

October 26, 2006

Mr. Michael R. Kansler
President
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 2 - REQUEST FOR
ADDITIONAL INFORMATION REGARDING STEAM GENERATOR TUBE
EXAMINATION RESULTS (TAC NO. MD2783)

Dear Mr. Kansler:

On June 14, 2006, Entergy Nuclear Operations, Inc. (Entergy), submitted the results of the Indian Point Nuclear Generating Unit Unit 2 steam generator tube examinations conducted during the spring 2006 refueling outage, as required by Technical Specification 5.5.7.f.2.

The Nuclear Regulatory Commission staff is reviewing the submittal and has determined that additional information is needed to complete its review. The specific questions are found in the enclosed request for additional information (RAI). On October 12, 2006, the Entergy staff indicated that a response to the RAI would be provided within 60 days of the date of this letter.

Please contact me at (301) 415-2901 if you have any questions on this issue.

Sincerely,

/RA/

John P. Boska, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosure:
RAI

cc w/encl: See next page

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OFFICIAL RECORD COPY

Indian Point Nuclear Generating Unit No. 2

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Indian Point Nuclear Generating Unit No. 2

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REQUEST FOR ADDITIONAL INFORMATION
REGARDING STEAM GENERATOR TUBE EXAMINATIONS
ENTERGY NUCLEAR OPERATIONS, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

On June 14, 2006, Agencywide Documents Access and Management System accession number ML061780303, Entergy Nuclear Operations, Inc. (Entergy), submitted the results of the Indian Point Nuclear Generating Unit 2 steam generator tube examinations conducted during the spring 2006 refueling outage, as required by Technical Specification 5.5.7.f.2. The Nuclear Regulatory Commission staff is reviewing the submittal and has the following questions:

1. Section 2.a.7 of your report dated June 14, 2006, indicated that the inspection program for refueling outage (RFO) 17 consisted of special interest examinations of "abnormal" indications. Please discuss the results of any special interest examinations on these indications. In addition, discuss what constitutes an "abnormal" indication.
2. Section 2.b, Secondary Side Examinations, states that the top support plate in steam generators (SGs) 23 and 24 were visually inspected. Please discuss the results of these visual inspections. If any degradation was found, please discuss the basis for the inspection scope.
3. It is stated in Section 3.0 that the largest bobbin indication (28-percent) located at an anti-vibration bar (AVB) was confirmed to be single-sided wear by +Point™ examination. Please discuss whether all AVB wear scars were inspected with a rotating probe to confirm that they were also single-sided wear. If any of these indications were not single-sided wear (e.g., row 36 column 44 in SG 22), please discuss the method used to size these indications. If a "qualified" sizing technique does not exist for wear indications that are not "single-sided," please discuss your basis for leaving them in service (if any were left in service).
4. Please discuss whether the low levels of primary-to-secondary leakage observed during prior cycles is still present.
5. For the tubes identified with permeability variations, discuss whether these locations were inspected with a rotating probe (e.g., magnetically biased +Point™ probe) to confirm the absence of degradation at these locations. Please discuss whether the size of these permeability variations limit the ability to reliably inspect these locations. If so, discuss your long-term strategy for addressing these indications.
6. In your February 13, 2006 (ML060530378), letter you indicated that you would be performing primary side visual inspections and that you would be inspecting the top of the flow distribution baffle in all 4 SGs. Please discuss the results of these inspections.

Enclosure