

November 7, 2006

Mr. Timothy J. O'Connor  
Vice President, Nine Mile Point  
Nine Mile Point Nuclear Station, LLC  
P.O. Box 63  
Lycoming, NY 13093

SUBJECT: NUCLEAR REGULATORY COMMISSION (NRC) RECEIPT OF NINE MILE POINT NUCLEAR STATION, UNITS 1 AND 2 (NMP1 & 2), LICENSEE RESPONSE TO GENERIC LETTER (GL) 2003-01, "CONTROL ROOM HABITABILITY" (TAC NOS. MB9825 and MB9826)

Dear Mr. O'Connor:

The NRC acknowledges the receipt of your responses to GL 2003-01 "Control Room Habitability" dated August 11, 2003 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML032320280); March 29, 2004 (ADAMS Accession No. ML040970413); December 28, 2004 (ADAMS Accession No. ML050070173); January 31, 2005 (ADAMS Accession No. ML050460309); December 29, 2005 (ADAMS Accession No. ML060100460); December 30, 2005 (ADAMS Accession No. ML060120193); and, January 27, 2006 (ADAMS Accession No. ML060380518). This letter provides a status of your response and describes any actions that may be necessary to consider your response to GL 2003-01 complete.

GL 2003-01 requested that you confirm that your control rooms meet their design bases (e.g. General Design Criteria (GDCs) 1, 3, 4, 5, and 19, draft GDC, or principal design criteria), with special attention to: (1) determination of the most limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in your design bases for meeting control room operator dose limits from accidents (GL 2003-01, Item 1a); (2) determination that the most limiting unfiltered inleakage is incorporated into your hazardous chemical assessments (GL 2003-01, Item 1b); and, (3) determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 2003-01, Item 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability (CRH), and plans to retire them (GL 2003-01, Item 2).

You reported the results of ASTM E741 (American Society for Testing Materials, "Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution") tracer gas tests for the NMP1 & 2 control rooms which are in separate units.

For NMP1, you determined that the maximum tested value for inleakage into the control room envelope (CRE) was 45 scfm which is larger than the value of 30 scfm assumed in your current licensing basis. You stated that an evaluation based upon margin in the existing design-basis radiological analyses has been developed per GL 91-18 that shows that the requirements of GDC 19 will be met with up to 147 scfm unfiltered inleakage. You currently have a license

condition that requires you to have potassium iodide (KI) available to control room operators as compensatory measure. You plan to retire this compensatory measure by using alternative source term (AST) methodology, for which you committed to submit a license amendment request (LAR) by December 15, 2006.

For NMP2, you determined that the maximum tested value for inleakage into the CRE was 204 scfm which is larger than the value of 0 scfm assumed in your current design-basis radiological analysis. You stated that an evaluation has been developed per GL 91-18 using AST methodology that shows that the GDC 19 dose limits will not be exceeded with 225 scfm of unfiltered inleakage. You committed to submit an LAR to incorporate this AST methodology into your licensing basis by May 31, 2007.

You indicated that your 2004 review of offsite, onsite, and mobile sources of hazardous chemicals, per Regulatory Guide 1.78, Revision 1, "Evaluating the Habitability of a Nuclear Power Plant Control Room During a Postulated Hazardous Chemical Release," December 2001 (ADAMS Accession No. MI013100014), confirmed that no hazards exist for NMP1 & 2 control room personnel. You also indicated that reactor control capability is maintained from either the control room or the alternate shutdown panel in the event of smoke.

GL 2003-01 further requested that you assess your Technical Specifications (TSs) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design basis analysis for CRH, and in light of the demonstrated inadequacy of a delta ( $\Delta$ ) P measurement to alone provide such verification (GL 2003-01, Item 1.c) As permitted by the GL, you provided a schedule for revising the Surveillance Requirement (SR) in the TSs to reference an acceptable surveillance methodology. In your January 31, 2005, response you committed to submit an LAR to adopt TS SRs that verify CRH per Technical Specification Task Force (TSTF)-448, within 6 months following NRC approval of TSTF-448.

The information you provided also supported the conclusion that NMP1 is required to meet the principal design criteria and meets the intent of GDC 19 regarding CRH which is documented in your Updated Final Safety Analysis Report (UFSAR).

The information you provided also supported the conclusion that NMP2 is required to meet the GDC regarding CRH which is documented in your UFSAR.

Your commitment to submit an LAR to revise your licensing basis using AST methodology and revise your TS SRs based on TSTF-448, following our formal review and approval, is acceptable for purposes of closing out your response to GL 2003-01. The NRC staff will monitor submission of the LAR and interact with you as necessary during the amendment review process.

T. O'Connor

-3-

If you have any questions regarding this correspondence, please contact me at 301-415-1402.

Sincerely,

***/RA/***

Timothy G. Colburn, Project Manager  
Plant Licensing Branch I -1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-220 and 50-410

cc: See next page

T. O'Connor

-3-

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