

September 8, 2006

LICENSEE: AmerGen Energy Company, LLC

FACILITY: Oyster Creek Nuclear Generating Station

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALLS HELD ON JULY 6, AND JULY 7, 2006, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND AMERGEN ENERGY COMPANY, LLC, CONCERNING CLARIFICATION OF REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE OYSTER CREEK NUCLEAR GENERATING STATION, LICENSE RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission staff (the staff), and representatives of AmerGen Energy Company, LLC, held two telephone conference calls on July 6, 2006, and July 7, 2006, to discuss and clarify the staff's requests for additional information concerning the Oyster Creek Generating Station license renewal application. The conference calls were useful in clarifying the intent of the staff.

Enclosure 1 provides a listing of the conference call participants. Enclosure 2 contains a summary of the information discussed with the applicant, including a brief description on the status of the items discussed.

The applicant had an opportunity to comment on this summary.

***/RA/***

Donnie J. Ashley, Project Manager  
License Renewal Branch A  
Division of License Renewal  
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosures:  
As Stated

cc w/encls: See next page

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**LIST OF PARTICIPANTS FOR TELEPHONE CONFERENCE CALLS  
TO DISCUSS THE OYSTER CREEK NUCLEAR GENERATING STATION  
LICENSE RENEWAL APPLICATION**

July 6, and July 7, 2006

**Participants**

Donnie Ashley  
Veronica Rodriguez  
Hansraj Ashar  
Don Warfel  
John Hufnagel  
Ahmed Ouaou

**Affiliations**

U.S. Nuclear Regulatory Commission (NRC)  
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NRC  
AmerGen Energy Company, LLC (AmerGen)  
AmerGen  
AmerGen

**DISCUSSION OF REQUESTS FOR ADDITIONAL INFORMATION  
OYSTER CREEK NUCLEAR GENERATING STATION  
LICENSE RENEWAL APPLICATION**

July 6, and July 7, 2006

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of AmerGen Energy Company, LLC (AmerGen), held telephone conference calls on July 6, 2006, and July 7, 2006, to discuss and clarify the staff's requests for additional information (RAIs) concerning the Oyster Creek Generating Station (Oyster Creek), license renewal application (LRA).

The following topics were discussed during the telephone conference calls:

In the original RAI 3.5-4, the staff needed information on the subject of "Shielding Blocks and Plates," specifically a patented material described as "Permali," for which no aging effects were indicated in Table 3.5.2.1.1. The staff requested the applicant to provide a brief description of the material, and the aging management review results that justified it does not need aging management during the period of extended operation.

The applicant responded with the following information:

Permali consists of vacuum impregnated material based on wood veneers (rosewood) and phenolic resin. The material was provided in Oyster Creek original design in combination with steel blocks to provide neutron shielding around recirculation piping nozzles at biological shield wall penetrations. The material is designed for its operating environment and aging management reviews did not identify aging effects requiring aging management during the period of extended operation.

The staff noted that the hydrogen content in wood veneers would make the material susceptible to neutron radiation, and high temperatures around the penetration could affect the stability of phenolic resin. AmerGen is requested to provide a detailed justification of its aging management review for this material that resulted in the conclusion that no aging management is required for this material. The staff requested AmerGen provide material test reports that show the material is capable of performing its intended function through the period of extended operation, or monitor aging effects of the material.

The applicant stated that they would research the information and provide an appropriate response.

In the original RAI 3.5-6, the staff requested additional information on Oyster Creek's evaluation and actions taken as a result of the through-wall cracking of Fitzpatrick torus that indicated a need for closer examination of the highly restrained and structurally discontinuous areas subjected to operational cyclic loads. The primary aging management program (AMP) used

ENCLOSURE 2

for managing degradation of the primary containment structure is Subsection IWE (AMP B.1.27). The program is focused towards detecting loss of material. The staff requested the applicant to discuss how the program would detect initiation of such cracking in the Oyster Creek primary containment.

The applicant responded with the following information:

The Oyster Creek ASME Section XI, Subsection IWE aging management program (B.1.27) is not credited for managing crack initiation and growth. The program is based on visual examinations that may not detect cracking experienced at Fitzpatrick. However crack initiation and growth mechanism experienced at Fitzpatrick is not applicable to Oyster Creek.

Since the initial review, NRC issued Information Notice 2006-01, Torus Cracking in Boiling-Water Reactor (BWR) Mark I Containment, on January 12, 2006, to alert licensees of Fitzpatrick condition. Exelon is currently reviewing the impact of the Fitzpatrick experience on Oyster Creek. Corrective actions will be initiated if it is determined the condition described in the information notice is applicable to Oyster Creek.

The staff noted that loads similar to those generated by the Fitzpatrick event are also generated during safety/relief valve (SRV) discharges in Mark I containments. The staff requested AmerGen to provide the results of its current review of the event and a explanation of why loads from SRV discharges won't be a concern for Oyster Creek.

Note to: AmerGen Energy Company, LLC, from Donnie Ashley dated September 8, 2006.

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CLARIFICATION OF REQUESTS FOR ADDITIONAL INFORMATION  
PERTAINING TO THE OYSTER CREEK NUCLEAR GENERATING STATION,  
LICENSE RENEWAL APPLICATION

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