

October 3, 2006

Mr. L. M. Stinson
Vice President - Nuclear
Hatch Project
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, AL 35201-1295

SUBJECT: EVALUATION OF EDWIN I. HATCH NUCLEAR PLANT, UNIT NOS. 1 AND 2,
RESPONSE TO GENERIC LETTER 2003-01, "CONTROL ROOM
HABITABILITY" (TAC NOS. MB9810 AND MB9811)

Dear Mr. Stinson:

The Nuclear Regulatory Commission (NRC) acknowledges the receipt of your responses for Edwin I. Hatch Nuclear Plant, Unit Nos. 1 and 2 (Hatch), to Generic Letter (GL) 2003-01, "Control Room Habitability," dated August 4, 2003 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML032190245); March 29, 2004 (ADAMS Accession No. ML040910289); October 27, 2004 (ADAMS Accession No. ML043030315); November 10, 2005 (ADAMS Accession No. ML053180074); and September 1, 2006 (ADAMS Accession No. ML062480132). This letter provides a status of your response and describes any actions that may be required to consider your response to GL 2003-01 complete.

GL 2003-01 requested that you confirm that your control room meets its design bases (e.g., General Design Criteria (GDC) 1, 3, 4, 5, & 19, draft GDC, or principal design criteria), paying special attention to the: (1) determination of the most-limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in your design bases for meeting control room operator dose limits from accidents (GL 2003-01, Item 1a); (2) determination that the most-limiting unfiltered inleakage is incorporated into your hazardous chemical assessments (GL 2003-01, Item 1b); and, (3) determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 2003-01, Item 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability, and the plans to retire them (GL 2003-01, Item 2).

In your letter dated September 1, 2006, you reported the results of tracer gas tests of the Hatch control room that were conducted in accordance with American Society for Testing and Materials E741, "Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution." The control room uses a positive pressure design. You determined that the maximum tested value for inleakage into the control room envelope (CRE) was 5 standard cubic feet per minute (scfm), which is less than the value of 110 scfm assumed in your design-basis radiological dose analysis.

You also reported that the maximum tested unfiltered inleakage for toxic chemical release operation mode is 16 scfm, but did not report what you assumed in your chemical assessment evaluations. However, you indicated that all industrial facilities located within 5 miles of the plant and all hazardous chemicals currently manufactured, stored, or transported in the vicinity

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of the plant have been assessed for control room habitability (CRH) and determined to pose no threat to the plant. You also indicated that reactor control capability is maintained from either the control room or the remote safe shutdown panel in the event of smoke.

The GL further requested that you assess your technical specifications (TSs) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design-basis analysis for CRH, in light of the demonstrated inadequacy of a differential pressure measurement to alone provide such verification (GL 2003-01, Item 1c). As permitted by the GL, you provided a schedule for revising the surveillance requirement in the TSs to reference an acceptable surveillance methodology. In your September 1, 2006, response you stated that you will implement Technical Specification Task Force (TSTF) 448, "Control Room Habitability," once it has been approved by the NRC staff.

The information you provided supported the fact that there are no compensatory measures needed to be in place to demonstrate CRH. The information you provided also supported the fact that you meet the intent of the GDCs for CRH. Your commitment to implement TS changes based on TSTF-448, following our formal review and approval of TSTF-448, is acceptable for the purpose of closing out your response to GL 2003-01.

The NRC staff has completed its review of your response to GL 2003-01 and finds it acceptable. The NRC staff looks forward to working with the Southern Nuclear Operating Company, Inc., staff to implement the provisions of TSTF-448.

Please contact me if you have any questions regarding this correspondence.

Sincerely,

/RA/

Christopher Gratton, Sr. Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-321 and 50-366

cc: See next page

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Christopher Gratton, Sr. Project Manager
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 Office of Nuclear Reactor Regulation

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