

December 19, 2006

Gene F. St. Pierre, Site Vice President  
FPL Energy Seabrook, LLC  
c/o Mr. James M. Peschel  
Seabrook Station  
P. O. Box 300  
Seabrook, NH 03874

SUBJECT: RECEIPT OF SEABROOK STATION UNIT 1 RESPONSE TO GENERIC  
LETTER 2003-01 "CONTROL ROOM HABITABILITY" (TAC NO. MB9855)

Dear Mr. St. Pierre:

The Nuclear Regulatory Commission (NRC) acknowledges the receipt of your responses to Generic Letter (GL) 2003-01, "Control Room Habitability," dated August 11, 2003 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML032260325), December 9, 2003 (ADAMS Accession No. ML033450304), and October 29, 2004 (ADAMS Accession No. ML033450304). This letter provides the results of the NRC staff's review of FPL Energy Seabrook, LLC's (FPLE's) response to the GL 2003-01 and documents the follow-up actions committed to by FPLE.

GL 2003-01 requested that FPLE confirm that the Seabrook Station, Unit No. 1 (Seabrook) control room meets the design bases (e.g. General Design Criteria (GDC) 1, 3, 4, 5 and, 19, draft GDC, or principal design criteria), with special attention to: (1) determination of the most limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in the Seabrook design bases for meeting control room operator dose limits from accidents (GL 2003-01, Item 1a); (2) determination that the most limiting unfiltered inleakage is incorporated into the Seabrook hazardous chemical assessments (GL 2003-01, Item 1b); and (3) determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 2003-01, Item 1b). GL 2003-01 further requested information on any compensatory measures in use to demonstrate control room habitability, and plans to retire them (GL 2003-01, Item 2).

By letter dated December 12, 2003 (ADAMS Accession No. ML033450304), FPLE reported the results of ASTM E741 (American Society for Testing Materials, Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution) tracer gas tests for the Seabrook control room which is pressurized for accident mitigation.

This testing determined that the maximum tested value for unfiltered inleakage into the control room envelope (CRE) was 14 (+/- 22) standard cubic feet per minute (scfm), which is less than the value of 150 scfm assumed in Seabrook's current design basis radiological dose analyses for control room habitability (CRH). It should be noted that this design basis was revised by Amendment Number 100 to Facility Operating License (FOL) NPF-86 (ADAMS Accession No. ML050320373) issued on February 24, 2005.

In its response, FPLE indicated that compliance to Regulatory Guide (RG) 1.78 and RG 1.95 at Seabrook is documented in Final Safety Analysis Report (FSAR) Amendment 63, Section 1.8 and that toxic chemical hazards do not exist on-site or in the vicinity of the plant. Therefore, the unfiltered inleakage value measured during tracer gas testing does not have any impact on the hazardous chemical assessment for Seabrook. FPLE also indicated that reactor control capability is maintained from either the control room or the remote shutdown location panel in the event of smoke.

The GL further requested that licensees assess their Technical Specifications (TSs) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design basis analysis for CRH in light of the demonstrated inadequacy of a delta ( $\Delta$ ) P measurement to alone provide such verification (GL 2003-01, Item 1c). As permitted by the GL, FPLE provided a schedule for revising the surveillance requirement (SR) in the TSs to reference an acceptable surveillance methodology. In its December 9, 2003, response, FPLE indicated that it will submit a license amendment request (LAR) to adopt TS SRs that verify CRH per Technical Specification Task Force (TSTF)-448, within six months following NRC approval of TSTF-448.

The information provided indicated that there were no compensatory measures needed to be in place to demonstrate CRH. The NRC staff notes that FPLE relied on an operability determination to demonstrate CRH prior to the issuance of Amendment Number 100 to FOL NPF-87, as noted above.

The information provided also supported the conclusion that FPLE is committed to meet the GDC regarding CRH as documented in the Seabrook FSAR and its Updated Final Safety Analysis Report.

Further, FPLE committed to submit an LAR based on TSTF-448, following NRC formal review and approval. Based on the above discussions, the NRC staff consider your August 11, 2003, December 9, 2003, and October 29, 2004 letters responsive to GL 2003-01 and all actions with respect to the GL to be closed.

If you have any questions regarding this correspondence, please contact me at (301) 415-2481.

Sincerely,

**/RA/**

G. Edward Miller, Project Manager  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-443

cc: See next page

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