

December 11, 2006

Mr. D. E. Grissette
Vice President
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, AL 35201-1295

SUBJECT: EVALUATION OF VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2 (VOGTLE 1 AND 2), RESPONSE TO GENERIC LETTER 2003-01, "CONTROL ROOM HABITABILITY" (TAC NOS. MB9869 AND MB9870)

Dear Mr. Grissette:

The Nuclear Regulatory Commission (NRC) acknowledges the receipt of your responses to Generic Letter (GL) 2003-01, "Control Room Habitability," dated August 4, 2003 (Agencywide Documents Access and Management System [ADAMS] Accession No. ML032190245), June 16, 2004 (ADAMS Accession No. ML041700064), and October 20, 2006 (ADAMS Accession No. ML062960230). This letter provides a status of your responses and describes any additional information that may be necessary to consider your response to GL 2003-01 complete.

The NRC staff requested in the GL that you confirm that your control rooms meet their design bases (e.g. General Design Criteria (GDC) 1, 3, 4, 5, and 19, draft GDC, or principal design criteria), with special attention to: (1) Determination of the most-limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in your design bases for meeting control room operator dose limits from accidents (GL 2003-01, Item 1a); (2) Determination that the most limiting unfiltered inleakage is incorporated into your hazardous chemical assessments (GL 2003-01, Item 1b); and, (3) Determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 2003-01, Item 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability (CRH), and plans to retire them (GL 2003-01, Item 2).

You reported the results of ASTM E741(American Society for Testing and Materials), "Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution," tracer gas tests for the Vogtle 1 and 2 control rooms, which are separate for each unit but are located in a common control room envelope (CRE) and pressurized for accident mitigation. You determined that the maximum tested value for inleakage into the CRE was 0 standard cubic feet per minute (scfm), which is less than the value of 5 scfm assumed in the design basis radiological dose analyses for CRH.

You indicated that isolation of the CRE is not assumed for hazardous chemical analyses; therefore, the maximum inleakage value is the normal ventilation intake value of about

1800 cfm (cubic feet per minute), which is less than the value of 3000 cfm assumed in your design basis hazardous chemical analyses. You also provided information that adequately supported a conclusion that reactor control capability is maintained from either the control room or the alternate shutdown panel in the event of smoke.

The NRC staff further requested in the GL that you assess your technical specifications (TSs) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design-basis analysis for CRH. This request resulted from the demonstrated inadequacy of a differential pressure measurement to alone provide such verification (GL 2003-01, Item 1c). As permitted by the GL, you provided a schedule for revising the surveillance requirement in the TSs to reference an acceptable surveillance methodology. In your June 16, 2004, response, you indicated that you would submit a License Amendment Request (LAR) to adopt TS surveillance requirements (SRs) that verify CRH per Technical Specification Task Force (TSTF) Change Traveler TSTF-448. In your October 20, 2006, letter, you committed to submit the LAR 1 year after NRC approval of TSTF-448.

The information you provided also supported the fact that there are no compensatory measures needed to be in place to demonstrate CRH, and the conclusion that you are committed to meet the GDC regarding CRH.

Your commitment to submit an LAR based on TSTF-448, following our formal review and approval, is acceptable for purposes of closing out your response to GL 2003-01. The staff will monitor submission of the LAR and interact with you as necessary during the amendment process.

If you have any questions regarding this correspondence, please contact me.

Sincerely,

/RA/

Robert Martin, Sr. Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-424 and 50-425

cc: See next page

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Vogtle Electric Generating Plant, Units 1 & 2

cc:

Mr. D. E. Grissette, Vice President
Southern Nuclear Operating Company, Inc.
P.O. Box 1295
Birmingham, AL 35201-1295

Mr. N. J. Stringfellow
Manager, Licensing
Southern Nuclear Operating Company, Inc.
P.O. Box 1295
Birmingham, AL 35201-1295

Mr. T. E. Tynan, General Manager
Vogtle Electric Generating Plant
Southern Nuclear Operating Company, Inc.
7821 River Road
Waynesboro, GA 30830

Mr. Jeffrey T. Gasser
Executive Vice President
Southern Nuclear Operating Company, Inc.
P.O. Box 1295
Birmingham, AL 35201-1295

Mr. Steven M. Jackson
Senior Engineer - Power Supply
Municipal Electric Authority of Georgia
1470 Riveredge Parkway, NW
Atlanta, GA 30328-4684

Mr. Reece McAlister
Executive Secretary
Georgia Public Service Commission
244 Washington St., SW
Atlanta, GA 30334

Mr. Harold Reheis, Director
Department of Natural Resources
205 Butler Street, SE, Suite 1252
Atlanta, GA 30334

Attorney General
Law Department
132 Judicial Building
Atlanta, GA 30334

Mr. Laurence Bergen
Oglethorpe Power Corporation
2100 East Exchange Place
P.O. Box 1349
Tucker, GA 30085-1349

Arthur H. Dombey, Esquire
Troutman Sanders
Nations Bank Plaza
600 Peachtree Street, NE
Suite 5200
Atlanta, GA 30308-2216

Resident Inspector
Vogtle Plant
8805 River Road
Waynesboro, GA 30830

Office of the County Commissioner
Burke County Commission
Waynesboro, GA 30830