

October 20, 2006

Mr. James Scarola, Vice President
Brunswick Steam Electric Plant
Carolina Power & Light Company
Post Office Box 10429
Southport, North Carolina 28461

SUBJECT: BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2 - NRC RECEIPT
OF RESPONSE TO GENERIC LETTER 2003-01 "CONTROL ROOM
HABITABILITY" (TAC NOS. MB9779 AND MB9780)

Dear Mr. Scarola:

The Nuclear Regulatory Commission (NRC) acknowledges the receipt of your responses to Generic Letter (GL) 2003-01, "Control Room Habitability," in letters dated August 11, 2003 (Agencywide Document and Access Management System (ADAMS) Accession No. ML032310430), December 9, 2003 (ML033500350), and July 29, 2004 (ML042170286). This letter provides a status of your response and describes any additional information that may be necessary to consider your response to GL 2003-01 complete.

The GL requested that you confirm that your control room meet its design bases (e.g., General Design Criteria (GDC) 1, 3, 4, 5, and 19, draft GDC, or principal design criteria), with special attention to: (1) Determination of the most limiting unfiltered and/or filtered inleakage into the control room and comparison to values used in your design bases for meeting control room operator dose limits from accidents (Item 1(a)), (2) Determination that the most limiting unfiltered inleakage is incorporated into your hazardous chemical assessments, and (3) Determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (Item 1(b)). The GL further requested information on any compensatory measures in use to demonstrate control room habitability (CRH), and plans to retire them (Item 2).

You reported the results of American Society for Testing and Materials, ASTM E741, tracer gas tests for the Brunswick Steam Electric Plant (BSEP), Units 1 and 2, control room. For BSEP, you determined that the tested value for inleakages into the control room envelope (CRE) is 762 +/- 42 cubic feet per minute (cfm) for toxic gas and 869 (+/- 146) cfm for radiation protection, which is less than the values of 2000 cfm (toxic gas mode) and 10,000 cfm (radiation protection mode) that were assumed in the design basis analysis for CRH.

You also provided information that adequately supported a conclusion that reactor control capability is maintained from either the control room or the alternate shutdown panel in the event of smoke.

The GL further requested that you assess your technical specifications to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design basis analysis for CRH, and in light of the demonstrated inadequacy of a delta (Δ) P measurement to alone provide such verification (GL Item 1(c)). As permitted by the GL, you provided a schedule for revising the surveillance requirement in the TS to reference an acceptable surveillance methodology. In your December 9, 2003, response you indicated that you will submit a license amendment request to revise your TS utilizing TSTF-448 within 6 months of the approval of TSTF-448 by the NRC.

The information you provided also supported the fact that there are no compensatory measures in place to demonstrate control room habitability.

The information you provided also supported the conclusion that you are committed to meet the intent of the GDC regarding control room habitability, which is documented in your Safety Evaluation.

Your commitment to submit a license amendment request based on Technical Specification Task Force (TSTF) Traveler TSTF-448, following our formal review and approval, is acceptable for purposes of closing out your response to GL 2003-01. The NRC staff will monitor submittal of the license amendment request and interact with you as necessary during the amendment process.

If you have any questions regarding this correspondence, please contact me.

Sincerely,

/RA/

Brenda Mozafari, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-325 and 50-324

cc: See next page

The GL further requested that you assess your technical specifications to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design basis analysis for CRH, and in light of the demonstrated inadequacy of a delta (Δ) P measurement to alone provide such verification (GL Item 1(c)). As permitted by the GL, you provided a schedule for revising the surveillance requirement in the TS to reference an acceptable surveillance methodology. In your December 9, 2003, response you indicated that you will submit a license amendment request to revise your TS utilizing TSTF-448 within 6 months of the approval of TSTF-448 by the NRC.

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Brenda Mozafari, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-325 and 50-324

cc: See next page

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Brunswick Steam Electric Plant
Units 1 and 2

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