



10 CFR 50.90

LR-N06-0418
LCR H05-01, Rev. 1
October 20, 2006

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Hope Creek Generating Station
Facility Operating License No. NPF-57
NRC Docket No. 50-354

Subject: Supplement to License Amendment Request for Extended Power Uprate

Reference: 1) PSEG letter LR-N06-0286, Request for License Amendment: Extended Power Uprate, September 18, 2006

In Reference 1, PSEG Nuclear LLC (PSEG) requested an amendment to Facility Operating License NPF-57 and the Technical Specifications (TS) for the Hope Creek Generating Station (HCGS) to increase the maximum authorized power level to 3840 megawatts thermal (MWt).

In a telephone conference call on October 3, 2006, the NRC staff stated that some of the documents listed as references in the "Steam Dryer Evaluation" (Attachment 7 to Reference 1) would be required to be submitted for NRC review. In response, CDI Technical Memorandum 06-23P, "Comparison of the Hope Creek and Quad Cities Steam Dryer Loads at EPU Conditions" is provided in Attachment 1 to this letter. Attachment 1 contains information which Continuum Dynamics, Inc. (CDI) considers to be proprietary. CDI requests that the proprietary information be withheld from public disclosure in accordance with 10 CFR 2.390(a)(4). An affidavit supporting this request is provided in Attachment 1. A non-proprietary version of CDI Technical Memorandum 06-23P is provided in Attachment 2.

In a telephone conference call on October 10, 2006, the NRC staff requested that PSEG provide the calculations performed to assess the impact of Extended Power Uprate on the radiological consequences of design basis accidents described in the HCGS Updated Final Safety Analysis Report. The requested calculations are provided

ADD 1

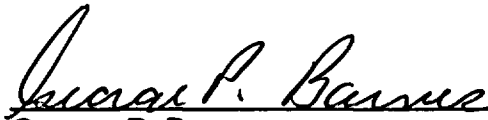
in Attachments 3 through 8 to this letter. The post-Instrument Line Pipe Break Accident (ILPBA) doses reported in the Attachment 4 calculation are higher than those contained in Table 9-4 of the "Safety Analysis for Hope Creek Constant Pressure Power Uprate," NEDC-33076P, Revision 2, (Reference 1). The assumed fraction of coolant which flashes to steam is higher in the Attachment 4 calculation than in the analysis described in NEDC-33076P, Revision 2. The post-ILPBA doses remain within their applicable regulatory limits. The issue involving incorrect values in NEDC-33076P, Revision 2, Table 9-4 has been entered in PSEG's Corrective Action Program.

PSEG has determined that the information contained in this letter and attachments does not alter the conclusions reached in the 10CFR50.92 no significant hazards analysis previously submitted.

Should you have any questions regarding this submittal, please contact Mr. Paul Duke at 856-339-1466.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on 10/20/06
(date)


George P. Barnes
Site Vice President – Hope Creek

Attachments (8)

1. CDI Technical Memo 06-23P (Proprietary), "Comparison of the Hope Creek and Quad Cities Steam Dryer Loads at EPU Conditions," Revision 0, dated September 2006
2. CDI Technical Memo 06-23P (Non-Proprietary), "Comparison of the Hope Creek and Quad Cities Steam Dryer Loads at EPU Conditions," Revision 0, dated September 2006
3. Calculation No. H-1-AB-MDC-1854, Revision 1IR0, "Main Steam Line Break Accident"
4. Calculation No. H-1-BG-MDC-1859, Revision 1, "Instrument Line Pipe Break Accident"
5. Calculation No. H-1-ZZ-MDC-1880, Revision 2IR0, "Post-LOCA EAB, LPZ and CR Doses"
6. Calculation No. H-1-ZZ-MDC-1929, Revision 0, "Fuel Handling Accident Radiological Consequences"
7. Calculation No. H-1-CG-MDC-1795, Revision 4, "Control Rod Drop Accident Radiological Consequences"
8. Calculation No. H-1-AE-MDC-1868, Revision 1, "Feedwater Line Break Accident Outside Primary Containment"

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cc: S. Collins, Regional Administrator – NRC Region I
S. Bailey, Project Manager - Hope Creek, USNRC
NRC Senior Resident Inspector - Hope Creek
K. Tosch, Manager IV, NJBNE