

October 11, 2006

Mr. Thomas D. Walt, Vice President  
Carolina Power & Light Company  
H. B. Robinson Steam Electric Plant Unit No. 2  
3581 West Entrance Road  
Hartsville, South Carolina 29550

SUBJECT: H. B. ROBINSON STEAM ELECTRIC PLANT UNIT 2 - NRC RECEIPT OF  
RESPONSE TO GENERIC LETTER 2003-01, "CONTROL ROOM  
HABITABILITY" (TAC NO. MB9848)

Dear Mr. Walt:

The Nuclear Regulatory Commission (NRC) acknowledges the receipt of your response to Generic Letter (GL) 2003-01, "Control Room Habitability," in a letter dated December 08, 2003 (Agencywide Document and Access Management System (ADAMS) Accession No. ML033440212). This letter provides a status of your response and describes any additional information that may be necessary to consider your response to GL 2003-01 complete.

GL 2003-01 requested that you confirm that your control room meet its design bases (e.g., General Design Criteria (GDC) 1, 3, 4, 5, and 19, draft GDCs, or principal design criteria), with special attention to: (1) Determination of the most limiting unfiltered and/or filtered leakage into the control room and comparison to values used in your design bases for meeting control room operator dose limits from accidents (Item 1.a), (2) Determination that the most limiting unfiltered leakage is incorporated into your hazardous chemical assessments, and (3) Determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (Item 1.b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability (CRH), and plans to retire them (Item 2).

Your response referenced an April 10, 2003, letter (ADAMS Accession No. ML031050027) in which you reported the results of ASTM E741 (American Society for Testing Materials, Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution) tracer gas tests for the pressurized H. B. Robinson Steam Electric Plant, Unit No. 2 control room. You determined that the maximum tested value for leakage into the control room envelope (CRE) was 141 (+/- 5) cubic feet per minute (cfm) for Mode 1; 64 (+/- 15) cfm for Mode 2; 30 (+/- 14) cfm for Mode 3. These values were all less than the values of 160 cfm loss-of-coolant accident (LOCA)-Mode 1) and 290 cfm (non-LOCA-Mode 1); 160 cfm (Mode 2); and 90 cfm (Mode 3) assumed in the design basis radiological dose analyses for CRH.

In your April 10, 2003, letter, you also reported that the maximum tested leakage value for the hazardous chemical recirculation mode was 93 (+/-3) cfm, which is less than the total intake flow of 570 cfm assumed in your design basis hazardous chemical analysis. You indicated that reactor control capability is maintained from either the control room or the alternate shutdown location in the event of smoke.

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The GL further requested that you assess your technical specifications to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the design basis analysis for control room habitability, and in light of the demonstrated inadequacy of a delta ( $\Delta$ ) P (pressure change) measurement to alone provide such verification (Item 1.c). As permitted by the GL, you provided a schedule for revising the surveillance requirement in the technical specification (TS) to reference an acceptable surveillance methodology. In your December 9, 2003, response you indicated that you will submit a license amendment request to revise your TS utilizing Technical Specification Task Force (TSTF) Traveler TSTF-448 within 6 months of the approval of TSTF-448 by the NRC.

The information you provided also supported the fact that all compensatory measures needed to demonstrate CRH have been retired following the July 11, 2006, NRC approval of your LOCA Alternate Source Term submittal (ADAMS Accession No. ML061910159).

The information you provided also supported the fact that there are no compensatory measures in place to demonstrate control room habitability.

The information you provided also supported the conclusion that you are committed to meet the intent of the GDC regarding control room habitability which is documented in your Safety Evaluation.

Your commitment to submit a license amendment request based on TSTF-448, following our formal review and approval, is acceptable for purposes of closing out your response to GL 2003-01. The NRC staff will monitor submittal of the license amendment request and interact with you as necessary during the amendment process.

If you have any questions regarding this correspondence, please contact me.

Sincerely,

*/RA/*

Brenda Mozafari, Senior Project Manager  
Plant Licensing Branch II-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-261

cc: See next page

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*/RA/*

Brenda Mozafari, Senior Project Manager  
Plant Licensing Branch II-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-261

cc: See next page

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