

October 25, 2006

MEMORANDUM TO: Stacey L. Rosenberg, Chief
Special Projects Branch
Division of Policy and Rulemaking
Office of Nuclear Reactor Regulation

FROM: Tanya M. Mensah, Senior Project Manager */RA/*
Special Projects Branch
Division of Policy and Rulemaking
Office of Nuclear Reactor Regulation

SUBJECT: FORTHCOMING CATEGORY 3 PUBLIC WORKSHOP ON INTERIM
REACTIVITY-INITIATED ACCIDENTS (RIA) CRITERIA

DATE & TIME: Thursday, November 9, 2006
8:30 a.m. - 5:00 p.m.

LOCATION: U.S. Nuclear Regulatory Commission
Two White Flint North
11555 Rockville Pike, Room T-7A1
Rockville, Maryland 20852

PURPOSE: The NRC staff is convening this meeting for the purpose of seeking input which will be used in the development of the interim criteria for RIA. The interim criteria will appear as an update to the Standard Review Plan (NUREG-0800) and public comments will be addressed. Background information on the reactivity initiated accident (RIA) is available in Research Information Letter No. 0401 dated March 31, 2004 (Agencywide Documents Access and Management System Accession No. ML040920167).

CATEGORY 3*: This is a Category 3 meeting. The public is invited to participate in this meeting by providing comments and asking questions throughout the meeting. Interested members of the public can participate in this meeting via a toll-free teleconference. Please email the meeting contact by November 2, 2006, to obtain the telephone number and passcode.

PARTICIPANTS: Participants from the NRC include members of the Office of Nuclear Reactor Regulation (NRR) and Office of Research (RES).

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Project Nos. 669 and 689

Enclosure: Agenda

cc w/encl: See next page

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* Commission's Policy Statement on "Enhancing Public Participation in NRC Meetings," (67 FR 36920), May 28, 2002

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Date: October 25, 2006

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**Agenda for Reactivity-Initiated Accident (RIA) Public Workshop
November 9, 2006**

1. Two-Stage Approach to Revising RIA Criteria and Guidance

- a. NRC staff discussion on the following:
 - i. Schedule for interim criteria.
 - ii. Schedule for revising Regulatory Guide 1.77.

2. Control Rod Worth and Pulse Width

- a. Industry representatives should be prepared to address the following:
 - i. Control rod worths and its relation to fuel assembly burnup, core power level, and core operating limits for both Boiling Water Reactors (BWR) and Pressurized Water Reactors (PWR).
 - ii. The amount of time that a plant operates with the maximum calculated control rod worth and the conditions necessary to achieve this maximum worth.
 - iii. The axial length and location of high fuel pellet enthalpy.
 - iv. Calculated pulse widths for both BWRs and PWRs.

3. Core Coolability Criteria

- a. NRC presentation on draft criteria for long-term core coolability.
- b. Industry representatives should be prepared to address the following:
 - i. Fuel pellet microstructure and fission gas accumulation as a function of fuel rod burnup as its relation to grain boundary separation (fuel powdering) and fuel dispersal during a RIA.
 - ii. The effect of pulse width on fuel dispersal during a RIA.

4. Draft Fuel Rod Failure Threshold

- a. NRC presentation on draft fuel rod failure threshold.
- b. Industry representatives should be prepared to address the following:
 - i. BWR cold zero power scenarios and the effect of temperature on cladding failure threshold.
 - ii. The range of BWR coolant temperature and flow values at conditions during which Control Rod Drop Accident may occur.
 - iii. Fuel failure based upon post-Departure from Nucleate Boiling (dryout) conditions and the use of current methods and critical heat flux correlations to predict cladding failure.
 - iv. Hydrogen pickup fraction in BWR cladding versus PWR cladding.
 - v. The accuracy of predicted oxide thickness and use of best-estimate peak nodal oxide to convert corrosion-dependent criteria to a more useable burnup-dependent criteria.

5. Draft Guidance

- a. NRC presentation on draft guidance on radiological source term during an RIA.
- b. Industry representatives should be prepared to address the following:
 - i. Transient-induced fission gas release (from the pellet) and radiological source term.
 - ii. Available analytical methods and implementation of burnup and corrosion dependent criteria.

Electric Power Research Institute

Project No. 669

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9/28/06