

September 20, 2006

Mr. H. L. Sumner, Jr.
Vice President - Farley Project
Southern Nuclear Operating Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

SUBJECT: NRC RECEIPT OF JOSEPH M. FARLEY NUCLEAR PLANT UNITS 1 AND 2,
RESPONSE TO GENERIC LETTER 2003-01 "CONTROL ROOM
HABITABILITY" (TAC NOS. MB9802 AND MB9803)

Dear Mr. Sumner:

The Nuclear Regulatory Commission (NRC) acknowledges the receipt of your responses to Generic Letter (GL) 2003-01 "Control Room Habitability" dated August 4, 2003 (ML032190245) and August 25, 2004 (ML042400122) for the Farley Nuclear Plant, Units 1 and 2, (Farley). This letter provides the status of our review of your response and describes any actions that may be necessary to consider your response to GL 2003-01 complete.

The GL requested that you confirm that your control rooms meet their design bases (e.g., General Design Criteria (GDC) 1, 3, 4, 5, & 19, draft GDC, or principal design criteria), with special attention to: (1) Determination of the most limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in your design bases for meeting control room operator dose limits from accidents (GL item 1a); (2) Determination that the most limiting unfiltered inleakage is incorporated into your hazardous chemical assessments; and (3) Determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL item 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability, and plans to retire them (GL item 2).

You reported the results of ASTM E741, (American Society for Testing Materials, Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution), tracer gas tests for the Farley control room which is pressurized for accident mitigation. You determined that the maximum tested unfiltered inleakage into the Control Room Envelope (CRE) was 25 cubic feet per minute (cfm), which is less than the unfiltered inleakage value of 53 cfm assumed in your latest design basis radiological dose calculation.

You also reported that the maximum tested inleakage value of 87 cfm into the CRE is less than the value of 2350 cfm incorporated into the hazardous chemical assessments, and that reactor control capability is maintained from either the control room or the alternate shutdown panel in the event of smoke.

The GL further requested that you assess your Technical Specifications (TS) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design basis analysis for control room habitability, in light of the demonstrated inadequacy of a differential pressure measurement alone to provide such verification (GL Item 1.c). The

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NRC staff noted that you have recently revised your TS to incorporate inleakage surveillance based on the model TS and TS Bases contained in NRC Regulatory Guide 1.196, "Control Room Habitability at Light-Water Nuclear Power Reactors".

The information you provided also confirms that no compensatory measures are required to demonstrate control room habitability and that Farley conforms to the GDC regarding control room habitability.

Based on your responses, the NRC staff finds that no additional actions are required and considers your response to GL 2003-01 to be complete.

Sincerely,

/RA/

Robert E. Martin, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

cc w/encl: See next page

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Joseph M. Farley Nuclear Plant, Units 1 & 2

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