

April 11, 2007

Mark B. Bezilla  
Vice President  
FirstEnergy Nuclear Operating Company  
Davis-Besse Nuclear Power Station  
Mail Stop A-DB-3080  
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SUBJECT: NRC RECEIPT OF DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1,  
RESPONSES TO GENERIC LETTER 2003-01 "CONTROL ROOM  
HABITABILITY" (TAC NO. MB9796)

Dear Mr. Bezilla:

The Nuclear Regulatory Commission (NRC) acknowledges the receipt of your responses to Generic Letter (GL) 2003-01 "Control Room Habitability" dated August 11, 2003 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML032250649); August 13, 2004 (ADAMS Accession No. ML042310180); March 31, 2006 (ADAMS Accession No. ML060940119); and, March 27, 2007 (ADAMS Accession No. ML070880703) for the Davis-Besse Nuclear Power Station, Unit 1 (Davis-Besse). This letter provides a status of your responses and describes any additional information that may be necessary to consider your response to GL 2003-01 complete.

The GL requested that you confirm that your control rooms meet their design bases (e.g. General Design Criterion [GDC] 1, 3, 4, 5, and 19, draft GDC, or principal design criteria), with special attention to: (1) determination of the most limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in your design basis for meeting control room operator dose limits from accidents (GL 2003-01, Item 1a); (2) determination that the most limiting unfiltered inleakage is incorporated into your hazardous chemical assessments (GL 2003-01, Item 1b); and, (3) determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 2003-01, Item 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability, and plans to retire them (GL 2003-01, Item 2).

You reported the following results of American Society for Testing Materials (ASTM) E741, "Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution" tracer gas tests for the Davis-Besse control room which is pressurized for accident mitigation:

You determined that the maximum tested unfiltered in-leakage into the control room when the control room is pressurized is 0 cubic feet per minute (cfm), which is less than the 10 cfm assumed in the CRE (control room envelope) dose calculations when the control room is pressurized. You also determined that the maximum tested unfiltered inleakage into the control room was 60 cfm with the Control Room Emergency Ventilation System (CREVS) in recirculation mode, which is higher than the value of 55.4 cfm assumed in the control room dose calculation without taking credit for the CREVS. However, you stated that since

the CREVS will be initiated in the pressurization mode in approximately 10 minutes, there is no significant impact on control room doses due to slightly higher unfiltered inleakage rate than that assumed in the analyses. You stated that a small increase in the inleakage rate during the first 10 minutes of the accident will not impact the ability of the CREVS to maintain the dose below the GDC 19 guidelines. You also indicated that you are currently evaluating the impact of emergency core cooling system leakage into the auxiliary building and into the borated water storage tank, as well as radioactive material accumulated on charcoal adsorbers on the control room doses.

You indicated that you have evaluated both off-site and on-site stationary and mobile hazardous chemicals in accordance with Regulatory Guide 1.78, "Evaluating Control Room Habitability of a Nuclear Power Plant Control Room During a Postulated Hazardous Chemical Release," and concluded that unfiltered inleakage is not relevant to toxic gas assessments. You also indicated that reactor control capability is maintained from either the control room or the alternate shutdown locations in the event of smoke.

The information you provided also supported the fact that there are no compensatory measures needed to be in place to demonstrate control room habitability, although you did state that there are open corrective actions and administrative controls in place to limit leakage area during maintenance activities.

The information you provided also supported the conclusion that you are committed to meet the GDC regarding control room habitability.

GL 2003-01 further requested that you assess your Technical Specifications (TSs) to determine if you verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design-basis analysis for control room habitability, (GL 2003-01, Item 1c). In your March 31, 2006, response you indicated that you do not have a TS surveillance requirement for CRE integrity and that you would evaluate the incorporation of appropriate changes to your TSs within 90 days following NRC approval of TS Task Force Traveler No. 448 (TSTF-448) "Control Room Habitability." In your March 27, 2007, response you indicated that you would request to modify the Davis-Besse TSs based on TSTF-448 as part of the planned conversion to the improved standard TSs, currently scheduled for July 2007.

Your commitment, to submit a license amendment request (LAR) based on TSTF-448, is acceptable for purposes of closing out your response to GL 2003-01. The NRC staff will monitor the submittal of the LAR and interact with you as necessary during the amendment process.

M. Bezilla

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If you have any questions regarding this correspondence, please contact me at (301) 415-4037.

Sincerely,

*/RA/*

Thomas J. Wengert, Project Manager  
Plant Licensing Branch III-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-346

cc: See next page

M. Bezilla

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