

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components.
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I:

Part 20, Section 30.34 of Part 30; Section 40.41 of Part 40; Section 50.54 and 50.59 of Part 50; and Section 70.32 of Part 70. The license is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect and is also subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 1850 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, which is attached hereto, as revised through Amendment No. 191 is hereby incorporated into this license. Nine Mile Point Nuclear Station, LLC shall operate the facility in accordance with the Technical Specifications.

2.C.3 - Deleted by Amdt. # 104, dated April 10, 1989

- (4) The licensee shall submit an application for license amendment, including supporting analyses and evaluations by December 18, 1998. This amendment application shall contain the proposed methods for compliance with GDC 19 dose guidelines under accident conditions based upon system design and without reliance upon the use of potassium iodide.

Amendment No. ~~1, 5, 16, 25, 31, 36,~~
~~104, 161, 172~~
Correction Letter dated 3/30/00,
191

TABLE 3.6.11-1

ACCIDENT MONITORING INSTRUMENTATION

<u>Parameters</u>	<u>Total Number of Channels</u>	<u>Minimum Number of Operable Sensors or Channels</u>	<u>Action (See Table 3.6.11-2)</u>
1) Relief Valve Position Indication	2/Valve	1/Valve*	1
2) Safety Valve Position Indication	2/Valve	1/Valve*	1
3) Reactor Vessel Water Level	2	1*	2
4) Drywell Pressure Monitor	2	1	4
5) Suppression Chamber Water Level	2	1*	4
6) Deleted			
7) Containment High Range Radiation Monitor	2	1	3
8) Suppression Chamber Water Temperature	2	1	2

* A channel may be placed in an inoperable status for up to 6 hours for required surveillance provided at least one Operable channel is monitoring that Parameter.

TABLE 4.6.11

ACCIDENT MONITORING INSTRUMENTATION

SURVEILLANCE REQUIREMENT

<u>Parameter</u>	<u>Instrument Channel Test</u>	<u>Instrument Channel Calibration</u>
(1) Relief valve position indicator (Primary - Acoustic)	Once per quarter	Once during each major refueling outage
Relief valve position indicator (Backup - Thermocouple)	Once per quarter	Once during each major refueling outage
(2) Safety valve position indicator (Primary - Acoustic)	Once per quarter	Once during each major refueling outage
Safety valve position indicator (Backup - Thermocouple)	Once per quarter	Once during each major refueling outage
(3) Reactor vessel water level	Once per quarter	Once during each major refueling outage
(4) Drywell Pressure Monitor	Once per month	Once during each major refueling outage
(5) Suppression Chamber Water Level Monitor	Once per quarter	Once during each major refueling outage
(6) Deleted		
(7) Containment High Range Radiation Monitor	Once per month	Once during each major refueling outage
(8) Suppression Chamber Water Temperature	Once per month	Once during each major refueling outage