

November 27, 2006

Mr. Christopher M. Crane
President and Chief Nuclear Officer
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: BYRON STATION, UNIT NOS. 1 AND 2, AND BRAIDWOOD STATION, UNIT NOS. 1 AND 2 - ISSUANCE OF AMENDMENTS RE: REACTOR COOLANT SYSTEM PRESSURE AND TEMPERATURE LIMITS REPORT (TAC NOS. MC8693, MC8694, MC8695 AND MC8696)

Dear Mr. Crane:

The Commission has issued the enclosed Amendment No. 148 to Facility Operating License No. NPF-37 and Amendment No. 148 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively, and Amendment No. 142 to Facility Operating License No. NPF-72 and Amendment No. 142 to Facility Operating License No. NPF-77 for the Braidwood Station, Unit Nos. 1 and 2, respectively. The amendments are in response to your application dated October 3, 2005. An Amendment No. to Facility Operating License No. NPF-72 (Braidwood Station, Unit No. 1) will not be issued so that the Amendment numbers for Braidwood Station are concurrent.

The amendments revise the Technical Specification (TS) Section 1.1, "Definitions," description of the Pressure and Temperature Limits Report (PTLR), by deleting reference to specifications containing limits in the PTLR; revise administrative controls TS 5.6.6, "Reactor Coolant System (RCS) PTLR," by requiring the NRC approval documents to be identified by date and topical reports to be identified by number and title; and (3) add Westinghouse Electric Company, LLC report, WCAP-16143, "Reactor Vessel Closure Head/Vessel Flange Requirements Evaluation for Byron/Braidwood Units 1 and 2," to the list of analytical methods provided in TS 5.6.6.

C. Crane

-2-

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA by C.Gratton for/

Robert F. Kuntz, Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455,
STN 50-456 and STN 50-457

Enclosures:

1. Amendment No. 148 to NPF-37
2. Amendment No. 148 to NPF-66
3. Amendment No. 142 to NPF-72
4. Amendment No. 142 to NPF-77
5. Safety Evaluation

cc w/encls: See next page

C. Crane

-2-

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA by C. Gratton for/

Robert F. Kuntz, Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
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1. Amendment No. 148 to NPF-37
2. Amendment No. 148 to NPF-66
3. Amendment No. 142 to NPF-72
4. Amendment No. 142 to NPF-77
5. Safety Evaluation

cc w/encls: See next page

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Amendment: ML062610513

Tech Spec Pages: ML * No major changes to SE **NLO

OFFICE	PM:LPL3-2	LA:LPL3-2	BC:DCI/CVIB	DIRS/ITSB	OGC	BC:LPL3-2
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DATE	11/27/06	11/27/06	4/20/2006	11/13/2006	11/22/2006	11/27/2006

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EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-454

BYRON STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 148
License No. NPF-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated October 3, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-37 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 148 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Michael Marshall, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance: November 27, 2006

EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-455

BYRON STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 148
License No. NPF-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated October 3, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A (NUREG-1113), as revised through Amendment No. 148 and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-37, dated February 14, 1985, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Michael Marshall, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance: November 27, 2006

ATTACHMENT TO LICENSE AMENDMENT NOS. 148 AND 148

FACILITY OPERATING LICENSE NOS. NPF-37 AND NPF-66

DOCKET NOS. STN 50-454 AND STN 50-455

Replace the following pages of the Facility Operating License and Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

Unit 1 License Page 3
Unit 2 License Page 3
1.1-6
5.6-5

Insert

Unit 1 License Page 3
Unit 2 License page 3
1.1-6
5.6-5

EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-456

BRAIDWOOD STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. NPF-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated October 3, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 142 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Michael Marshall, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance: November 27, 2006

EXELON GENERATION COMPANY, LLC

DOCKET NO. STN 50-457

BRAIDWOOD STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. NPF-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated October 3, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 142 and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-72, dated July 2, 1987, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Michael Marshall, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance: November 27, 2006

ATTACHMENT TO LICENSE AMENDMENT NOS. 142 AND 142

FACILITY OPERATING LICENSE NOS. NPF-72 AND NPF-77

DOCKET NOS. STN 50-456 AND STN 50-457

Replace the following pages of the Facility Operating Licenses and Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

Unit 1 License Page 3
Unit 2 License Page 3
1.1-6
5.6-5

Insert

Unit 1 License Page 3
Unit 2 License Page 3
1.1-6
5.6-5

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 148 TO FACILITY OPERATING LICENSE NO. NPF-37,
AMENDMENT NO. 148 TO FACILITY OPERATING LICENSE NO. NPF-66,
AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. NPF-72,
AND AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. NPF-77

EXELON GENERATION COMPANY, LLC

BYRON STATION, UNIT NOS. 1 AND 2

BRAIDWOOD STATION, UNIT NOS. 1 AND 2

DOCKET NOS. STN 50-454, STN 50-455, STN 50-456 AND STN 50-457

1.0 INTRODUCTION

By letter to the Nuclear Regulatory Commission (NRC, the Commission) dated October 3, 2005 (Agencywide Documents Access and Management System (ADAMS) Accession Number ML052780469), Exelon Generation Company, LLC (Exelon) requested changes to the technical specifications (TSs) for Byron Station, Unit Nos. 1 and 2 (Byron), and the Braidwood Station, Unit Nos. 1 and 2 (Braidwood).

The proposed amendment would, in accordance with Technical Specification Task Force (TSTF) Standard Technical Report Specification Change Traveler, TSTF-419, "Revise PTLR Definition and References in ISTS [Improved Standard Technical Specifications] 5.6.6, RCS [Reactor Coolant System] PTLR:"

- revise TS Section 1.1, "Definitions," description of the PTLR by deleting reference to specifications containing limits in the PTLR; and
- revise the administrative controls TS 5.6.6, "Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR)," by requiring the NRC approval documents to be identified by date, and topical reports to be identified by number and title.

The proposed license amendment would also add Westinghouse Electric Company, LLC, WCAP-16143, "Reactor Vessel Closure Head/Vessel Flange Requirements Evaluation for Byron/Braidwood Units 1 and 2," to the list of analytical methods provided in TS 5.6.6. The associated exemption request was considered separately.

2.0 BACKGROUND

Exelon has requested that the Byron and Braidwood TS be modified to incorporate the changes in TSTF-419 that have been approved generically. TSTF-419 received NRC approval on March 21, 2002 (ADAMS Accession Number ML020800488).

NRC Generic Letter (GL) 96-03, "Relocation of the Pressure and Temperature Limit Curves and Low Temperature Overpressure Protection [LTOP] System Limits," dated January 31, 1996, allowed licensees to relocate the pressure-temperature (P-T) limit curves and the LTOP system limits from the TS to a PTLR.

The methodology used to determine the P-T limit curves and the LTOP system limits is required to comply with the specific requirements of Appendix G, "Fracture Toughness Requirements," and Appendix H, "Reactor Vessel Material Surveillance Program Requirements," to Title 10 of the *Code of Federal Regulations* (10 CFR), Part 50, "Domestic Licensing of Production and Utilization Facilities," to be documented in an NRC-approved topical report or an NRC-approved plant-specific submittal, and to be incorporated by reference into the TS. Subsequent changes in methodology must be approved by a license amendment.

The proposed changes are as follows:

A) Change to the TS Definition Section for PTLR (TS Section 1.1)

TS Section 1.1 currently defines the PTLR as the unit-specific document that provides the reactor vessel P-T limits including heatup and cooldown rates, and the pressurizer power-operated relief valve (PORV) lift settings for the current reactor vessel fluence period. These P-T limits shall be determined for each fluence period in accordance with TS 5.6.6. Unit operation within these limits is addressed in Limiting Condition for Operation (LCO) 3.4.3, "RCS Pressure and Temperature (P/T) Limits," and LCO 3.4.12, "Low Temperature Overpressure Protection (LTOP) System."

The proposed change deletes the reference to specifications containing limits in the PTLR. The TS Section 1.1 definition of PTLR has been revised to state:

The PTLR is the unit specific document that provides the reactor vessel pressure and temperature limits including heatup and cooldown rates, and the pressurizer Power Operated Relief Valve (PORV) lift settings for the current reactor vessel fluence period. These pressure and temperature limits shall be determined for each fluence period in accordance with Specification 5.6.6.

B) Changes to the Administrative Controls Section 5.6.6, Pressure Temperature Limits Report

The requirements in TS 5.6.6, to identify the NRC staff approval document by date, are revised to allow identification of the topical report(s) by number and title, or the NRC safety evaluation (SE) for a plant-specific methodology by NRC letter and date.

A requirement is added to specify that the complete citation in the PTLR for each topical report includes the report number, title, revision, date, and any supplements.

TS 5.6.6.b requires that the analytical methods used to determine the RCS P-T limits shall be those previously reviewed and approved by the NRC, specifically those described in NRC letters dated January 21, 1998, "Byron Station Units 1 and 2, and Braidwood Station, Units 1 and 2, Acceptance for Referencing of Pressure Temperature Limits Report," and August 8, 2001, "Issuance of Exemption from the requirements of 10 CFR 50 Part 60 and Appendix G for Byron Station, Units 1 and 2 and Braidwood Station, Units 1 and 2."

- C) The proposed change reformats TS 5.6.6.b. WCAP-16143 is added as an analytical method approved by the NRC as follows:

The analytical methods used to determine the RCS pressure and temperature limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents (Note: Citation number 2, NRC letter dated August 8, 2001, was titled "Issuance of Exemption from the requirements of 10 CFR 50 Part 60 and Appendix G, for Byron Station, Units 1 and 2 and Braidwood Station, Units 1 and 2." To clarify the regulation being referenced the title has been altered in the citation as noted below):

1. NRC letter dated January 21, 1998, "Byron Station Units 1 and 2, and Braidwood Station, Units 1 and 2, Acceptance for Referencing of Pressure Temperature Limits Report,"
2. NRC letter dated August 8, 2001, "Issuance of Exemption from the requirements of 10 CFR 50.60 and Appendix G, for Byron Station, Units 1 and 2 and Braidwood Station, Units 1 and 2,"
3. Westinghouse WCAP-16143, "Reactor Vessel Head Closure/Vessel Flange Requirements Evaluation for Byron/Braidwood Units 1 and 2,"
4. The PTLR will contain the complete identification for each of the TS referenced Topical Reports used to prepare the PTLR (i.e., report number, title, revision, date, and any supplements).

Exelon also requested an exemption from 10 CFR 50.60, "Acceptance criteria for fracture prevention measures for light-water nuclear power reactors for normal operation." Specifically, the requested exemption will allow the use of WCAP-16143 in calculating the reactor pressure vessel (RPV) RPV P-T limits for Byron and Braidwood, in lieu of 10 CFR Part 50, Appendix G paragraph IV.A.2.c (specifically, those requirements related to the application of Footnote 2 to Table 1) as required by 10 CFR 50.60(a). The change in methodology must be approved by the NRC staff in a license amendment. As discussed below, the exemption request was considered separately. This amendment request, therefore, proposes to incorporate WCAP-16143 in the approved methodologies listed in TS 5.6.6.b.

3.0 TECHNICAL EVALUATION

The proposed changes to Byron and Braidwood TSs are evaluated below.

A. Change to the TS 1.1 Definitions Section on PTLR

The definition of PTLR currently identifies the specifications in which the P-T limits are addressed. TS 5.6.6.a requires that the individual specifications that address RCS P-T limits be referenced. The proposed changes to the definition eliminate the duplication between the definition of PTLR in TS 1.1 and TS 5.6.6. Therefore, the NRC staff finds that the proposed change to the TS definitions section for PTLR is acceptable.

B. Changes to the Administrative Controls Section 5.6.6, Pressure Temperature Limits Report

The revision to TS 5.6.6 to allow the topical reports to be identified by number and title, or the NRC SE for a plant-specific methodology by NRC letter and date, would allow licensees to use current NRC-approved topical reports to support limits in the PTLR without having to submit an amendment to the facility operating license every time the topical report is revised. The PTLR would provide the specific information identifying the particular approved topical report(s) used to determine the P-T limits or LTOP system limits. This provides reasonable assurance that only NRC-approved versions of the referenced topical reports or plant-specific methodologies will be used for the determination of the P-T limits or LTOP system limits since the complete citation will be provided in the PTLR.

The requirement to operate within the limits in the PTLR is specified in and controlled by the TS. Only the figures, values, and parameters associated with the P-T limits and LTOP setpoints are relocated to the PTLR. The methodology for their development must be reviewed and approved by the NRC. The proposed changes do not change the requirements associated with the review and approval of the methodology or the requirement to operate within the limits specified in the PTLR. Therefore, the NRC staff finds that the proposed changes to the administrative controls Section 5.5.6 are acceptable.

C. Addition of WCAP-16143 to the list of approved methodologies

The NRC staff reviewed the methodology provided in WCAP-16143 as part of the evaluation of Exelon's request for an exemption from 10 CFR 50.60. The NRC staff examined Exelon's rationale to support the exemption request and agreed that based on the information provided in WCAP-16143-P and Exelon's October 3, 2005 letter, an acceptable technical basis has been established to exempt Byron and Braidwood from requirements related to the application of Footnote 2 to Table 1 of 10 CFR Part 50, Appendix G. The technical basis provided by Exelon has established that an adequate margin of safety against brittle failure would continue to be maintained for the Byron and Braidwood RPVs without the application of those requirements related to the application of Footnote 2 to Table 1 of 10 CFR Part 50, Appendix G for normal operation under both core critical and core non-critical conditions and RPV hydrostatic and leak test conditions. The NRC staff approved the methodology and the exemption in a letter dated

November 22, 2006 (ADAMS Accession Number ML061890003). The NRC staff finds the proposed addition of WCAP-16143 to the list of references for the Byron and Braidwood PTLR methodologies in TS 5.6.6.b to be acceptable.

3.1 Technical Evaluation Conclusion

The NRC staff has reviewed Exelon's proposed changes to the Byron and Braidwood TS. The proposed changes are consistent with the ISTS, satisfy the 10 CFR 50.36(c)(2)(ii) criteria, maintain safety consistent with the Byron and Braidwood safety analyses and are acceptable for inclusion in the TSs.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments change the requirements with respect to installation or use of a facilities components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (71 FR 13175; March 14, 2006). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Hardies

Date: November 27, 2006

- (3) Exelon Generation Company, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Exelon Generation Company, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (5) Exelon Generation Company, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. The license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels is not in excess of 3586.6 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein and other items identified in Attachment 1 to this license. The items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license.
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 142, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
 - (3) Emergency Planning

In the event that the NRC finds that the lack of progress in completion of the procedures in the Federal Emergency Management Agency's final rule, 44 CFR Part 350, is an indication that a major substantive problem exists in achieving or maintaining an adequate state of emergency preparedness, the provisions of 10 CFR Section 50.54(s)(2) will apply.

Amendment No. 142

material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (4) Exelon Generation Company, LLC pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts are required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Exelon Generation Company, LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. The license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels is not in excess of 3586.6 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein and other items identified in Attachment 1 to this license. The items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license.

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No.142, and the Environmental Protection Plan contained in Appendix B, both of which are attached to License No. NPF-72, dated July 2, 1987, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Emergency Planning

In the event that the NRC finds that the lack of progress in completion of the procedures in the Federal Emergency Management Agency's final rule, 44 CFR Part 350, is an indication that a major substantive problem exists in achieving or maintaining an adequate state of emergency preparedness, the provisions of 10 CFR Section 50.54(s)(2) will apply.

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulation set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of 3586.6 megawatts thermal (100 percent power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 148, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Deleted.

(4) Deleted.

(5) Deleted.

(6) The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the licensee's Fire Protection Report, and as approved in the SER dated February 1987 through Supplement No. 8, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulation set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of 3586.6 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A (NUREG-1113), as revised through Amendment No. 148, and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-37, dated February 14, 1985, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- (3) Deleted.
- (4) Deleted.
- (5) Deleted.

Byron/Braidwood Stations

cc:

Dwain W. Alexander, Project Manager
Westinghouse Electric Corporation
Energy Systems Business Unit
Post Office Box 355
Pittsburgh, PA 15230-0355

Howard A. Learner
Environmental Law and Policy
Center of the Midwest
35 East Wacker Dr., Suite 1300
Chicago, IL 60601-2110

U.S. Nuclear Regulatory Commission
Byron Resident Inspectors Office
4448 N. German Church Road
Byron, IL 61010-9750

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
Suite 210
2443 Warrenville Road
Lisle, IL 60532-4351

Ms. Lorraine Creek
RR 1, Box 182
Manteno, IL 60950

Chairman, Ogle County Board
Post Office Box 357
Oregon, IL 61061

Mrs. Phillip B. Johnson
1907 Stratford Lane
Rockford, IL 61107

Attorney General
500 S. Second Street
Springfield, IL 62701

Illinois Emergency Management
Agency
Division of Disaster Assistance &
Preparedness
110 East Adams Street
Springfield, IL 62701-1109

Plant Manager - Byron Station
Exelon Generation Company, LLC
4450 N. German Church Road
Byron, IL 61010-9794

Site Vice President - Byron
Exelon Generation Company, LLC
4450 N. German Church Road
Byron, IL 61010-9794

U.S. Nuclear Regulatory Commission
Braidwood Resident Inspectors Office
35100 S. Rt. 53, Suite 79
Braceville, IL 60407

County Executive
Will County Office Building
302 N. Chicago Street
Joliet, IL 60432

Plant Manager - Braidwood Station
Exelon Generation Company, LLC
35100 S. Rt. 53, Suite 84
Braceville, IL 60407-9619

Ms. Bridget Little Rorem
Appleseed Coordinator
117 N. Linden Street
Essex, IL 60935

Document Control Desk - Licensing
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

Site Vice President - Braidwood
Exelon Generation Company, LLC
35100 S. Rt. 53, Suite 84
Braceville, IL 60407-9619

Senior Vice President - Operations Support
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

Byron/Braidwood Stations

- 2 -

Director - Licensing and Regulatory
Affairs
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

Senior Vice President - Midwest Operations
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

Manager Regulatory Assurance - Braidwood
Exelon Generation Company, LLC
35100 S. Rt. 53, Suite 84
Braceville, IL 60407-9619

Manager Regulatory Assurance - Byron
Exelon Generation Company, LLC
4450 N. German Church Road
Byron, IL 61010-9794

Assistant General Counsel
Exelon Generation Company, LLC
200 Exelon Way
Kennett Square, PA 19348

Vice President - Regulatory & Legal Affairs
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

Manager Licensing - Braidwood/Byron
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555