

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.225 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. Records

ENO shall keep facility operating records in accordance with the requirements of the Technical Specifications.

D. Equalizer Valve Restriction - DELETED

E. Recirculation Loop Inoperable - DELETED

F. Fire Protection

ENO shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility and as approved in the SER dated December 21, 1978 as supplemented subject to the following provision:

ENO may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

G. Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21, is entitled: "Pilgrim Nuclear Power Station Physical Security, Training and Qualification, and Safeguards Contingency Plan, Revision 0" submitted by letter dated October 13, 2004.

LIMITING CONDITIONS FOR OPERATION

3.6 PRIMARY SYSTEM BOUNDARY (Cont)

C. Coolant Leakage

Any time irradiated fuel is in the reactor vessel and coolant temperature is above 212°F, the following limits shall be observed:

1. Operational Leakage

a. Reactor coolant system leakage shall be limited to:

1. No Pressure Boundary Leakage
2. ≤ 5 gpm Unidentified Leakage
3. ≤ 25 gpm Total Leakage averaged over any 24 hour period.
4. ≤ 2 gpm increase in Unidentified Leakage within any 24 hour period when in RUN mode.

b. With any reactor coolant system leakage greater than the limits of 2. and/or 3., above, reduce the leakage to within acceptable limits within 4 hours or be in at least Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

c. With any reactor coolant system leakage greater than the limits of 4. above, identify the source of leakage within 4 hours or be in at least Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.6 PRIMARY SYSTEM BOUNDARY (Cont)

C. Coolant Leakage

Any time irradiated fuel is in the reactor vessel and coolant temperature is above 212°F, the following surveillances shall be performed:

1. Operational Leakage

Demonstrate drywell leakage is within the limits specified in 3.6.C.1 at least once every 8 hours.

LIMITING CONDITIONS FOR OPERATION

3.6 PRIMARY SYSTEM BOUNDARY (Cont)

C. Coolant Leakage (Cont)

- d. When any Pressure Boundary Leakage is detected be in at least Hot Shutdown within the next 12 hours and be in Cold Shutdown within the next 24 hours.

2. Leakage Detection Systems

- a. The following reactor coolant system leakage detection systems shall be Operable:

1. The drywell floor drain sump monitoring system, and either
2. One channel of a drywell atmospheric particulate radioactivity monitoring system, or
3. One channel of a drywell atmospheric gaseous radioactivity monitoring system.

- b. 1. With the drywell floor drain monitoring system required by 3.6.C.2.a.1 inoperable, restore it to Operable status within 30 days, otherwise, be in Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

2. With both the gaseous and particulate radioactivity monitoring channels required by 3.6.C.2.a.2 and 3.6.C.2.a.3 inoperable, reactor operation may continue for up to 30 days provided drywell atmosphere grab samples are analyzed every 12 hours, otherwise, be in Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.6 PRIMARY SYSTEM BOUNDARY (Cont)

C. Coolant Leakage (Cont)

2. Leakage Detection Systems

The following reactor coolant leakage detection systems shall be demonstrated Operable:

- a. For the drywell floor drain sump monitoring system perform:

1. An instrument functional test at least once per 31 days, and
2. An instrument channel calibration at least once per operating cycle.

- b. For each required drywell atmospheric radioactivity monitoring system perform:

1. An instrument check at least once every 12 hours,
2. An instrument functional test at least once per 31 days, and
3. An instrument channel calibration at least once per operating cycle.

LIMITING CONDITIONS FOR OPERATION

3.6 PRIMARY SYSTEM BOUNDARY (Cont)

- c. With no required leakage detection systems Operable, be in Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

D. Safety and Relief Valves

1. During reactor power operating conditions and prior to reactor startup from a Cold Condition, or whenever reactor coolant pressure is greater than 104 psig and temperature greater than 340°F, both safety valves and the safety modes of all relief valves shall be operable. The nominal setpoint for the relief/safety valves shall be selected between 1095 and 1115 psig. All relief/safety valves shall be set at this nominal setpoint \pm 11 psi. The safety valves shall be set at 1240 psig \pm 13 psi.
2. If Specification 3.6.D.1 is not met, an orderly shutdown shall be initiated and the reactor coolant pressure shall be below 104 psig within 24 hours.

NOTE

Technical Specifications 3.6.D.3 - 3.6.D.5 apply to the two Stage Target Rock relief valves.

3. If the temperature of any safety relief discharge pipe exceeds 212°F during normal reactor power operation for a period of greater than 24 hours, an engineering evaluation shall be performed justifying continued operation for the corresponding temperature increases.

SURVEILLANCE REQUIREMENTS

4.6 PRIMARY SYSTEM BOUNDARY (Cont)

D. Safety and Relief Valves

1. Testing of safety and relief/safety valves shall be in accordance with 3.13.
2. At least one of the relief/safety valves shall be disassembled and inspected each refueling outage.
3. Whenever the safety relief valves are required to be operable, the discharge pipe temperature of each safety relief valve shall be logged daily.
4. Instrumentation shall be calibrated and checked as indicated in Table 4.2.F.