by applicants, there will be variations in system requirements. For purposes of this SRP section, a typical system is assumed to have fully redundant subsystems, each with an identical essential (safety-related) portion. For deviations from this typical arrangement, the reviewer adjusts the review procedures below; however, the system design must comply with the SRP acceptance criteria and the relevant NRC requirements identified in subsection II.

1. The SAR is reviewed to verify that the system description and piping and instrumentation diagrams (P&IDs) show the TAVS equipment used for normal and emergency operations, and the ambient temperature limits for the areas serviced. The system performance requirements are reviewed to determine that it describes allowable component operational degradation (e.g., loss of function, damper leakage) and describes the procedures that will be followed to detect and correct these conditions. The reviewer, using results from failure modes and effects analyses as appropriate, determines that the safety-related portion of the system is capable of functioning in spite of the loss of any active component.

The system review also should demonstrate compliance with applicable industry standards: American National Standards Institute/American Nuclear Society (ANSI/ANS) 59.2-1985, "Safety Criteria for Nuclear Power Plant HVAC Systems Located Outside Primary Containment," and American National Standards Institute/American Society of Mechanical Engineers (ANSI/ASME) AG-1-1985, "Code on Nuclear Air and Gas Treatment."

2. The system P&IDs, layout drawings, and component descriptions and characteristics are then reviewed to determine that:
 - A. Essential portions of the TAVS are correctly identified and are isolable from nonessential portions of the system. The P&IDs are reviewed to verify that they clearly indicate the physical divisions between each portion and indicate design

classification changes. System drawings are also reviewed to verify that they show the means for accomplishing isolation, and the system description is reviewed to identify minimum performance requirements for the isolation dampers.

For the typical system, the drawings and description are reviewed to verify that two automatically operated isolation dampers in series separate nonessential portions and components from the essential portions.

- B. Essential portions of the TAVS, including the isolation dampers separating essential from nonessential portions, are classified seismic Category I. Component and system descriptions in the SAR that identify mechanical and performance characteristics are reviewed to verify that the above seismic classifications have been included, and that the P&IDs indicate any points of change in design classification.
 - C. Design provisions have been made that permit appropriate inservice inspection and functional testing of system components important to safety. Compliance with the industry standard American Society for Testing and Materials (ASTM) D3803-89, "Standard Test Methods for Radiological Testing of Nuclear-Grade Gas-Phase Adsorbers," should be demonstrated. It is acceptable if the SAR information delineates a testing and inspection program and if the system drawings show the necessary test recirculation loops around fans or isolation dampers that would be required by this program.
3. The reviewer verifies that the system has been designed so that system function will be maintained as required, in the event of an earthquake or loss of offsite power. The reviewer evaluates the system, using engineering judgment and the results of failure modes and effects analyses to determine that:
- A. The failure of nonessential portions of the system or of other systems not designed to seismic Category I standards and located close to essential portions of the system or of nonseismic Category I structures that house, support, or are close to essential portions of the TAVS, will not preclude their operation. Reference to SAR sections describing site features and the general arrangement and layout drawings and to the SAR tabulation of seismic design classifications for structures and systems will be necessary. Statements in the SAR verifying that the above conditions are met are acceptable.
 - B. Components and subsystems necessary for preventing releases of radioactive contaminants can function as required in the event of loss of offsite power. The system design will be acceptable if the TAVS meets minimum system requirements as stated in the SAR assuming a failure of a single active component, within the system itself, or in the auxiliary electric power source which supplies the system. The SAR is reviewed to verify that, for each TAVS component or subsystem affected by the loss of offsite power, the resulting system flow capacity will not cause the loss of preferred direction of air flow from areas of low potential radioactivity to areas of higher potential radioactivity. Statements in the SAR and the results of failure modes and effects analyses are

considered in verifying that the system meets these requirements. This will be an acceptable verification of system functional reliability.

4. The descriptive information, P&IDs, TAVS drawings, and failure modes and effects analyses in the SAR are reviewed to ensure that essential portions of the system can function following design basis accidents assuming a concurrent single active failure. The reviewer evaluates the analyses presented in the SAR to ensure function of required components, traces the availability of these components on system drawings, and checks that the SAR contains verification that minimum system isolation or filtration requirements are met for each accident situation for the required time spans. For each case the design will be acceptable if minimum system requirements are met.
5. For reviews of DC and COL applications under 10 CFR Part 52, the reviewer should follow the above procedures to verify that the design set forth in the safety analysis report, and if applicable, site interface requirements meet the acceptance criteria. For DC applications, the reviewer should identify necessary COL action items. With respect to COL applications, the scope of the review is dependent on whether the COL applicant references a DC, an ESP, or other NRC-approved material, applications, and/or reports.

After this review, SRP Section 14.3 should be followed for the review of Tier I information for the design, including the postulated site parameters, interface criteria, and ITAAC.

IV. EVALUATION FINDINGS

The reviewer verifies that the applicant has provided sufficient information and that the review and calculations (if applicable) support conclusions of the following type to be included in the staff's safety evaluation report (SER). The reviewer also states the bases for those conclusions.

The TAVS includes all components and ducting from air intake to the point of discharge. All portions of the system whose failure may result in release of radioactivity which causes an offsite dose of more than 5 mSv (0.5 rem) to the whole body or an equivalent dose to any part of the body shall be classified seismic Category I and safety related. Based on the review of the applicant's proposed design criteria, the design bases and safety classification for the TAVS and the requirements (if any) for system performance to preclude any adverse effect on safety-related functions during all conditions of plant operation, the staff concludes that the design of the turbine area ventilation system and supporting systems complies with NRC regulations as set forth in GDC 2, "Design Bases for Protection Against Natural Phenomena," GDC 5, "Sharing of Structures, Systems, and Components," and GDC 60, "Control of Releases of Radioactive Materials to the Environment." This conclusion is based on the following findings:

1. The applicant has met the requirements of GDC 2, "Design Bases for Protection Against Natural Phenomena," with respect to the system being capable of withstanding the effects of earthquakes by meeting the guidelines of Regulatory Guide 1.29, "Seismic Design Classification," Position C.1 for safety-related portions of the system and Position C.2 for nonsafety-related portions of the system.

2. The applicant has met the requirements of GDC 5, "Sharing of Structures, Systems, and Components Important to Safety to Perform Required Safety Function," with respect to capability of shared systems and components important to safety to perform required safety functions.
3. The applicant has met the requirements of GDC 60, "Control of Releases of Radioactive Materials to the Environment," with respect to the capability of the system to suitably control release of gaseous radioactive effluents to the environment by meeting the guidelines of RGs 1.52 and 1.140 as related to design, inspection, testing, and maintenance criteria for post-accident and normal atmosphere cleanup systems, ventilation exhaust systems, air filtration, and adsorption units of light-water-cooled nuclear power plants. For RG 1.52 rev 2, the applicable regulatory position is C.2. For RG 1.52 rev 3, the applicable regulatory position is C.3. For RG 1.140 rev 1, the applicable regulatory positions are C.1 and C.2. For RG 1.140 rev 2, the applicable regulatory positions are C.2 and C.3.

The staff concludes that the TAVS design complies with all applicable GDCs and RG positions cited and is, therefore, acceptable.

For DC and COL reviews, the findings will also summarize (to the extent that the review is not discussed in other SER sections) the staff's evaluation of the ITAAC, including design acceptance criteria, as applicable, and interface requirements and combined license action items relevant to this SRP section.

V. IMPLEMENTATION

The staff will use this SRP section in performing safety evaluations of DC applications and license applications submitted by applicants pursuant to 10 CFR Part 50 or 10 CFR Part 52. Except when the applicant proposes an acceptable alternative method for complying with specified portions of the Commission's regulations, the staff will use the method described herein to evaluate conformance with Commission regulations.

The provisions of this SRP section apply to reviews of applications docketed six months or more after the date of issuance of this SRP section, unless superceded by a later revision.

Implementation schedules for conformance to parts of the method discussed herein are contained in the referenced regulatory guides.

VI. REFERENCES

1. 10 CFR Part 50, Appendix A, GDC 2, "Design Bases for Protection Against Natural Phenomena."
2. 10 CFR Part 50, Appendix A, GDC 5, "Sharing of Structures, Systems, and Components."
3. 10 CFR Part 50, Appendix A, GDC 60, "Control of Releases of Radioactive Materials to the Environment."

4. RG 1.29, "Seismic Design Classification."
5. RG 1.140, "Design, Testing, and Maintenance Criteria for Normal Ventilation Exhaust System Air Filtration and Adsorption Units of Light-Water-Cooled Nuclear Power Plants."
6. RG 1.52, "Design, Testing, and Maintenance Criteria for Post Accident Engineered-Safety-Feature Atmospheric Cleanup System Air Filtration and Adsorption Units of Light-Water-Cooled Nuclear Power Plants."
7. NRC Inspection Manual Chapter IMC-2504, "Construction Inspection Program - Non-ITAAC Inspections," issued April 25, 2006.

PAPERWORK REDUCTION ACT STATEMENT

The information collections contained in the draft Standard Review Plan are covered by the requirements of 10 CFR Part 50 and 10 CFR Part 52, which were approved by the Office of Management and Budget, approval number 3150-0011 and 3150-0151.

PUBLIC PROTECTION NOTIFICATION

The NRC may not conduct or sponsor, and a person is not required to respond to, a request for information or an information collection requirement unless the requesting document displays a currently valid OMB control number.

SRP Section 9.4.4
Description of Changes

This SRP section affirms the technical accuracy and adequacy of the guidance previously provided in Draft Revision 3, dated April 1996, of this SRP section. See ADAMS accession number ML040900015.

In addition this SRP section was administratively updated in accordance with NRR Office Instruction, LIC-200, Revision 1, "Standard Review Plan (SRP) Process." The revision also adds standard paragraphs to extend application of the updated SRP section to prospective submittals by applicants pursuant to 10 CFR Part 52.

The technical changes are incorporated in Revision 3, dated [Month] 2007:

Review Responsibilities - Reflects changes in review branches resulting from reorganization and branch consolidation. Change is reflected throughout the SRP.

I. AREAS OF REVIEW

The area of review describing the other review responsibilities of the primary review organization has been removed.

II. ACCEPTANCE CRITERIA

The acceptance criteria for GDC 60 have been changed to include the 2001 revisions of RGs 1.52 and 1.140.

III. REVIEW PROCEDURES

None

IV. EVALUATION FINDINGS

The finding for GDC 60 has been changed to include the 2001 revisions of RGs 1.52 and 1.140.

V. IMPLEMENTATION

None

VI. REFERENCES

None