

November 29, 2006

Mr. Peter T. Dietrich
Site Vice President
James A. FitzPatrick Nuclear Power Plant
Entergy Nuclear Operation, Inc.
P.O. Box 110
Lycoming, NY 13093-0110

SUBJECT: REQUESTS FOR ADDITIONAL INFORMATION REGARDING THE REVIEW
OF THE LICENSE RENEWAL APPLICATION FOR JAMES A. FITZPATRICK
NUCLEAR POWER PLANT (TAC NO. MD2667)

By letter dated July 31, 2006, Entergy Nuclear Operations, Inc., submitted an application pursuant to 10 CFR Part 54, to renew the operating license for James A. FitzPatrick Nuclear Power Plant for renew by the U.S. Nuclear Regulatory Commission (NRC). The NRC staff is reviewing the information contained in the license renewal application and has identified, in the enclosure, an area where additional information is needed to complete the review.

Based on discussions with Mr. Rick Plasse of your staff, a mutually agreeable date for your response is within 30 days from the date of this letter. If you have any questions regarding this letter or if circumstances result in your need to revise the response date, please contact me at 301-415-1458 or by e-mail at nbl@nrc.gov.

Sincerely,

/RA/

N. B. (Tommy) Le, Senior Project Manager
License Renewal Branch B
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosure:
As stated

cc w/encl: See next page

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ADAMS Accession No.: **ML063200126**

OFFICE	PM:RLRB:DLR	LA:RLRA:DLR	BC:RLRB:DLR
NAME	NBLe	IKing	RAuluck
DATE	11/24/06	11/22/06	11/29/06

OFFICIAL RECORD COPY

Letter to Peter Dietrich from N B Le dated November 29, 2006

SUBJECT: REQUESTS FOR ADDITIONAL INFORMATION REGARDING THE REVIEW
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NUCLEAR POWER PLANT (TAC NO. MD2667)

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REQUEST FOR ADDITIONAL INFORMATION
JAMES A. FITZPATRICK LICENSE RENEWAL APPLICATION
SECTION 2.4 SCOPING AND SCREENING RESULTS: STRUCTURES

RAI 2.4.1-1

Section 2.4-1, Reactor Building and Primary Containment of the LRA, under Inner Refueling Bellows and Bulkhead Assembly does not clearly state the makeup of the bulkhead assembly. The staff requests the applicant to provide a list of bulkhead assembly components that are within the scope of license renewal and a drawing that depicts the bulkhead assembly.

RAI 2.4.1-2

Table 2.4-1, Reactor Building of the LRA, does not include drywell head closure bolts and double-gasketed drywell head. These components have the intended function of the drywell pressure boundary. The staff requests the applicant to provide justification for not including them in the scope of license renewal.

RAI 2.4.1-3

Table 2.4-1, Reactor Building of the LRA, does not incorporate refueling cavity seal components within the scope of license renewal. The proposed license renewal interim staff guidance LR-ISG-2006-01, "Plant Specific Aging Management Program for Inaccessible Areas of Boiling Water Reactor Mark 1 Steel Containment Drywell Shell," which was published in the Federal Register on May 9, 2006, stated that the most likely cause of corrosion of the drywell shell in the sand-pocket areas (near the bottom of the drywell), and in the spherical portion of the drywell at higher elevations, was the water in the gap between the drywell and the concrete shield, and the source of the water was noted as leakage through the seal between the drywell and the refueling cavity and leakage through cracked stainless steel liner of the refueling cavity wall. Therefore, the staff requests the applicant include all the refueling cavity seal components within the scope of license renewal and provide a drawing that depicts the refueling cavity seal components.

RAI 2.4.1-4

Table 2.4-1, Reactor Building of the LRA, does not include the metal drywell sump screens that provide functional support for safety-related equipment. The staff requests the applicant to provide justification for not including them in the scope of license renewal.

RAI 2.4.1-5

Table 2.4-1, Primary Containment of the LRA, does not include the reinforced concrete shield plugs that provide shielding over the top of the drywell. Exclusion of the reinforced concrete shield plugs from the scope of license renewal may lead to long-term unmanaged degradation of the plugs (e.g., full sectional concrete cracking, rebar corrosion, loss of bond, partial spalling or cracking of concrete due to handling, loss of load carrying capacity of plug attachments, etc.) with a seismic II/I implication, potentially affecting the structural integrity of the drywell head. The staff requests the applicant to provide justification for not including them in the scope of license renewal.

Enclosure

RAI 2.4.1-6

Table 2.4-1, Primary Containment of the LRA, does not incorporate spent fuel racks' neutron absorbing material within the scope of license renewal. Long-term unmanaged degradation of the component may excessively reduce the margin of nuclear subcriticality in the fuel pool. The staff requests the applicant to provide justification for not including them in the scope of license renewal.

RAI 2.4.2-1

Section 2.4-2, Water Control Structures of the LRA, under Discharge Tunnel states that "failure of this Class II structure could impact the proper operation of the emergency service water system." The staff needs additional information regarding the potential consequences of the Screenwell-Pumphouse (Class II structure) failure. Specifically, the licensee is requested to confirm that appropriate measures have been taken to preclude the failure of this structure which might have adverse effects on the proper operation of the emergency service water system.

RAI 2.4.2-2

Table 2.4-2, Water Control Structures of the LRA, do not include anchors, and post-tensioned tendons for the Intake Structure that anchor the main structure to the natural bedrock below the lake bottom. The staff requests the applicant to provide justification for not including them in the scope of license renewal and also provide a summary of operating experience regarding settlement of the intake structure.

RAI 2.4.2-3

Table 2.4-2, Water Control Structures of the LRA, lists "Beams, columns, floor slabs and walls" as a component and "Exterior walls" as another component. The staff requests the applicant to provide clarifications for the two different components by listing all structural members under each component.

RAI 2.4.3-1

Section 12.3.9, Main Stack of the UFSAR, states that "The stack is not specifically designed to resist a tornado." The staff needs additional information regarding the potential interaction between some seismic Class II SSCs and the tornado-induced failure of the main stack. Specifically, the licensee is requested to confirm that appropriate measures have been taken to preclude potential interactions between the main stack and some other nearby seismic Class I and Class II SSCs, whose failure might have adverse effects on seismic Class I SSCs.

RAI 2.4.3-2

Section 2.4-3, Turbine Building Complex and Yard Structures, of the LRA under Turbine Building (TB), including Feedwater Heater Bay (HBAY) states that "The main steam lines to the turbine generator from the reactor are housed in a reinforced concrete tunnel that enters the turbine building after passing under the adjacent administration building." The staff needs

additional information regarding the potential interaction between the administration building and the failure of main steam lines. Specifically, the licensee is requested to confirm that appropriate measures have been taken to preclude the failure of the main steam lines that might have adverse effects on the administration building.

RAI 2.4.3.-3

Table 2.4-3, Turbine Building Complex and Yard Structures of the LRA, lists "Exterior walls" as a component. The staff is unable to determine whether this component include main stack, main steam line tunnel, and electric bay tunnel. If not, the staff requests the applicant to include these components in the scope of license renewal.

RAI 2.4.3-4

Table 2.4-3, Turbine Building Complex, and Yard Structures of the LRA, does not include sumps that provide functional support for safety and nonsafety-related equipment. The staff requests the applicant to provide justification for not including them in the scope of license renewal.

RAI 2.4.4-1

Table 2.4-4, Bulk Commodities of the LRA, provides a list of bulk commodities for the Fitzpatrick Nuclear Power Plant. The location of each component is unclear to the staff. The staff requests the applicant to describe all commodities on the list as well as to provide a comprehensive listing of components, component supports and locations for each commodity.

RAI 2.4.4-2

Table 2.4-4, Bulk Commodities of the LRA, lists "Insulation" with its intended functions. It is unclear to the staff why "Support for Criterion (a)(1) equipment" is not part of the intended functions when Table 2.0-1 defines the intended function of insulation as "provide insulating characteristics to reduce heat transfer," which is meant to apply to safety related and non-safety related components. The staff requests the applicant to provide additional information on insulation and a list of in-scope components that have insulation included with their intended functions.

FitzPatrick Nuclear Power Plant

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FitzPatrick Nuclear Power Plant

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