

October 24, 2006

Mr. Jeffrey Archie
Vice President, Nuclear Operations
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
Post Office Box 88
Jenkinsville, S.C. 29065

SUBJECT: VIRGIL C. SUMMER NUCLEAR STATION — NRC RECEIPT OF RESPONSE
TO GENERIC LETTER 2003-01 "CONTROL ROOM HABITABILITY"
(TAC NO. MB9860)

Dear Mr. Archie:

The Nuclear Regulatory Commission (NRC) acknowledges the receipt of your responses to Generic Letter 2003-01 "Control Room Habitability," dated August 8, 2003 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML032240028); January 31, 2005 (ADAMS Accession No. ML050340205) and November 18, 2005 (ADAMS Accession No. ML053260454). This letter provides a status of your response and describes any additional information that may be necessary to consider your response to GL 2003-01 complete.

The GL requested that you confirm that your control rooms meet their design bases (e.g. General Design Criterion (GDC) 1, 3, 4, 5, & 19, draft GDC, or principal design criteria), with special attention to: (1) determination of the most limiting unfiltered and/or filtered leakage into the control room and comparison to values used in your design bases for meeting control room operator dose limits from accidents (GL 2003-01, Item 1a); (2) determination that the most limiting unfiltered leakage is incorporated into your hazardous chemical assessments (GL 2003-01, Item 1b); and, (3) determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 2003-01, Item 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability (CRH), and plans to retire them (GL 2003-01, Item 2).

You reported the results of ASTM E741 [American Society for Testing Materials, Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution] tracer gas tests for the pressurized Virgil C. Summer Nuclear Station control room.

You determined that the tested value for unfiltered leakage into the control room envelope (CRE) was 41 standard cubic feet per minute (scfm), which is less than the value of 55 scfm assumed in the design basis radiological dose analyses for CRH.

You indicated that your current design basis hazardous chemical assessment assumes 55 scfm leakage and has inherent margin to accommodate potential increases in CRE leakage; therefore, the most limiting unfiltered leakage into your CRE (41 scfm) is incorporated into

your assessment. You also indicated that reactor control capability is maintained from either the control room or the alternate Control Room Evacuation Panel rooms in the event of smoke.

The GL further requested that you assess your Technical Specifications (TS) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design basis analysis for CRH and in light of the demonstrated inadequacy of a differential pressure measurement alone to provide such verification (GL 2003-01, Item 1c). As permitted by the GL, you provided a schedule for revising the surveillance requirement in the TS to reference an acceptable surveillance methodology. In your November 18, 2005 response, you committed to submit a license request based on Technical Specification Task Force (TSTF)-448 within 12 months following the NRC approval of TSTF-448.

The information you provided also supported the fact that there are no compensatory measures needed to be in place to demonstrate CRH. The information you provided also supported the conclusion that you are committed to meet the GDC regarding CRH.

Your commitment to submit a license amendment request (LAR) based on TSTF-448, following our formal review and approval, is acceptable for purposes of closing out your response to GL 2003-01. The staff will monitor submission of the LAR and interact with you as necessary during the amendment process.

If you have any questions regarding this correspondence, please contact me.

Sincerely,

/RA/

Robert E. Martin, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-395

cc: See next page

your assessment. You also indicated that reactor control capability is maintained from either the control room or the alternate Control Room Evacuation Panel rooms in the event of smoke.

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Robert E. Martin, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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