

December 18, 2006

Mr. Jeffrey S. Forbes  
Site Vice President  
Arkansas Nuclear One  
Entergy Operations, Inc.  
1448 S. R. 333  
Russellville, AR 72801

SUBJECT: ARKANSAS NUCLEAR ONE, UNIT NO. 2 - ISSUANCE OF AMENDMENT  
FOR MODIFICATIONS TO TECHNICAL SPECIFICATION SURVEILLANCE  
REQUIREMENT 4.6.1.1 REGARDING CONTAINMENT ISOLATION VALVE  
POSITION VERIFICATION (TAC NO. MD0059)

Dear Mr. Forbes:

The Commission has issued the enclosed Amendment No. 269 to Facility Operating License No. NPF-6 for Arkansas Nuclear One, Unit No. 2 (ANO-2). The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated February 14, 2006.

Specifically, the proposed amendment would eliminate the requirement to verify containment isolation valves that are maintained locked, sealed, or otherwise secured closed from the monthly position verification. A new surveillance requirement, (SR) 4.6.1.1.d, would also be added to replace the existing note and would reflect the SR for similar devices located inside containment. In addition, a new note would be included that will allow verification by use of administrative means of the valves and blind flanges that are located in high-radiation areas. In this regard, the amendment adopts TS Task Force (TSTF) Improved Standard TS Change Traveler No. 45 (TSTF-45-A), "Exempt Verification of Containment Isolation Valves that are Not Locked, Sealed, or Otherwise Secured."

J. Forbes

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A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

***/RA/***

Farideh E. Saba, Project Manager  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-368

Enclosures: 1. Amendment No. 269 to NPF-6  
2. Safety Evaluation

cc w/encls: See next page

J. Forbes

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Sincerely,  
**/RA/**  
Farideh E. Saba, Project Manager  
Plant Licensing Branch IV  
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ENTERGY OPERATIONS, INC.

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 269  
Renewed License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated February 14, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-6 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 269, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

David Terao, Chief  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Renewed  
Facility Operating License  
No. NPF-6 and the  
Technical Specifications

Date of Issuance: December 18, 2006

ATTACHMENT TO LICENSE AMENDMENT NO. 269  
RENEWED FACILITY OPERATING LICENSE NO. NPF-6  
DOCKET NO. 50-368

Replace the following page of the License with the attached revised page. The revised page is identified by an amendment number and contains a marginal line indicating the area of change.

Remove

3

Insert

3

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by an amendment number and contains marginal lines indicating the areas of change.

Remove

3/4 6-1

Insert

3/4 6-1

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 269 TO

RENEWED FACILITY OPERATING LICENSE NO. NPF-6

ENTERGY OPERATIONS, INC.

ARKANSAS NUCLEAR ONE, UNIT NO. 2

DOCKET NO. 50-368

1.0 INTRODUCTION

By application dated February 14, 2006 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML060530391), Entergy Operations, Inc. (the licensee, Entergy), requested changes to the Technical Specifications (TSs) for Arkansas Nuclear One, Unit No. 2 (ANO-2). The staff's original proposed no significant hazards consideration determination was published in the *Federal Register* on April 11, 2006 (71 FR 18373).

The proposed changes will revise the ANO-2 TS surveillance requirement (SR) 4.6.1.1.a. to eliminate the requirement to verify containment isolation valves that are maintained locked, sealed, or otherwise secured closed from the monthly position verification. A new surveillance requirement, (SR) 4.6.1.1.d, would also be added to replace the existing "\*note" and would reflect the SR for similar devices located inside containment. In addition, a new note would be included that will allow verification by use of administrative means of the valves and blind flanges that are located in high-radiation areas. In these respects, the amendment adopts TS Task Force (TSTF) Improved Standard TS Change Traveler No. 45 (TSTF-45-A), "Exempt Verification of Containment Isolation Valves that are Not Locked, Sealed, or Otherwise Secured."

2.0 REGULATORY EVALUATION

On July 26, 1999, the U.S. Nuclear Regulatory Commission (NRC) approved TSTF Improved "Standard TS Change Traveler No. 45 (TSTF-45-A)," which revised the Standard Technical Specifications. NUREG-1432, "Standard Technical Specifications Combustion Engineering Plants," Revision 3.1, SR 3.6.3.3 and SR 3.6.3.4 reflected in TSTF-45-A the elimination from the TS of the surveillance requirements for containment isolation valves that were verified to be in the closed position upon locking, sealing or securing. NRC Generic Letter 91-08, "Removal of Component Lists from Technical Specifications," provided the guidance for the administrative requirements associated with opening manual containment isolation valves intermittently.

### 3.0 TECHNICAL EVALUATION

#### 3.1 Proposed Change

The proposed change will specifically exclude verification of normally locked, sealed, or otherwise secured closed valves, blind flanges, and the deactivated automatic valves. SR 4.6.1.1.d will be modified to replace the existing “\*note” since the note contains a requirement to perform the surveillance for physically secured valves and flanges that are closed and located inside containment. The change also allows administrative means for performing valve verification when the valve is located in a high-radiation area. The proposed change reflects the wording that is included in NUREG-1432 with consideration for the difference in TS format.

#### 3.2 Evaluation of TS Change

The proposed TS change is consistent with the NRC-approved TSTF-45-A, which was approved by the NRC for use by licensees on July 26, 1999. This TSTF was approved by the NRC staff for the use of licensees in proposing changes to plant-specific TSs based on NUREG-1432. TSTF-45-A was revised to exclude locked, sealed, or otherwise secured closed valves that are located outside containment from the monthly verification.

TSTF-45-A stated that the proposed change was consistent with the valve position verification requirement for valves that have a function during an accident in other system TSs. Consistent with the logic of the TSTF, ANO-2 TS SR 4.5.2.b requires that each emergency core cooling system subsystem be demonstrated operable, in part, by performance of a valve alignment of each valve that is in the flow path that is not locked, sealed, or otherwise secured valves. The emergency feedwater system and the service water system are also demonstrated to be operable, in part, by verifying that each valve in the flow path of the system is not locked, sealed, or otherwise secured in position, and is in its correct position (SR 4.7.1.2.a.1 and SR 4.7.3.1.a, respectively). The addition of the note that allows an alternate (administrative) means of verifying valves that are located in high-radiation areas is in accordance with the requirements of ANO-2 TS 6.7, “High Radiation Area.”

The NRC staff finds that the proposed change to SR 4.6.1.1 is acceptable since it is consistent with other ANO-2 TS SRs and the NRC-approved TSTF-45-A.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arkansas State official was notified of the proposed issuance of the amendment. The State official had no comments.



## 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes a surveillance requirement. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The NRC has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding published April 11, 2006 (71 FR 18373) . Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 6.0 CONCLUSION

The NRC staff finds that the licensee's proposed change to SR 4.6.1.1 reflects the wording in NUREG-1432 with consideration of the different TS format and is, therefore, acceptable.

The NRC has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: B. Lee

Date: December 18, 2006

Arkansas Nuclear One

cc:

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