	CONST	ELLATION ENERG	iΥ	NO.:	06-1-1	REV: 0
	R. E. GINNA M	UCLEAR POWER	PLANT	TITLE:	SCENARIO #1	
	EXAMIN	ATION SCENAR	10	DATE:	03-29-06	PAGE: 1 of 29
	Written by: <u>Ken I</u> Sr. L Technical Review:		es		Date: <u>3/23/06</u> <u>5/18/06</u>	
	Time validated <u>10</u> Date of exam:	<u>0</u> minutes	By: <u>Roy Gill</u> Shift N	<u>low</u> Manager	Date: <u>5/18/06</u>	
	Examinees			-	Evaluators	
	······					
/	Final review	License Ins	tructor	4.17	Date:	
	Approved for use	Director Op or Designee	erations Trai	ining	Date:	
					ATTRIBUTE	±.
					Total Malfunctions	0

Total Malfunctions	8
Malf after EOP entry	2
Abnormal Events	4
Major Transients	2
EOP's beyond SCRAM	1
ECA's, FR's	0
Critical Tasks	3
T.S. Exercised Yes/No	Yes

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#### 1. <u>SCENARIO OVERVIEW</u>

- 1.1 The plant is at approximately 2% power in the process of starting up following a trip. The core is Xenon free. The startup procedure O-1.2 is complete up to the point of swapping to condenser steam dump, starting a MFW pump and transitioning from AFW to MFW.
- 1.2 Per the startup procedure, the operators will start a MFW pump and place MFW in service. Once MFW is in service AFW will be secured.
- 1.3 With MFW in service, the RO will begin increasing power in preparation for rolling the main turbine.
- 1.4 Spray Valve Controller PCV-431A fails resulting in 431A going full open. Manual control is available and the operators should take manual control of PCV-431A and close the valve.
- 1.5 The "A" MFW Regulating Valve fails open, manual control is available and the BOP should take manual control and control "A" S/G level manually.
- 1.6 Bus 16 develops a fault and the normal feed breaker opens deenergizing the bus. The operators should respond per AP-ELEC-14/16 to restore equipment and stabilize the plant (Tech Spec 3.8.9).
- 1.7 "B" RCP develops high vibration. Per the AR procedures the operators should manually trip the reactor and go to E-0 and ES-0.1.
- 1.8 Shortly after the Hi Vibration on the "B" RCP the Seal Package fails resulting in a SB LOCA in excess of the charging capacity.
- 1.9 SI fails to auto actuate require manual actuation.
- 1.10 Following SI Reset in E-1 offsite power is lost. The "A" D/G did not auto-start during the SI (and will not now). The operators must take manual action to start the "A" D/G and manually load safeguards equipment.

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#### 2. <u>SCENARIO OBJECTIVES</u>

- 2.1 Perform operator actions to establish MFW and secure AFW.
- 2.2 Manipulate Reactivity Control Systems to increase Rx power during a startup.
- 2.3 Respond to a failed spray valve controller. Take manual action to override the failed automatic control.
- 2.4 Respond to a Feed Reg Valve failure and manually control S/G level.
- 2.5 Respond to a loss of a safeguards bus. Take action to restore equipment and stabilize plant parameters.
- 2.6 Respond to a RCP High Vibration condition requiring a manual Rx Trip.
- 2.7 Respond to a SBLOCA using the EOP. Initiate SI to compensate for a failed auto signal.
- 2.8 Restore electrical power to one train of AC emergency equipment and manually start SI equipment needed to respond to the SBLOCA.

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3. CRITICAL TASKS (Cts)

CT #1

E-0--D

Task: Manually actuate at least one train of SIS actuated safeguards before:

- Transition to any E-1, 2 or 3 series procedures.
  Completion of ES-0.1 Step 8a

<u>Cues</u>: - Indication SI is required.

- Prz Pressure ≤ 1750 psig
  Exceeding SI actuation criteria of ES-0.1 foldout
- No indication or annunciation of SI Actuation

<u>Indication</u>: Manipulation of control to actuate at least one train of SI. <u>Feedback</u>: Indication that one train of SI is actuated.

CT #2

E-0--C

- <u>Task</u>: Energize at least one AC emergency bus prior to placing safeguard switches in pull to lock in ECA-0.0.
- <u>Cues</u>: Indications that all AC emergency buses are deenergized.
  - D/G Status
  - Bus Voltages
     Normal feed br
  - Normal feed breakers open

- No indication or annunciation of SI Actuation

<u>Indicator</u>: Manipulation of controls to restore power to at least one AC emergency bus.

Feedback: Indications that at least one AC emergency bus is energized.

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3. CRITICAL TASKS (Cts)

CT #3

E-0--I

Task: Establish flow from at least two SI pumps prior to proceeding in the procedure following the loss of power.

<u>Cue</u>: Indication SI pumps required

SI actuated
RCS below SI pump and shutoff pressure
AND

• Less than 2 SI pumps injecting into the core.

<u>Indicator</u>: Manipulation of control to establish flow to core from 2 SI pumps.

Feedback: Indication that two SI pumps are injecting.

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# 4. INSTRUCTOR ACTIONS

+ · <u></u>			
Problem Time		Actions	<u>Notes</u>
	- I F 1 - P	te: Setup saved as IC-171) nitialize Simulator to IC-22 Ex at 18%, Tur at 1800 RPM Decrease power to 2 % Trip Turbine lace Hold Tag on B FW Pump and il pumps	
	Ins	ert Malfunctions	
	•	MALF SIS02A,B Option 0 Manual available	SI fails to actuate (Both Trains)
	•	MALF GEN08 Option 1 A D/G 0 sec TD	"A" D/G Autostart failure
	•	MALF PZR1A 100% (open) 30 sec Ramp Trigger 1	PCV 431A control failure (Manual available)
	•	MALF FDW7A 100% (open) 300 sec Ramp Trigger 2	"A" MFRV Auto control fails (open). Manual Control available
	•	MALF EDS4B Trigger 3	Bus 16 Fault
	•	MALF RCS15B 25 mils 300 sec Ramp Trigger 4	"B" RCP Shaft Vibration
	•	MALF RCS15D 10 mils 300 sec Ramp 60 sec Time Delay Trigger 4	"B" RCP Seismic (Casing) Vibration
	•	MALF RCS12B 300 gpm 600 sec Ramp Trigger 5	"B" RCP #1 Seal Failure
	•	MALF RCS13B 300 gpm 600 sec Ramp Trigger 5	"B" RCP #2 Seal Failure

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	<ul> <li>MALF RCS14B</li> <li>300 gpm</li> <li>0 Ramp</li> <li>Trigger 5</li> </ul>		"B" RCP #3 Seal Failure
	• MALF EDS6 2 (Fast) Trigger 6		Station Blackout
	<ul> <li>OVR-FDW08A OFF</li> </ul>		Turn off green light for MFW Pump B Oil Pump
	• OVR-FDW20A OFF		Turn off green light for MFW Pump B Brkr
	• OVR-FDW20C OFF		Turn off green light for MFW Pump B Brkr

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# 4. INSTRUCTOR ACTIONS

<u>Problem Time</u>		Actions	Notes
Note: Malfunct of the Lead Ex		mes are approximate: Malfunction are ini	tiated at the direction
0 min	4.2	MFW Pump Start	Align SW to A MFW pumps requested. SW not modeled
20 min	4.3	Power escalation	Action as requested by crew
30 min	4.4	PCV-431A Failure Insert Trigger 1	
37 min	4.5	"A" MFRV Fails Open Insert Trigger 2	Report ADFAC Alarm as "A" MFRV output card failure when requested to check printer.
50 min	4.6	Bus 16 Fault Insert Trigger 3	Report Bus 16 is faulted after requested to investigate.
62 min	4.7	"B" RCP Hi Vibration Insert Trigger 4	
65 min	4.8	°B″ RCP Seal Failure Insert Trigger 5	
80 min	4.9	Station Blackout Insert Trigger 6	

Terminate Scenario as directed by the lead examiner

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### 5. <u>TURNOVER INFORMATION</u>

5.1 The plant is at 2-3% power during a startup following a trip five days ago. The core is Xenon free. RCS boron concentration is 1335 ppm. Procedure O-1.2 is complete up to step 6.4.6(20). Attachment 5 Main Feed Water Pump "A" is complete through step 1.8.2.

### 5.2 Equipment Out of Service

"B" MFW pump is out of service due to a gear box failure.

## 5.3 <u>Work in Progress</u>

"B" MFW pump gear box repair (additional 36 hours to repair).

### 5.4 <u>Planned Work</u>

Plant startup from approximately 2% power.

### 5.5 <u>Significant Events</u>

"B" MFW pump gear box failure resulted in a Rx trip five days ago.

# 5.6 <u>Remarks</u>

Startup plant per 0-1.2, hold at 50% power until work is complete on the "B" MFW pump.

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## 6. EVALUATION

# Event · 1

Event: <u>1</u>		
Event Title:	<u>Main Feedwater Pu</u>	mp Start/Transition to > 5% power
EVENT TASKS:	041-002-01-01	Shift Modes of Steam Dump Operation
	041-002-01-02	Direct Steam Dump Operations
	059-006-01-01	Startup the MFW System
	059-012-01-02	Direct Operations of MFW System
	061-007-01-01	Shutdown AFW System

061-003-01-02 Direct Operations of AFW and SAFW System

# Expected Response/Behavior

CUES:

Procedure Direction in 0-1.2

# Response:

Sponde.		RATING	<u>N/A</u>
SRO	Conduct Shift Briefing for Power Increase and Syncing Generator		
BOP/SRO	Place Condenser Steam Dump in Service		
	• Verify Rx Trip from Turbine Trip Blocked (P-9) Light lit		
	• Verify Air Ejector is in service		
	• Verify Vacuum >27.5 in Hg		
	<ul> <li>Lower HCV-484 to 1005 psi Verify SD valves respond</li> </ul>		
	ullet Setup SG ARV for Auto Operation at 1050 psig		

# Place Blow down key switches to normal

BOP

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Response:			RATING	<u>N/A</u>
BOP/SRO	Start the "A" MFW pumj Pump A	o per Attachment MFW		
	• Place Blowdown key s	switches to Normal		
	• Start the "A" MFW p	qmp		
вор	Open MOV-3977			
SRO	Dispatch AO to close V	<i>I</i> -39977A		
SRO	Dispatch AO to close v	<i>r</i> alves		
	• AOV 4262			
	• AOV 4263			
	• V-4060			
BOP/SRO	Verify operation of MM	W Reg and Bypass Valves		
BOP/SRO	Place MFW Reg and Bypa	ass Valves in Auto		
	(Note at this point th increase power to ~5%)	ne RO should begin to		

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OP/SRO	Secure AFW		
	• Close MOV 4007/4008		
	• Stop AFW Pumps		
	• Notify AO to restore E	lowdowns	
	• Close cross-tie 4000A/	Έ	
	• Open Discharges 3996/4	007/4008	
	• Open TDAFW Control Val	ves 4297/4298	
	• Close Bypass Valves 44	80/4481	
	• AFW Defeat Switches to	o Normal	
	• Verify Recirc Valves C	llogod	

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6. <u>EVALUATION</u> Event: 2	<u>1</u>					
Event Title:	Increase Power t	o Roll Turbi	ne		,	
EVENT TASKS:	001-023-01-01	<u>Operate th</u>	e Control Rods	at Power		
	001-001-01-02	<u>Direct Ope</u>	rations of the	<u>Rod Control</u>	System	
	N			1.0.0.0.0.0.0.0.0.0.0.0.0.0.0.0.0.0.0.0		
Expected Resp	onse/Behavior					
CUES:						
	water is in servi from Procedure O					
Response:					<u>RATING</u>	N
RO/SRO	Withdraw Con	trol Rod to :	increase power			
	<ul> <li>Stop even indication Attachment</li> </ul>	ons (Low Powe	to verify powe or Operations	er		

- RO/SRO
- Verify RCS Pressure and Temperature are within startup limits prior to exceeding 5% power

•

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6. EVALUATION	ī			
Event: 3	-			
Event Title:	PCV-431A fails open			
EVENT TASKS:		l to a Przr Press Control	Malf	
		Response to a Przr Press		
	<u></u>			
	<u> </u>			
Expected Resp	onse/Behavior			
CUES:				
	essure Alarm ress Decreasing			
Response:				
Response:			RATING	N
Response:	NOTE: Upon determinin	g PCV-431A has		N
Response:	failed, the operator PCV-431A in manual an	may take action to place d close valve without		N
Response:	failed, the operator	may take action to place d close valve without		N
Response: RO	failed, the operator PCV-431A in manual an	may take action to place d close valve without er A-503.1		<u>N</u>
-	failed, the operator PCV-431A in manual an procedural guidance p	may take action to place d close valve without er A-503.1		<u>N</u>
-	failed, the operator PCV-431A in manual an procedural guidance p	may take action to place d close valve without er A-503.1		<u>N</u>
RO	failed, the operator PCV-431A in manual an procedural guidance p Determine Przr Pressu	may take action to place d close valve without er A-503.1		<u>N</u>
RO	failed, the operator PCV-431A in manual an procedural guidance p Determine Przr Pressu Enter AP-PRZR.1 Check all Channels Pr	may take action to place d close valve without er A-503.1 re decreasing		<u>N</u>
RO SRO	failed, the operator PCV-431A in manual an procedural guidance p Determine Przr Pressu Enter AP-PRZR.1	may take action to place d close valve without er A-503.1 re decreasing		<u>N</u>
RO SRO	failed, the operator PCV-431A in manual an procedural guidance p Determine Przr Pressu Enter AP-PRZR.1 Check all Channels Pr	may take action to place d close valve without er A-503.1 re decreasing		<u>N</u>
RO SRO	failed, the operator PCV-431A in manual an procedural guidance p Determine Przr Pressu Enter AP-PRZR.1 Check all Channels Pr	may take action to place d close valve without er A-503.1 re decreasing zr Pressure equal and		<u>N</u>
RO SRO RO/SRO	failed, the operator of PCV-431A in manual an procedural guidance p Determine Przr Pressu Enter AP-PRZR.1 Check all Channels Pr tracking	may take action to place d close valve without er A-503.1 re decreasing zr Pressure equal and		<u>N</u>
RO SRO RO/SRO	failed, the operator of PCV-431A in manual an procedural guidance p Determine Przr Pressu Enter AP-PRZR.1 Check all Channels Pr tracking	may take action to place d close valve without er A-503.1 re decreasing zr Pressure equal and		<u>N</u>
RO SRO RO/SRO RO/SRO	failed, the operator of PCV-431A in manual an procedural guidance p Determine Przr Pressu Enter AP-PRZR.1 Check all Channels Pr tracking Check Rx power stable	may take action to place d close valve without er A-503.1 re decreasing zr Pressure equal and		<u></u>

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	Response:			RATING	<u>n/a</u>
	RO/SRO	Check Spray Valves Determine AOV-431A open Place 431A Controller a	ı ıt 0% demand in manual		
	RO/SRO	Check 431K Master Press	sure Controller		
	RO/SRO	Check PORV - closed			
	RO/SRO	Check Safeties - closed	1		
	RO/SRO	Check Aux Spray - close	ed		
	RO/SRO	Check Pressure trending step 16)	g to 2235 psi (go to		·
	RO/SRO	Check PRT normal			
	RO/SRO	Check Przr Pressure Con 431A in manual)	trol in Auto (leave		
	All	Check Annunciator Statu	IS		<u></u>
	SRO	Notification of Supervi	sion		

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# 6. EVALUATION

Event: <u>4</u>

Event Title: <u>"A" MFR Valve Fails Open</u>

EVENT TASKS: 059-002-04-01 Respond to an Excessive Feedwater increase

<u>059-002-04-02</u>

Direct Response to an Excessive Feedwater increase

## Expected Response/Behavior

CUES:

Alarm G-22 ADFACS System Trouble Excessive FW to "A" SG SG "A" Level rapidly increasing

### Response:

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Response:			RATING	<u>N/7</u>
BOP/SRO	Verify adequate MFW flow			
RO/SRO/ BOP	Verify Stable Plant Condit: • Tavg • Przr Press • Przr Level • MFW Valves • Rod Insertion Limits	ions		
BOP/SRO	Check MFW System			
BOP/SRO	Check Condensate System • Bypass Valve • Hotwell Level • Condensate Pumps (2 runn • Trim Valves	ing)		
RO/BOP/ SRO	Check Controls in Auto • PCV-431K • Spray Valve Controllers • Przr Heaters • Charging (Note Check EHC, MFW Reg V Dump but do not realign)	alves, Rods and Steam		
BOP/SRO	Check AFW (Not in service)			
RO/SRO/ BOP	Check Annunciator Status			
RO/SRO	Check if Przr mixing requi	red		
SRO	Notify Higher Supervision			

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. <u>EVALUATIO</u> Event: 5					
Event Title:	Bus 16 Normal Fe	eeder Break	er Trips		
EVENT TASKS:	062-034-04-01		to the loss of an indi	vidual safeguard	s Bus
	062-034-04-02		<u>esponse to a loss of a</u>		
			. <u>.</u>		
Expected Resp	onse/Behavior				
CUES:					
L-7 Bus 1 Bus 16 Vo	Safeguard Bus No 6 Undervoltage ltage Zero pplied Loads Trip		Breaker Trip		
				RATING	<u>n/</u>
Response:				RATING	<u>n/</u> .
			er AP-ELEC.14/16	RATING	<u>n/</u>
Response:		mptoms Ent	er AP-ELEC.14/16	<u>RATING</u>	<u>N/</u>
Response: SRO	Recognize sy Monitor/Cont	rmptoms Ent crol Tavg D/G Running	(running but will not		<u>N/</u>
Response: SRO RO/SRO	Recognize sy Monitor/Cont Verify "B" I tie in due t	rmptoms Ent crol Tavg O/G Running to bus faul	(running but will not		<u>N/</u>
Response: SRO RO/SRO BOP	Recognize sy Monitor/Cont Verify "B" I tie in due t Verify one t	omptoms Ent crol Tavg O/G Running to bus faul crain of AC	(running but will not t) Buses energized and		<u>N</u> Z.

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Response:			RATING	<u>N/2</u>
BOP/SRO	Verify MFW Controlling i control "A" MFRV)	n Auto (manually		
RO/BOP/ SRO	Check Buses Energized/Ma • CCW Pump A • Charging Pump A • Przr Proportional Heat • CNMT Recirc Fans A and • Boric Acid Pump A • RMW Pump A • Reactor Compartment Co • Penetration Cooling Fa • SFP Cooling (local NLO • Swap Lighting (local N • D/G Support System (local N	cers D poler A an A D action) ILO action)		
SRO	Direct AO to perform the • Swap Aux Bldg Lig • Provide Alternate • Cross-connect DG pump to DG B	-		
RO/SRO	Check VCT MU System			
RO/SRO	Check Charging Aligned t	O VCT		<u></u>
RO/SRO	Check the CVCS Operatior Pump)	n (adjust "A" Charging		
RO/SRO	Check Letdown in Service			
RO/SRO	Check Przr Heaters in Se	ervice		
RO/SRO	Check Rod in Auto (no Ac placed in Auto)	tion Rod cannot be		

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Response:			RATING N/A
RO/SRO	Stabilize Plant condition • Tavg • Przr Pressure • Przr Level		
BOP/SRO	Check if normal electrica restored • Refer to AR-L-5 (cannot restore due to f	_	
SRO	Check Tech Spec (3.8.9)		

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# 6. EVALUATION

Event: <u>6</u>		
Event Title:	<u>RCP B High Vibra</u>	tion
EVENT TASKS:	003-003-04-01	Respond to RCP Hi Vibration
	003-003-04-02	Direct Response to RCP High Vibration
	012-001-05-01	Manually Trip the Reactor

Expected Response/Behavior

CUES:

Alarms AR-AA-18 RCP Vibration Alert AR-AA-26 RCP Vibration Danger High Vibration indicated on B RCP at Vibration Panel

# Response:

$\smile$	SRO	Enter AR procedure AA-18 RCP Vibration Alert	 
	RO/SRO	Check RCP indications	 <u> </u>
	SRO	Notify Plant Personnel	 
	SRO	Enter AA-26 RCP Vibration Danger	 
	RO/SRO	Determine Rx Trip required Manual Trip Rx	 
	RO	Verify Rx is Tripped	 
	BOP	Verify Turbine Trip	 
	вор	Verify One Train AC Buses Energized	 

<u>RATING</u>

<u>N/A</u>

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Response:	
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# RATING N/A

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BOP/RO

SRO

Transition to ES-0.1

Determine SI not required

(NOTE RCP Seal Malfunction will start here. Some actions in ES-0.1 Rx Trip response are possible.)

<pre>vent:</pre>	C	ONSTELLATION ENERGY	NO.: 06-1-1	REV: 0
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Ivent:	E>	AMINATION SCENARIO	DATE: 03/29/06	PAGE: 23 of 29
RCP Seal High Temps CNMT Radiation Decreasing RCS Inventory Response: NOTE: Operators may go to AP-RCP.1 or AP-RCS.1 to address RCP Seal Failure and/or RCS Leakage indications prior to reaching SI Criteria.		<u>SBLOCA (RCP "B" Seal</u> 002-002-05-01 <u>Res</u>	pond to a SBLOCA	
Response: <u>RATING</u> <u>N/</u> NOTE: Operators may go to AP-RCP.1 or AP-RCS.1 to address RCP Seal Failure and/or RCS Leakage indications prior to reaching SI Criteria.	UES: RCP Se RCP Se	al Leakoff High al High Temps		
and/or RCS Leakage indications prior to reaching SI Criteria.		sing RCS Inventory		<u>RATING N/A</u>
RO/SRO Determine SI Required. Manually initiate SI.	NOTE: Op and/or RC	erators may go to AP-RG 5 Leakage indications <u>p</u>	CP.1 or AP-RCS.1 to address prior to reaching SI Criter	s RCP Seal Failure cia.
		Determine SI Requ	ired. Manually initiate S	I

CT #1 E-O- Manual	-D ly initiate at least one Train of SI	<u>SAT</u> <u>UNSAT</u>
SRO	Return to E-0.	
RO/BOP	Reverify Immediate Actions	
RO	Verify SI/RHR Pumps running (only A, C SI A RHR)	
RO	Verify CNMT Recirc Fans running	
RO	Verify CNMT spray not required	

CONSTELLATION ENERGY	NO.: 06-1-1	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #1	
EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 24 of 29

Response:		RATING	<u>N/A</u>
вор	Check MSIV isolation		
BOP	Check MFW Isolation		
вор	Verify AFW pump running A MDAFW TDAFW (Manual Start)		
вор	Verify two SW pumps running ● Ensure one pump running on Bus 14		
RO	Verify CI and CVI		
SRO	Dispatch AO to verify MOV-814 (Lost Power)		
RO	Verify the A CCW pump running		
RO	Verify SI/RHR Flow (SI flow only above RHR shut-off head)		
вор	Check AFW Valve Alignment		
BOP	Monitor Heat Sink (Maintain S/G level 7-50%)		
RO	Check ECCS Valve Alignment		
RO/SRO	Check CCW to Thermal Barriers		

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CO	NSTELLATION ENERGY	NO.: 06-1-1	REV: 0	
R. E. GIN	NA NUCLEAR POWER PLANT	TITLE: SCENARIO #1		
EX	AMINATION SCENARIO	DATE: 03/29/06	PAGE: 25 of	f 29
Response:			RATING	<u>N/A</u>
BOP	Check TDAFW Pump (Do no	ot stop)		
BOP/SRO	Control Tavg at 547°F (May require throttling closing MSIV's	g AFW to minimum and		
RO	Check PORV and Sprays (	closed		
RO/SRO	Monitor RCP Trip Crite	ria	<u>.</u>	
RO	Verify CREATS actuated			
BOP/SRO	Check S/G Secondary in	tact		
BOP/SRO	Check S/G Tube intact		<u></u>	
RO/SRO	Check RCS Intact/Trans	ition to E-1		
RO/SRO	Check RCP Trip Criteria	a		
BOP/SRO	Check that S/Gs are in	tact		
вор	Control S/G levels			
BOP/SRO	Monitor S/G Radiation			
RO	Check PORV and Block Va	alves		

CONS	STELLATION ENERGY	NO.: 06-1-1	REV: 0	
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: SCENARIO #1		<u></u>
EXAN	INATION SCENARIO	DATE: 03/29/06	PAGE: 26 o	f 29
Response:			RATING	<u>n/a</u>
RO/SRO	Reset SI		<u></u>	<b></b>
RO/SRO	Reset CI			
вор	Start a second SW pump	if required.		
SRO	Dispatch AO to perform	SD-1		

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(At this point offsite power will be lost)

CONSTELLATION ENERGY	NO.: 06-1-1	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #1	
EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 27 of 29
6. <u>EVALUATION</u>		
Event:9		

Event Title:	<u>Loss of Offsite</u>	Power		
EVENT TASKS:	064-003-04-01	Load the Diesel Generator in Unit		
	064-003-04-02	<u>Direct Action for a Diesel Generat</u> Does not start	or that	
	064-003-05-01	Respond to Inadequate SI Flow		
	<u></u>			
Expected Resp	onse/Behavior			
All SI	, 18 deenergized /RHR Pumps Trip G not Running			
Response:			RATING	<u>n/a</u>
	not running. the D/G with allowed per 1	ors may recognize the "A" D/G is The operators may manual start out referencing ECA-0.0 as A-503.1. The first part of the tten as if they go to ECA-0.0 to problem.		
SRO/BOP	Recognize lo Transition to	ss of all AC conditions. o ECA-0.0		
RO/BOP	Verify Immed • Rx trip • Turbine Tr			

BOP Control Tavg (adjust ARVs)

RO Check RCP stopped

 RO/SRO
 Isolate RCS (check)
 \_\_\_\_\_\_

 • PORV
 • Letdown

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CO	NSTELLATION ENERGY	NO.: 06-1-1	REV: 0	
R. E. GIN	NA NUCLEAR POWER PLANT	TITLE: SCENARIO #1		
EXA	MINATION SCENARIO	DATE: 03/29/06	PAGE: 28 o	of 29
Response: BOP	Verify Adequate TDAE	W Flow	RATING	<u>N/A</u>
вор	Restore "A" D/G • Check Unit/Auto • Start "A" D/G • Check Voltage/Freq • Check Cooling Avai • Start the A D/G			
CT #2 E-O Energiz to place	C te at least one train of A ting safeguard switch to p	AC Emergency buses prior pull to lock in ECA-0.0	<u>SAT</u>	<u>UNSAT</u>
SRO	Transition back to	E-0.		
SRO/RO	Recognize SI equipm started. Go to Atta Offsite Power.	ment must be manually achment 8.5 Loss of		. <u></u>
SRO/RO RO/SRO	started. Go to Atta	achment 8.5 Loss of		
	started. Go to Atta Offsite Power.	achment 8.5 Loss of		 
RO/SRO	started. Go to Atta Offsite Power. Verify CCW Pump Rur	achment 8.5 Loss of ning Running		
RO/SRO BOP/SRO	started. Go to Atta Offsite Power. Verify CCW Pump Rur Verify one SW Pump	achment 8.5 Loss of ning Running		· · · · · · · · · · · · · · · · · · ·
RO/SRO BOP/SRO BOP/SRO	started. Go to Atta Offsite Power. Verify CCW Pump Rur Verify one SW Pump Verify TDAFW Pump F Start 2 SI Pumps	achment 8.5 Loss of ning Running	 	

CONSTELLATION ENERGY	NO.: 06-1-1	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #1	
EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 29 of 29

Response:

RATING N/A

RO/SRO

1

Start Equipment • RHR (not required) • CNMT Recirc Fans

Terminate Scenario as directed by the lead examiner

CONST	ELLATION ENERGY	NO.:	06-1-2	REV: 0
R. E. GINNA I	NUCLEAR POWER PLANT	TITLE:	SCENARIO #2	
EXAMI	NATION SCENARIO	DATE:	04-04-06	PAGE: 1 of 24
Written by: <u>Ken</u>	<u>Masker</u> icense Instructor	-	Date: <u>3/29/06</u>	
01.1				
Technical Review:	Dennis Jones	-	Date: <u>5/18/06</u>	
Time validated <u>10</u>	<u>8</u> minutes By: <u>Roy Gill</u> Shift Ma	. <u>ow</u>	Date: <u>5/19/06</u>	
Date of exam:				
Examinees		•	Evaluators	
				1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 -
			· · · · · · · · · · · · · · · · · · ·	
Final review	License Instructor		Date:	
Approved for use	Director Operations Trai	ning	Date:	
	or Designee	-		

ATTRIBUTE	±
Total Malfunctions	7
Malf after EOP entry	1
Abnormal Events	5
Major Transients	1
EOP's beyond SCRAM	2
ECA's, FR's	1
Critical Tasks	5
T.S. Exercised Yes/No	Yes

CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2	
EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 2 of 24

#### 1. <u>SCENARIO OVERVIEW</u>

- 1.1 The plant is at 70% power MOL conditions holding power for Chemistry after returning "B" MFW Pump to Service following maintenance.  $C_B$  900 ppm 11,000 ppm in the BAST. The "B" MDAFW pump is OOS for motor work; out for 6 hours, expected back in ~ 6-8 hours (A-52.4 submitted 72 hour clock). The "B" SW pump is out for motor replacement.
- 1.2 The "C" Condensate Pump trips. The standby pump fails to start requiring a manual start.
- 1.3 Maintenance reports the "B" MFW has developed a severe oil leak and must be shut down immediately. Power must be rapidly reduced to less than 50% using AP-TURB.5 to allow stopping the MFW Pump.
- 1.4 Following the power reduction RCS High Activity occurs due to a minor fuel failure. The crew should enter AP-RCS.3 and place a 60 gpm Orifice in service.
- 1.5 A small letdown line leak occurs (ramp up to 30 gpm) after the letdown orifice swap causing high activity in the plant vent. The operators should remove Normal Letdown from service and place excess letdown in service.
- 1.6 A small steam line leak occurs on the tap for PT-479 causing PT-479 to fail low. The steam leak is unisolable (upstream of the Root Isolation Valve and makes the area unaccessible). The operator should defeat PT-479 per ER-INST.1 and notify higher supervision.
- 1.7 A SGTR occurs in the "B" SG.The operator should respond per E-0 and E-3. Following the cooldown in E-3 ruptured S/G pressure cannot be maintained due to the steam leak and ECA-3.1 should be entered.
- 1.8 The "A" MDAFW Pump fails to Auto Start and TDAFW pump trips when it attempts to start. The operators should manually start the "A" MDAFW Pump and establish flow to the "A" S/G to maintain a secondary heat sink.

CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2	
EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 3 of 24

### 2. <u>SCENARIO OBJECTIVES</u>

- 2.1 Respond to a Condensate Pump trip by manually starting the standby pump.
- 2.2 Perform a plant power reduction per AP-TURB.5, Rapid Load Reduction
- 2.3 Respond to an increase in RCS activity by increasing cleanup flow per AP-RCS.3, High RCS Activity
- 2.4 Respond to a letdown leak by securing letdown and placing excess letdown in service.
- 2.5 Respond to a failure of PT-479 and defeat the channel in accordance with ER-INST.1.
- 2.6 Respond to a SGTR with a secondary leak per E-0, E-3 and ECA-3.1
- 2.7 Respond to a Loss of AFW by manually starting the "A" MDAFW Pump and establishing AFW flow to the "A" S/G.

CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2	
EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 4 of 24

3. CRITICAL TASKS (Cts)

CT #1

AP-CVCS.1-A

Isolate RCS leakage path from CVCS System prior to exiting AP-CVCS.1, <u>Task</u>: CVCS Leak

Cues: Aux. Building and Plant Vent Rad monitor alarms

- •
- R-13 Plant Vent Particulate R-14 Plant Vent Gas •
- Area Monitors in the Aux. Building

Indications of CVCS Leakage Abnormal, Pressure, Temperature, Flows RCS Water balance indicate unknown leakage Aux. Building sump indications

Indicator: Closed valve indication on isolation valves in the leak path.

<u>Feedback</u>: Isolation Valve indicator closed RCS water balance indicate leakage has stopped Aux. Building indication return to normal

- Radiation • Sumps

CT #2

E-0--F

Establish 200 gpm AFW flow to the S/Gs before transition out of E-0 <u>Task</u>: occurs.

- Indications of SI Actuated Cues: Indications of SI Actuated
Indication that AFW Flowrate is less than minimum required
Indication "A" MDAFW Pump is not running
Indication of TDAFW Pump Tripped

Indicator: Manipulation of controls to establish flow to "A" S/G.

• MDAFW Pump "A" Control Switch

Feedback: Indication that at least minimum required flow is established to "A" S/G

	CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
	R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2	
	EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 5 of 24
3. <u>CRIT</u>	ICAL TASKS (Cts)		
СТ #	3		
E-3-	-A		
<u>Task</u>	: Isolate feedwater flow and st transition to ECA-3.1 occurs.		red S/G before a
<u>Cues</u>	: - Indications of a rupture in - Rx Trip - SI	one S/G	
Indi	cator: Manipulation of controls	to isolate the ruptured	S/G.
	<ul> <li>MSIV</li> <li>ARV</li> <li>Blowdown and Sample Val</li> <li>TDAFW</li> <li>AFW</li> <li>MFW</li> <li>Local Isolation</li> </ul>	ves	
Feed	<u>back</u> : Stable or Increasing Pres Ruptured S/G.	sure on Ruptured S/G. 1	No MFW or AFW flo
CT #	-4		
E-3-	-В		
<u>Task</u>	: Establish/maintain RCS Temper occur because the RCS tempera		of E-3 does not
	• Too high to obtain 20°F sub	cooling required by E-3	
	OR		
	• An Orange or Red Path occur	s on the Integrity CSFS	т.
<u>Cues</u>	<u>.</u> :		
	Indication that		
	- One S/G ruptured - Rx Trip - SI - Rupture S/G > 300 psig		
Indi	<u>cator</u> : Manipulation of control a temperature.	s required to establish	and maintain RCS

• ARVs

Feedback:

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- Steam Flow RCS Temperature decreasing RCS Temperature less than target temperature

CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2	
EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 6 of 24

3. CRITICAL TASKS (Cts)

CT #5

ECA-3.1-B

- Task: Cooldown the RCS to Cold Shutdown Conditions.
- Cues: SI Required

<u>AND</u>

Indication of a Faulted/Ruptured S/G

Indicator: Manipulations of controls as required to initiate RCS cooldown

- Steam Dump or ARVs
  AFW to maintain intact S/G level

<u>Feedback</u>: RCS Temperature Decreasing Intact S/G Pressure Decreasing

	CONSTELLATIO	N ENERGY	NO.: 06-1-2	REV: 0
R. E	R. E. GINNA NUCLEAR POWER P		TITLE: SCENARIO #2	
	EXAMINATION	SCENARIO	DATE: 03/29/06	PAGE: 7 of 24
4. <u>INSTRUC</u>	TOR ACTIONS			
Problem Tir	me	Actions		Notes
	4.1	Initialize the (setup saved as	Simulator to IC-27 s IC-172)	
		Place Block 1	≥ "B″ MDAFW Pump. Tag on Pump Switch 4000A/B or 4007)	
		• Pull stop the Place Block 1	e "B" SW Pump. Tag on Pump Switch	
		Insert Malfunct	cions	
		- MALF CND04C,		C CND Pump trips
		Trigger 1 - OVRD CND-08F		B CND Auto Start failure
		- MALF RCS16, 1 300 sec ramp Trigger 2	LO µci/ml,	RCS High Activity
		- MALF CVC2, 30 Trigger 3	) gpm, 0 ramp	CVCS Leak
		- MALF SGN3D 0	psig 0 Ramp	PT-479 Failure
		Trigger 4 - MALF STM2B, 5 Trigger4	5000 lbm/hr, 0 Ramp	Small steam leak
		- MALF SGN04B, Trigger 5	300 gpm, 0 ramp	SGTR B S/G
		- LOA FDW 30, F Trigger 30	Pump Trip	TDAFW Pump Trip
		- Set Trigger 3 T:N41B.LE.5.0		Trigger on N41<5%
		- MALF RPS07K		"A" MDAFW Pump Auto Start Failure
		IND-OVR-FDW21A OFF	/В	B MDAFW Pump oil Pu Lights
2 min.	4.2	Condensate Pum <u>r</u> Trigger 1	o Trip	
10 min	4.3	Plant Shutdown		Call as Maintenance Supervisor to repor

Call as Maintenance Supervisor to report a severe oil leak om "B" MFW Pump. Pump should be shut down immediately

TITLE: SCENARIO #2	
DATE: 03/29/06	PAGE: 8 of 24

<u>Problem_Time</u>		Actions	Notes
As soon as power decease started	4.4	RCS High Activity Trigger 2	When RP called report back on survey area dose per AB Area Monitors
3 min after swapping orifices	4.5	CVCS Leak Trigger 3	Crew should declare a Local Rad. Emergency. AO access with RP only
35 min	4.6	PT-479 Failure/Steam Leak Trigger 4	When called to check (or after 5 min.) report Intermediate Bldg. Steam Header area full of STM. Cannot get to isolation valves.
45 Min	4.7	SGTR B Steam Generator Trigger 5	If RP called for sample request applicable sample. Valve CV signal reset.
When Rx is Tripped		Ramp Steam leak up to 20K lbm/hr Malf STM2B 20000lbm/hr,120sec ramp mate_Scenario per direction of	
	the Le	ad Examiner	

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CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2	
EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 9 of 24

### 5. <u>TURNOVER INFORMATION</u>

5.1 The plant is at 70% power holding for Chemistry following return to service of "B" MFW Pump from maintenance on the oil system. RCS Boron Concentration is 900 ppm. BAST Boron Concentration is 11,000 ppm. The A and C Service Water pumps are selected for Auto start. Control Rods are in Manual due to a problem with the Rod Speed Controller.

#### 5.2 Equipment Out of Service

"B" MDAFW pump is OOS for motor work. "B" SW pump is out for motor replacement. Auto Rod Control is OOS due Rod Speed Controler Failure

#### 5.3 <u>Work in Progress</u>

"B" MDAFW Pump Motor Work "B" SW Pump Motor Replacement Rod Control System Trouble Shooting

### 5.4 <u>Planned Work</u>

Nothing additional.

#### 5.5 <u>Significant Events</u>

 $``B"\ SW$  Pump Motor was removed from service due to overheating of the winding.

### 5.6 <u>Remarks</u>

Continue power increase to 100% when Chemistry in spec. Perform AFW PT for the "B" MDAFW pump when work is complete (6-8 hours).

C	ONSTELLATION ENERGY	NO.: 06-1-2	REV: 0	
R. E. GI	NNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2		
E	CAMINATION SCENARIO	DATE: 03/29/06	PAGE: 10 of 2	24
6. <u>EVALUATION</u> Event: <u>1</u>	<u>v</u>			
Event Title:	Condensate Pump Trips			
EVENT TASKS:	056-001-04-01 Respond	ate Pump.		
	<u>056-001-04-02</u> Direct R	esponse to a loss of a	Condensate Pump	
		· · · · · · · · · · · · · · · · · · ·		
		· · · · · · · · · · · · · · · · · · ·		
Expected Resp	<u>onse/Behavior</u>			
MFW Pu	lensate Pump Trip mp NPSH Alarm			
MFW PU	mp Low Suction Press Alarm			
Response:	mp Low Suction Press Alarm			
Response:	ump Low Suction Press Alarm operator may also enter AP-F	W.1 for actions	RATING	<u>n/</u>
Response: Note the o	operator may also enter AP-H	W.1 for actions	RATING	<u>N/</u>
Response:		W.1 for actions	RATING	<u>n/</u>
Response: Note the o	operator may also enter AP-H Enter AR-H-1		RATING	<u>n/</u>
Response: Note the d	operator may also enter AP-H		<u>RATING</u>	<u>N/</u>
Response: Note the o SRO	operator may also enter AP-H Enter AR-H-1		<u>RATING</u>	<u>N/</u>
Response: Note the o SRO	operator may also enter AP-H Enter AR-H-1 Check Condensate Press Determine that the C C	ure open	<u>RATING</u>	<u></u> <u>N</u> /
Response: Note the o SRO BOP	operator may also enter AP-H Enter AR-H-1 Check Condensate Press	ure open	<u>RATING</u>	<u>n/</u>
Response: Note the o SRO BOP	operator may also enter AP-H Enter AR-H-1 Check Condensate Press Determine that the C C	ure open	<u>RATING</u>	<u>N</u> /
Response: Note the o SRO BOP	operator may also enter AP-H Enter AR-H-1 Check Condensate Press Determine that the C C	ure open ondensate Pump has	<u>RATING</u>	<u>N</u> /
Response: Note the o SRO BOP BOP/SRO	operator may also enter AP-H Enter AR-H-1 Check Condensate Press Determine that the C C tripped.	ure open ondensate Pump has	<u>RATING</u>	<u>N</u> /
Response: Note the o SRO BOP BOP/SRO BOP/SRO	Operator may also enter AP-H Enter AR-H-1 Check Condensate Press Determine that the C C tripped. Start the Standby Cond	ure open ondensate Pump has ensate Pump	<u>RATING</u>	<u>N</u> /
Response: Note the o SRO BOP BOP/SRO	operator may also enter AP-H Enter AR-H-1 Check Condensate Press Determine that the C C tripped.	ure open ondensate Pump has ensate Pump	<u>RATING</u>	<u>N</u> /
Response: Note the o SRO BOP BOP/SRO BOP/SRO	Operator may also enter AP-H Enter AR-H-1 Check Condensate Press Determine that the C C tripped. Start the Standby Cond	ure open ondensate Pump has ensate Pump BST pump		<u>N</u> /

BOP Stabilize Secondary Side Close Condensate Bypass Valve

	CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
11.15	R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2	
$\smile$	EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 11 of 24

### 6. EVALUATION

Event:	2												
Event	Title:	<u>Plant</u>	Shutdown	to	50%	power	to	remove	<b>"</b> В″	MFW	from	service	
EVENT	TASKS :												
						11 - 14 - 14 - 14 - 14 - 14 - 14 - 14 -							
			<u>.</u>										

### Expected Response/Behavior

### CUES:

Notification by Maintenance Supervisor of severe Oil Leak on "B" MFW Pump requiring pump Shutdown as soon as possible

## Response:

,

		RATING	<u>N/A</u>
SRO	Enter AP-TURB.5, Rapid Shutdown		
RO/SRO	Verify Rod in Auto. Initiate Boration		
BOP/SRO	Initiate Load Reduction		
RO	Monitor Tavg		
вор	Verify IA available to CNMT		
RO	Monitor Przr Press/Level		
BOP	Monitor S/G Levels		

CONSTELLATION ENERGY	NO.: 06-1-2 REV: 0				
R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2				
EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 12 of 24			

### Response:

BOP	Monitor Steam Dump Status	RATING	<u>N/A</u>
RO	Add Boric Acid		

CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2	
EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 13 of 24

### 6. EVALUATION

Event: <u>3</u>		
Event Title:	High RCS Activity	<u> </u>
EVENT TASKS:	002-005-04-01	Respond to High Rx Coolant Activity
	002-005-04-02	Direct Response to High Rx Coolant Activity
	344-012-04-03	Apply Tech Specs for Abnormal Events

### Expected Response/Behavior

CUES:

R-9 on Alarm Elevated Area Monitors in Aux. Building and CNMT

# 

C	ONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
R. E. GI	NNA NUCLEAR POWER PLAN	IT TITLE: SCENARIO #2	
E)	AMINATION SCENARIO	DATE: 03/29/06	PAGE: 14 of 24
6. <u>EVALUATIO</u>	Į.		
Event:4	-		
Event Title:	Letdown Line Leak		
EVENT TASKS:	004-006-04-01 R	espond to a Letdown Line Lea	ık
	<u>004-006-04-02</u> D	irect Response to a Letdown	Line_Leak
Expected Resp	onse/Behavior		
CUES:			
	ation Levels in the A ling Sump Alarms	uxiliary Building and Plant	Vent
Abnormal 1	Press/Temp/Flows in L	etdown System	
Response:			
			<u>RATING</u> <u>N/A</u>
SRO	Recognize sympto (May also enter optimal recover	oms Enter AP-CVCS.1 AP-RCS.1 but this is not the ry procedure)	e
RO/SRO	Monitor Przr Lev	vel (should be stable)	<u> </u>
RO/SRO	Check VCT M/U Sy	vstem	
RO/SRO	Check VCT M/U Sy • Align for M/U • Charging align		
RO/SRO	<ul> <li>Align for M/U</li> </ul>		
	<ul> <li>Align for M/U</li> <li>Charging align</li> </ul>	ned to VCT	
RO/SRO RO/SRO	<ul> <li>Align for M/U</li> </ul>	ned to VCT	
RO/SRO	<ul> <li>Align for M/U</li> <li>Charging align</li> <li>Check for RCS Le</li> </ul>	ned to VCT eakage	
	<ul> <li>Align for M/U</li> <li>Charging align</li> </ul>	ned to VCT eakage	
RO/SRO	<ul> <li>Align for M/U</li> <li>Charging align</li> <li>Check for RCS Le</li> </ul>	ned to VCT eakage in Aux. Building	

CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2	
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Response:

RO/SRO

<u>RATING</u> <u>N/A</u>

Check Letdown. Determine Abnormal Isolate Letdown. • Close AOV-427, 200A, B 202 • Close 371 • Close HCV-142 while adjusting charging • Close AOV-294 RO/SRO

Verify leak stopped

CT #1 AP-CVCS.1-A	<u>SAT</u>	<u>UNSAT</u>
Isolate leakage path from CVCS prior to exiting AP-CVCS.1, CVCS Leak		

RO/SRO	Place Excess L/D in service	
RO/SRO	Establish Auto Control • Charging • Przr Heaters	
All	Check Annunciators	
SRO	Notify Higher Supervision.	

	······		
c	CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
R. E. G	INNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2	
ΕΕ	XAMINATION SCENARIO	DATE: 03/29/06	PAGE: 16 of 24
6. <u>EVALUATIC</u> Event: <u>5</u> Event Title: EVENT TASKS:	Steam Leak on PT-479 Sen	a Rx Protection Channel	in the Tripped
Expected Resp CUES: PT-479 Fa	oonse/Behavior		
ADFAC Tro	ouble Alarm		
,	Note: The operato the plant	or may begin to shutdown	<u>RATING N/</u>
BOP/SRO	Verify MFW Reg and B Operating Normally (i	ypass Valves AR-G-22)	

SRO Recognize PT-479 Failed Low Enter ER-INST.1 (AR-G-22) Review Precautions

1

SRO Dispatch AO to Investigate Steam Noise

cc	INSTELLATION ENERGY	NO.: 06-1-2	REV: 0	
R. E. GINNA NUCLEAR POWER PLANT		TITLE: SCENARIO #2		
EX	AMINATION SCENARIO	DATE: 03/29/06	PAGE: 17 of 24	
BOP/SRO	Refer to Attachment fo	or PT-479	<u> </u>	
вор	Defeat Switches for P <u>479 Loop B-2</u> • LoLo Press SI • Lo Press	I-479 Defeat		
	<u>475 Loop B-2</u> ● High Trip ● HiHi Trip			
BOP	Perform Computer Defe	at		
SRO	Check Tech Specs			
	3.3.2-1 func 1e			
	3.3.2-1 func 4d and 4	e		
	3.3.3-1 func 24 and 2	5		

c	ONSTELLATION ENERGY	NO.: 06-1-2	REV: 0	
R. E. GI	NNA NUCLEAR POWER PLAN	NT TITLE: SCENARIO #2	T	
EXAMINATION SCENARIO		DATE: 03/29/06	PAGE: 18 of 24	
6. <u>EVALUATIO</u> Event: <u>6/</u> Event Title: EVENT TASKS:	7 SGTR <u>"B" S/G (Rupt</u> 035-010-05-01 F	tured Faulted S/G) Respond to a SGTR with a Fau Direct Response to a SGTR wi		
CUES:	oonse/Behavior			
R-15 A R-32 A				
R-15 A	Alarm		RATING	<u>N/</u>
R-15 A R-32 A Response: NOTE: Th Ac	Alarm Alarm e SGTR is fairly smal	ll, operators may perform sc S Leak prior to Tripping an	ome	<u>n/</u>
R-15 A R-32 A Response: NOTE: Th Ac	Alarm Alarm e SGTR is fairly smal tion in AP-RCS.1, RCS curring. Trip/verify Rx	S Leak prior to Tripping an Trip ip Required due to failure o	ome SI	<u>N/</u>
R-15 A R-32 A Response: NOTE: Th Ac occ	Alarm Alarm tion in AP-RCS.1, RCS curring. Trip/verify Rx • Manual Tr	S Leak prior to Tripping an Trip Tip Required due to failure of Actions rip rgized	ome SI	<u>N/</u>
R-15 A R-32 A Response: NOTE: Th Ac occ SRO/RO	Alarm Alarm Alarm E SGTR is fairly small tion in AP-RCS.1, RCS curring. Trip/verify Rx • Manual Tr Auto Trip Verify Immediat • Rx trip • Turbine tr • Buses ener • SI activat	S Leak prior to Tripping an Trip Tip Required due to failure of Actions rip rgized	ome SI of	<u>N</u> /

[	CON	STELLATION ENERGY	NO.: 06-1-2	REV: 0	
	R. E. GINN	A NUCLEAR POWER PLANT	TITLE: SCENARIO #2		
	EXA	MINATION SCENARIO	DATE: 03/29/06	PAGE: 19 o	f 24
	Response: BOP	Check if MSIV should be	closed	<u>RATING</u>	<u>n/a</u>
	вор	Verify MFW Isolation			
	BOP SRO/BOP	Verify MDAFW Pumps and TDAFW Pump running dete Start "A" MDAFW Pump an the "A" S/G			
	CT #2 E-0F Establis Out of E	h 200 gpm AFW flow to the -0 occurs.	S/G's before transition	<u>SAT</u>	<u>UNSAT</u>
	RO	Verify CI and CVI			
	RO	Check CCW Status			
	RO	Verify SI/RHR Flow (onl	y SI flow)		
	BOP	Verify AFW Alignment			

Γ	CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
	R. E. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2	
<i>x</i>	EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 20 of 24

### Response

		<u>Rating</u>	NA
BOP	Monitor Heat Sink • Check S/G level <50%		
RO	Check if TDAFW pump can be stopped		Change and a state of the state
RO	Check SI and RHR alignment		
вор	Check Tavg trending to 547°F (Action will need to be taken to control AFW and close the MSIV to control Tavg)		
RO	Check PORVs and Sprays		
RO/SRO	Determine if RCP Trip Criteria is met		
вор	Verify CREATS Isolation	<u> </u>	<u>-,</u>
BOP	Check S/G Secondary intact		
BOP/SRO	Check S/G Tube intact Determine SGTR occurred Transition to E-3		<u></u>
SRO/RO	Check RCP Trip criteria		
BOP/SRO	Identify "B" S/G as ruptured		

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CONSTELLATION ENERGY		NO.: 06-1-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT		TITLE: SCENARIO #2	
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BOP/SRO	Isolate Ruptured S/G • ARV 1050 psi in Auto • TDAFW Stm Valve closed • Blowdown Valve closed • MSIV closed • Attachment 16.0 (AO Di		
BOP/SRO	Check Rupture S/G Level Isolate AFW to Rupture S • MOV 4008 closed • B MDAFW Pump Pull Stor • AOV 4298 closed (TDAFW • MOV 4000A, B closed	5/G	
CT #3 E-3	A		<u>SAT</u> UNS

-

RO/SRO Reset SI

ł	CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0		
R. E. C	BINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2			
	EXAMINATION SCENARIO	DATE: 03/29/06	PAGE: 22 c	PAGE: 22 of 24	
BOP/SRO	<ul> <li>Initiate RCS Cooldown</li> <li>Determine Cooldown Ter</li> <li>Check Rupture MSIV clo</li> <li>Dump Steam via "A" AR' dump if avalible)</li> <li>Stop Cooldown and Cont</li> <li>Tavg when less than re</li> </ul>	osed V (or condenser steam trol			
E-3 d follo • Too	lish/maintain a RCP Temp. So oes not occur because RCS To wing conditions: high to obtain 20°F subcool ow the Red or Orange Path C	emp. Is either of the ling (E-3 step 20)	<u>SAT</u>	<u>UNSA'</u>	
Estab E-3 d follo • Too • Bel CSF	lish/maintain a RCP Temp. So oes not occur because RCS To wing conditions: high to obtain 20°F subcool ow the Red or Orange Path C	emp. Is either of the ling (E-3 step 20) riteria of the Integrity	<u>SAT</u>		
Estab E-3 d follo • Too • Bel	lish/maintain a RCP Temp. So oes not occur because RCS Te wing conditions: high to obtain 20°F subcool ow the Red or Orange Path C: ST	emp. Is either of the ling (E-3 step 20) riteria of the Integrity ls	<u>SAT</u>		
Estab E-3 d follo • Too • Bel CSF	lish/maintain a RCP Temp. So oes not occur because RCS Te wing conditions: high to obtain 20°F subcool ow the Red or Orange Path C: ST Monitor Intact S/G leve	emp. Is either of the ling (E-3 step 20) riteria of the Integrity ls	<u>SAT</u>		
Estab E-3 d follo • Too • Bel CSF	lish/maintain a RCP Temp. So oes not occur because RCS To wing conditions: high to obtain 20°F subcool ow the Red or Orange Path C: ST Monitor Intact S/G leve Check PORV and Block Va	emp. Is either of the ling (E-3 step 20) riteria of the Integrity ls lves open	<u>SAT</u>		

SRO/RO	Establish IA to CNMT
51107 110	<ul> <li>13/15 Normal Feed closed</li> </ul>
	• 2 SW pumps running
	<ul> <li>Turbine Building SW isolation valves open</li> </ul>
	<ul> <li>Verify adequate air compressor</li> </ul>

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	CONSTELLATION ENERGY	NO.: 06-1-2	REV: 0	
R. E	. GINNA NUCLEAR POWER PLANT	TITLE: SCENARIO #2		
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	<ul> <li>"C" Instrument Or</li> <li>Service Air Or</li> <li>Both A and B Instrum</li> <li>Reset XY Relay for AOV</li> <li>Verify AOV-5392 open</li> </ul>			
RO	Stop RHR pumps			
RO	<ul> <li>Establish Charging Flow</li> <li>Align Charging to RWST</li> <li>Start charging and ali charging flow</li> </ul>	gn to get 75 gpm		
BOP/SRC	Check if Cooldown should already be stopped)	be stopped (should		
BOP/SRC	Check Rupture S/G pressu (Note: Pressure will be decreases to < 250 psig transition to ECA-3.1)	decreasing when it		
RO/BOP/ SRO	E-3) • Reset SI/CI (step 1,2) • SW adequate (step 3)	<ul> <li>Reset SI/CI (step 1,2)</li> <li>SW adequate (step 3)</li> <li>IA restored to CNMT (step 4)</li> </ul>		
RO/SRO	Deenergize Przr Heater			
RO	Check CNMT Spray stopped	Check CNMT Spray stopped		
BOP/SRC	Check Ruptured S/G level	Check Ruptured S/G level		
RO	Check RHR pumps stopped			
SRO/RO/ BOP	Evaluate plant status • Aux. Building Rad.			

С	ONSTELLATION ENERGY	NO.: 06-1-2	REV: 0
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E	XAMINATION SCENARIO	DATE: 03/29/06	PAGE: 24 of 24
	<ul> <li>Samples</li> <li>Shroud and Reactor Comp</li> </ul>	partment Coolers	
RO	Verify 75 gpm charging es	stablished	
BOP/SRO		Check Secondary Sides of S/G intact (B decreasing but already isolated)	
вор	Control "A" (Intact) S/G		
BOP/SRO	Initiate Cooldown to Cold • Dump steam from the "A"		

CT #5 ECA-3.1		SAT	<u>UNSAT</u>
(Initiate)	Cooldown the RCS to CSD		

Terminate the scenario when a satisfactory cooldown rate is established.

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R. E. GINNA NUCLEAR POWER PLANT	TITLE: Scenario #3	
EXAMINATION SCENARIO	DATE: 4/19/06	PAGE: 1 of 25
Written by: <u>Ken Masker</u> Sr. License Instructor	Date: <u>3/30/0</u>	6
Technical Review: <u>Dennis Jones</u> Sr. License In	Date: <u>5/18/</u> structor	06
	<u>y Gillow</u> Date: <u>5/18/</u> ft Manager	06
Date of exam:		
Examinees	Evaluators	
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Final review	Date:	
License Instru	ctor	
Approved for use Director Opera or Designee	Date: tions Training	
	ATTRIBUTE	±
	Total Malfunctions Malf after EOP ent Abnormal Events Major Transients EOP's beyond SCRAM ECA's, FR's Critical Tasks T.S. Exercised Yes	ry 1 5 1 1 1 3

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CONSTELLATION ENERGY R. E. GINNA NUCLEAR POWER PLANT	NO.: 06-1-3	REV: 0
	TITLE: Scenario #3	
EXAMINATION SCENARIO	DATE: 04/19/06	PAGE: 2 of 25

#### 1. <u>SCENARIO\_OVERVIEW</u>

- 1.1 The plant is at 100% power MOL conditions RCS  $C_B$  845 ppm, BAST  $C_B$  11,000 ppm. Xenon is at equilibrium. The "A" SI Pump is out of service for motor work. "B" Charging Pump is out of service for belt replacement, "B" SW Pump is out of service for motor work.
- 1.2 A Loop "B" Thot fails high. The Tavg- average Tavg rod stops fail to function. The operators should manually control rods and defeat the channel per ER-INST.1.
- 1.3 A 20 gpm CCW leak develops in the Seal Water Heat Exchanger. The operators respond using AP-CCW.2, Loss of CCW during Power Operation, to control CCW Surge Tank level and bypass the Seal Water Heat Exchanger.
- 1.4 Both Generator Bus Duct Cooling Fans trip. This requires a load decrease to approximately 70% power in 10 minutes (AR-J-4) using procedure AP-TURB.5.
- 1.5 During the load decrease a rod lift coil for a D Bank rod fails (blown fuse) causing a misaligned rod. The operator will respond per AP-RCC.1, RCC/RPI Manlfunction. (Tech Spec 3.1.4)
- 1.6 The "D" Service Water Pump trips. The operators should respond by attempting to start the "C" SW Pump per AP-SW.2, Loss of Service Water. (Tech Spec 3.7.8. One SW Train inoperable)
- 1.7 The "C" Service Water Pump trips immediately when started resulting in only one SW Pump in service. The operators should respond by placing DG "B" in Pull Stop and isolate non-essential loads.
- 1.8 An inadvertent SI occurs. When MOV 852B opens (RHR to Rx Vessel) CV-853B fails causing an inter-system LOCA (to RHR). The RHR common header fails from over pressure resulting in a LOCA outside Containment. The operators should respond per E-0, ECA-1.2 and E-1.
- 1.9 The "B" SI Pump fails to Auto start on the SI resulting in inadequate SI for the LOCA. The operator should resond per E-0 and manully start the "B" SI Pump.

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R. E. GINNA NUCLEAR POWER PLANT	TITLE: Scenario #3	
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### 2. <u>SCENARIO OBJECTIVES</u>

- 2.1 Respond to a Thot RTD failure by controlling rods in manual and defeating the effected channel.
- 2.2 Respond to a CCW Seal Water Heat Exchanger leak per AP-CCW.2
- 2.3 Perform a rapid load reduction per AP-TURB.5 in response to a loss of Bus Duct Cooling.
- 2.4 Respond to a misaligned RCC during the power reduction using AP-RCC.2 (Apply Tech Spec 3.1.4)
- 2.5 Respond to a SW Pump trip per AP-SW.2 (Apply Tech Spec 3.7.8)
- 2.6 Respond to an inadvertant SI causing a LOCA outside Containment using E-0, ECA-1.2 and E-1.

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. L	·	EXAMINATION SCENARIO	DATE: 04/19/06	PAGE: 4 of 25		
3. CT	#1 E-0I <u>Task</u> : <u>Cues</u> :	closed <u>tor</u> : Manipulation of control r	red n SI shutoff head SI Pumps are injecting : on indicate that breake	into the RCS rs for 2 SIPs are NC		
	<u>Indicator</u> : Manipulation of control required to establish flow from at least 2 SIPs					
	Feedbad	<u>ck</u> : Indication at least 2 SI	Pumps are injecting			
]		• SI Pump Flowrate				
	CT #2					
	E-1C					
	<u>Task</u> :	Trip all RCPs within 5 minute	es of reaching Trip Crit	zeria		
	<u>Cues</u> :	Indications of a Small Break	LOCA			
		AND				
		Indications of SI				
		AND				
		Indications of only one train	n of SI available			
		AND				
		RCP Trip Criteria met				
	Indicat	tor: Manipulation of controls	required to trip both R	CPs		
	Feedba	<u>ck</u> : Both RCPs stopped				
		• RCP Breaker postion lig	<b>b b c</b>			

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R. E. GINNA NUCLEAR POWER PLANT	TITLE: Scenario #3	
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3. CRITICAL TASKS (Cts)

CT #3

ECA-1.2--A

- Task: Isolate the LOCA outside Containment before transitioning out of ECA-1.2
- <u>Cues</u>: Indication SI is actuated and required

<u>AND</u>

Indication of abnormally high radiation levels in the Auxiliary Building

<u>Indicator</u>: Manipulation of control to close the isolation valve upstream of the break (MOV 852B)

Feedback: Indication of increasing RCS pressure

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R. E. GINNA NUCLEAR POWER PLANT	TITLE: Scenario #3	
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### 4. <u>INSTRUCTOR ACTIONS</u>

	4.	INSTRUCTO	JR ACTION		
	Problem	Time		Actions	<u>Notes</u>
			4.1	Initialize the Simulator to IC-19 (setup saved as IC-173)	
				Pull stop and place Hold Tags on the following components:	Submit A52.4 for all components
				<ul> <li>A SI Pump</li> <li>B Charging Pump (A &amp; C running)</li> <li>B SW Pump (A &amp; D running)</li> <li>Set Lake Temp at 45°F EVN MIS06 45, 300 sec ramp</li> <li>Insert Malfunctions</li> </ul>	
				- MALF ROD12 Option 1	Rod Stop Failure
				- MALF CLG1C	SW Pump C trip
				- MALF RPS7B	B SIP Auto Start Failure
~				- MALF RCS-11E, 980°F,300 sec ramp Trigger1	Loop B Thot Fail High
				- MALF CLG06, 20 gpm, 0 ramp Trigger 2	Seal Water HX Leak
				- ANN OVR EDS11 Trigger 3	Isophase Bus duct Cooler Alarm
				- MALF ROD3-C7,0 Trigger 30	Rod C-7 sticks at 85% power
				SET Trigger 30 T:N41B.LE.85	
				- MALF CLG1D Trigger 4	SW Pump D trip
				- MALF SIS1,1 Trigger 5	Inadvertant SI "B" Train
				- MALF RCS19D, 900 gpm Trigger 5	LOCA to RHR System
	2 mi	n	4.2	B Loop Thot Failure Trigger 1	
	15 m	lin	4.3	Seal Water HX Leal Trigger 2	When called to check Seal Water HX flow report flow has decreased by 20 gpm from previous value

	CONSTELLATIO	N ENERGY	NO.: 06-1-3	REV: 0
R.	R. E. GINNA NUCLEAR POWER PLANT		TITLE: Scenario #3	
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35 min	4.4	Bus Duct Cooler Trigger 3	r Failures	When requested repo "A Fan Belt broken and "B" Fan tripped
				During power decrea D Bank Rod will sti When requested repo no visible damage o alarms at the rod drive cabinets
				Verify trigger 30 actuates at 85%
48 min	4.5	∾D″ SW Pump Tr: Trigger 4	Ļρ	If called to observent of "C" SW Pur report no abnormal indication locally except breaker tripped immediately on closure
1				When requested to check "D" SW Pump after trip report smell of burnt insulation.
56 min	4.6	LOCA Outside CM Trigger 5	MT	Note Aux Building Radiation levels w be high
				SI Pump Discharge Breakers
				MOV-878B - LOA EDS MOV-878D - LOA EDS

Terminate Scenario per direction of the Lead Examiner.

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CONSTELLATION ENERGY	NO.: 06-1-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Scenario #3	
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### 5. <u>TURNOVER INFORMATION</u>

5.1 The plant is at 100% power. Xenon is at equilibrium. RCS Boron Concentration is 845 ppm. BAST Boron Concentration is 11,000 ppm. The A and C Service Water pumps are selected for Auto start.

#### 5.2 Equipment Out of Service

"A" SI pump is OOS for motor work. "B" SW pump is out for motor replacement. Generator Hydrogen Control is in Manual "B" Charging Pump

### 5.3 <u>Work in Progress</u>

"A" SI pump "B" SW Pump Motor Replacement "B" Charging Pump belt Replacement (See A52.4's)

#### 5.4 Planned Work

Nothing additional.

### 5.5 <u>Significant Events</u>

``A" SI Pump had high vibration during testing that was traced to the inboard motor bearing

 $``B"\ SW$  Pump Motor was removed from service due to overheating of the winding.

"B" Charging pump belt failed.

### 5.6 <u>Remarks</u>

Continue 100% operation.

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R. E. GINNA NUCLEAR POWER PLANT	TITLE: Scenario #3	
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L	22		
6. EVALUATION	I		
Event: <u>1</u>			
Event Title:	Loop "B" Thot fa	ils High	
EVENT TASKS:	<u>001-023-01-01</u>	<u>Operate Control Rods in manual at</u>	power
	<u>012-006-01-01</u>	<u>Place a Reactor Protection Channe</u> condition	l in a tripped
Expected Respo	onse/Behavior		
CUES :			
Rods Stepp Temp. Chan Tavg - Tav	nels T403/407 fai	iling HIGH	
Response:			<u>RATING N/A</u>
	y enter AP-RCC.1 t tering into AP-RCC	to place rods in manual. This scena: C.1	rio is written for
BOP/SRO	Check Rod op	erability	
	<ul> <li>Place Room</li> </ul>	Load Stable ds in Manual od Motion Stops	
RO/SRO	Restore Tavg	to Tref	
RO/SRO	Check Tavg C	hannels	·····
	<ul><li>Determine</li><li>Refer to</li></ul>	e T403 Failed ER-INST.1	
RO/BOP/SRO	Defeat T403/	407 per ER-INST.1	
RO	Operate Rods	in Manual	
BOP	Check Steam	Dump not operating	
RO	Place Chargi PRZR Level	ng in Manual/Manually control	
RO	Check RIL Al	arms	

C	CONSTELLATION ENERGY	NO.: 06-1-3	REV: 0
R. E. G	INNA NUCLEAR POWER PLANT	TITLE: Scenario #3	
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6. EVALUA' Response:	FION (continued)		
			RATING N/
SRO	Determine Hot Leg Stre	eaming not occurring	
BOP	Defeat the Channel per Tavg 403 / Delta T	r Attachment 407	
BOP	Verify Rods in Manual,	/ Charging in Manual	
BOP	RIL Rack- Place T/405	F to Loop B Unit 1	
BOP	Steam Dump Rack- Place	e T/401B to Loop B Unit 1	
вор	B-1 Protection Rack- H switches to TRIP 403 Loop B-1 • High Tavg • Low Tavg	Place the following	
	407 Loop B-1 • Over Temp Trip • Over Power Trip		
BOP	Verify Bistable Lights	S	
BOP	Delete 403/407 group b	from PPCS Scanning	

CONS	STELLATION ENERGY	NO.: 06-1-3	REV: 0	
R. E. GINNA NUCLEAR POWER PLANT		TITLE: Scenario #3		
EXAN	EXAMINATION SCENARIO DATE: 04/19/06		PAGE: 11 of 25	
O/SRO	Restore Control System PRZR Level Control Rod Control			
BRO .	Check Tech Specs • 3.3.1 Table 3.3.1- • 3.3.2 Table 3.3.2-	1 5,6 1 4.d		
RO/BOP/SRO	Verify Control Systems • Rods	s in Auto		
	<ul> <li>EHC</li> <li>PRZR Press.</li> <li>PRZR Level</li> <li>Steam Dump</li> <li>MFW</li> <li>ARV's</li> </ul>			

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SRO

Notify Supervision/Maintenance

	ONSTELLATION ENERGY		NO.: 06-1-3	REV: 0	
R. E. G	INNA NUCLEAR POWER P	LANT	TITLE: Scenario #3		
E	XAMINATION SCENARIO		DATE: 04/19/06	PAGE: 12 of 2	25
5. <u>EVALUATIC</u> Event: <u>2</u>	<u>)N</u>				
Event Title:	Seal Water Heat	Exchanger	Leak		
EVENT TASKS:	008-011-04-01	Identify	z a CCW Leak		
	008-011-04-02	<u>Direct</u>	identification of a (	CCW Leak	
	<i>i</i>				
Expected Resp	ponse/Behavior				
CUES:					
VCT Level RCS Tavg	e Tank Level decrea Increase Increasing Low Level Alarm (A				
Response:					
				RATING	<u>N/</u>
	ne CCW Leak will ca nvestigate the caus		ution of the RCS. Th Tavg change		<u>n/</u> .
	nvestigate the caus	se of the			<u>N/</u>
ir	nvestigate the caus Recognize A-:	se of the 13 Alarm e at CCW Sur	Tavg change enter AR procedure rge Tank Level is		<u>N/</u>
ir RO/SRO	nvestigate the caus Recognize A-: Determine tha	se of the 13 Alarm e at CCW Sur Enter AP-C	Tavg change enter AR procedure rge Tank Level is		<u>N/</u>
ir RO/SRO RO/SRO	nvestigate the caus Recognize A-: Determine the decreasing. 1	se of the 13 Alarm e at CCW Sur Enter AP-C mp status	Tavg change enter AR procedure rge Tank Level is CCW.2		<u>N/</u> .
ir RO/SRO RO/SRO RO	nvestigate the caus Recognize A-: Determine the decreasing. 1 Check CCW Pur Makeup to the Open N • Start	se of the 13 Alarm e at CCW Sur Enter AP-C mp status e CCW Surg 40V 823 RMW Pump(	Tavg change enter AR procedure ge Tank Level is CCW.2		<u>N/</u>
ir RO/SRO RO/SRO RO	nvestigate the caus Recognize A-: Determine the decreasing. 1 Check CCW Pur Makeup to the Open N • Start	se of the 13 Alarm e at CCW Sur Enter AP-C mp status e CCW Surg 40V 823 RMW Pump( Y Tank lev	Tavg change enter AR procedure rge Tank Level is CCW.2 ge Tank s) el is increasing		<u>N/</u> 2

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Response:
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		RATING	<u>N/A</u>
RO	Check Letdown		
RO/SRO	<ul> <li>Check CCW Valve alignment</li> <li>Attachment 1.0 (MCB)</li> <li>Attachment 1.1 (NLO)</li> </ul>		
SRO	Dispatch NLO to check Seal Water HX (NLO reports flow is 20 gpm lower than normal)		
SRO	Direct NLO to bypass and isolate the Seal Water HX		
SRO	Notify RP to sample the RCS for Chromates		
RO	Terminate Makeup to CCW Surge Tank when level ~50%		
RO	Check for leakage in CNMT and Auxiliary Building (Step 8, 9)		
SRO	Verify leak identified and isolated		. <u> </u>
RO	Verify Normal LTDN in service		

CO	INSTELLATION ENERGY	NO.: 06-1-3	REV: 0	
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6. <u>EVALUATION</u>	, ,			
Event: 3/4				
	Isophase Bus Duct Cooling	g Failure / Stuck Rod		
		e Misaligned Control Rod		
		m a Boration of the RCS		
	045-027-04-01 Respond	d to a Rapid Load Reduction	<u>n</u>	
	045-027-04-02 Direct	Response to a Rapid Load I	Reduction	
Expected Respo	<u>&gt;nse/Behavior</u>			
CUES: Alarm A	AR-J-4			
<b>D</b>				
Response:			RATING	<u>n/a</u>
BOP	Check SW System			
	-			
SRO	Dispatch an AO to inv	vestigate		
	Note: AO will report	no fans running		
SRO	Direct Load Reduction AP-TURB.5	n to ~70% power. Enter		
RO/BOP/SRO	Initiate Load Reducti	lon		
	<ul> <li>Verify Rods in</li> </ul>	n Auto (RO)		
	<ul> <li>EHC Rate Selection</li> <li>initiated. (BO)</li> </ul>	cted and Load Reduction OP)		
		applicable only if the on method is not		
SRO	Determine Boration Te	chnique	<u> </u>	•••••
SRO	Determine Boration Te • Dispatch AO to	_	<u> </u>	
SRO		_		

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Response:

		RATING	<u>n/a</u>
RO	Monitor Tavg 545°F - 566°F		
BOP	Verify IA available to CNMT		
RO	Monitor PRZR Pressure	<u>.</u>	•
BOP	Monitor S/G Level Control		
RO	Monitor PRZR Level Control		
вор	Monitor Steam Dump		
NOTE: At 85%	power the Stuck Rod Malfunction will Auto Actuate		
RO	Determine Rod C-7 misaligned		·
SRO	Enter AP-RCC.2		
SRO/RO	Place Rods in Manual (will need to finish Load Reduction on Boric Acid or RWST to control reactivity)		
RO	Check for Dropped Rod		
RO	Check Tavg - Place EHC in Manual		
BOP	Check Steam Dump		
BOP	Check Generator Load > 15 MW		
RO/BOP/SRO	Stabilize Plant conditions		
	<ul> <li>Tavg</li> <li>PRZR Press.</li> <li>PRZR Level</li> <li>MFW</li> </ul>		

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# Response:

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RO/SRO	Check Rod Alignment	<u>RATING N/A</u>
SRO	Refer to Tech spec 3.1.4	
RO/SRO	Check QPTR within limits	
RO	Verify MRPI system operable	
RO/SRO	Check Rod Operability	

cc	DNSTELLATION ENERGY	NO.: 06-1-3	REV: 0
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6. <u>EVALUATION</u> Event: <u>5</u> Event Title: EVENT TASKS:	<u>SW Pump Trip</u> 076-004-04-01 <u>Respond</u>	to Partial Loss of SW esponse to Partial Loss c	> <u>f</u> _SW
Expected Resp	onse/Behavior		
CUES:			
"D" SW Pum PPCS SW He	mp Trips eader Low Pressure Alarms		
Response:			
		,	RATING 1
RO/SRO	Recognize SW Pump Trip Enter AP-SW.2, Loss of	Service Water	
BOP	Verify AC Emergency Bus energized	sses 17 and 18	
BOP/SRO	Verify one SW Pump runn (Only one pump running)	ning in each loop )	
NOTE: SW P	ump C will trip upon stating	a	
BOP/SRO	Start SW Pump "C" When Pump Trips go to 2	AP-SW.2 Step 3	
SRO	Dispatch AO to align a DG	lternate cooling to "B"	
BOP	Isolate SW to Non-Esser	ntial Loads	<u> </u>
вор	Isolate SW to Non-Esser Screen House Air Conditioning		<u> </u>

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### Response:

вор	Start the Service Air Compressor	<u>RATING</u>	<u>N/A</u>
BOP	Stop the C Inst Air Compressor		
SRO	Dispatch AOs to monitor SW cooled equipment		
SRO	Notify Supervision		
BOP	Check SW System		

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6. <u>EVALUAT</u> Event:	_		
Event Title	: LOCA Outside Containmen	nt	
EVENT TASKS	: <u>002-025-05-01</u> <u>Respo</u>	ond to a LOCA Outside Cont	ainment
	002-025-05-02 Direc	t Response to a LOCA Outs	<u>ide Containment</u>
Expected Re	sponse/Behavior		
CUES:			
	ated vel/Press Decreasing ry Building Radiation Alarm	ns	
Response:			RATING N/A
SRO	Recognize Rx Trip/ Enter E-0	SI	
RO/BOP	E-0 Immediate Actio	ns	
	<ul> <li>Verify Rx Trip</li> <li>Verify Turbine Tr</li> <li>Verify Busses Ener</li> <li>Check SI Actuated</li> </ul>	ip rgized (Manually Actuate SI)	
RO/SRO	Recognize "B" SI Pu	mp fail to auto start	
RO	Manually Start "B"	SI Pump	
CT #1 E-	DI		SAT UNSAT
Estak trans	lish flow from at least 2 sition out of E-0	SI pumps before	
RO	Verify CNMT Recirc :	Fans Running	

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Response:

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		RATING	<u>N/A</u>
RO	Verify CNMT Spray not required		
BOP	Check if MSIVs should be closed		
BOP	Verify MFW Isolation		
BOP	Verify MDAFW Pumps running		
BOP	Verify SW Pumps running		
RO	Verify CI and CVI		
RO	Check CCW Status		
RO	Verify SI/RHR Flow (Only SI Flow)		
BOP	Verify AFW Alignment		
вор	Monitor Heat Sink <ul> <li>Check S/G Level &lt;50%</li> </ul>		
	- Secure AFW to S/G with >50% level		
	- Control SG levels between 17-50%		
RO	Check SI and RHR Alignment		
RO	Check CCW to Thermal Barriers		

	NSTELLATION ENERGY	NO.: 06-1-3	REV: 0
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6. <u>EVALUATIO</u>	ON (continued)		
Response:			<u>RATING N/A</u>
BOP	Check if TDAFW Pump c	an be stopped	<u> </u>
	• Stop the TDAFW	V Pump	
вор	Check Tavg trending t	:0 547°F	
RO	Check PORVs and Spray	Valves	
RO/SRO			<u> </u>
			<u>SAT</u> <u>UNSAT</u>
Trip RCF	s within 5 minutes of re-	aching Trip Criteria	
вор	Verify CREAT Isolatic	n	
вор	Check S/G Secondary i	ntact	
	6. EVALUATION Response: BOP BOP RO RO RO/SRO CT #2 E-1 Trip RCE BOP	6. EVALUATION (continued) Response: BOP Check if TDAFW Pump construction of the total of total of the total of tot	6. EVALUATION (continued) Response: BOP Check if TDAFW Pump can be stopped • Stop the TDAFW Pump BOP Check Tavg trending to 547°F RO Check PORVs and Spray Valves RO/SRO Determine RCP Trip Criteria is met • Trip RCP's when met CT #2 E-1C Trip RCPs within 5 minutes of reaching Trip Criteria BOP Verify CREAT Isolation

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~ L	RO/SRO	MINATION SCENARIO Check RCS intact (No L	DATE: 04/19/06	
	RO/BOP/SRO	Check SI Termination C		
	SRO	Notify STA to monitor	CSFST	
	BOP	Control S/G Levels		<u> </u>
	RO	Reset SI/CI		
	BOP/SRO	Check SW Flow • Dispatch AO to	perform SD-1	
	BOP/SRO	Establish IA to CNMT		<u> </u>
	BOP/SRO	Check S/G Rad Levels		

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Response:		RATING	<u>n/a</u>
RO/SRO	Check Aux Building Rad Levels		
	<ul> <li>Determine Rad levels are abnormal and Transistion to ECA-1.2</li> </ul>		

The operators, based on abnormal RHR System indications may perform an anticipatory action per A-503.1 and go to the steps for isolating RHR (Step 3) NOTE:

RO

Verify normal RHR Alignment

- 700/701 Closed 720/721 Closed •
- •

RO

#### Verify CVCS Alignment

- 310/296/392A Closed 313/371 Closed •
- •

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SRO/RO Check for backflow into ECCS System

- SI Reset
- Close 852A
- Check for RCS pressure increase (none)
- Open 852A
- Close 852B
- Verify RCS Pressure increasing

CT #3 ECA-1.2--A <u>SAT</u> UNSAT Isolate the LOCA Outside Containmant before \_\_\_\_\_\_ \_\_\_\_ transistioning out of ECA-1.2

RO/SRO

Check if Leak is isolated

• Go to E-1

NOTE: At some point SI Termination Criteria will be met. When it is, the operator should trainsition to ES-1.1, SI Termination.

NOTE: Action Step 1-9 have already been completed in E-0. Step 10 begins new actions.

CONSTELLATION ENERGY R. E. GINNA NUCLEAR POWER PLANT EXAMINATION SCENARIO		NO.: 06-1-3	REV: 0
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RO/BOP/SRO	<ul> <li>Verify action already</li> <li>RCP Trip Criter</li> <li>S/Gs intact</li> <li>Control S/G lev</li> <li>Monitor Seconda</li> <li>PORVs</li> </ul>	els	
POD /CDO	<ul> <li>Reset SI/CI</li> <li>SW Flow</li> <li>IA to CNMT rest</li> </ul>		
BOP/SRO RO	Verify Normal Power t Establish Charging Fl		·
ĸŬ			
	• Check RCP Seal		
		Suction to RWST	
	• Start Charging	to restore PRZR level	
RO/BOP/SRO	Check SI Termination	Criteria	. <u> </u>
	• Transition to E	8-1.1	

• Transition to ES-1.1