

September 18, 2006

Mr. Paul D. Hinnenkamp
Vice President - Operations
Entergy Operations, Inc.
River Bend Station
5485 US Highway 61N
St. Francisville, LA 70775

SUBJECT: RIVER BEND STATION, UNIT 1 - RE: RESPONSE TO GENERIC LETTER
2003-01, "CONTROL ROOM HABITABILITY" (TAC NO. MB9847)

Dear Mr. Hinnenkamp:

The U.S. Nuclear Regulatory Commission (NRC) acknowledges the receipt of your responses, dated August 11, 2003 (Agencywide Documents and Access Management System (ADAMS) Accession No. ML032270447), and January 25, 2005 (ADAMS Accession No. ML050310352), to the NRC Generic Letter (GL) 2003-01, "Control Room Habitability," dated June 12, 2003 (ADAMS Accession No. ML031620248).

The GL 2003-01 requested that you confirm that your control room (CR) meets its design bases (e.g., General Design Criteria (GDC) 1, 3, 4, 5, and 19, draft GDC, or principal design criteria), with special attention to the following:

- (1) GL 2003-01, "Requested Information," Item 1a: Determination of the most limiting unfiltered and/or filtered inleakage into the CR and comparison to values used in your design bases for meeting CR operator dose limits from accidents.
- (2) GL 2003-01, "Requested Information," Item 1b: Determination that the most limiting unfiltered inleakage is incorporated into your hazardous chemical assessments.
- (3) GL 2003-01, "Requested Information," Item 1b: Determination that reactor control capability is maintained in the CR or at the alternate shutdown location in the event of smoke.
- (4) GL 2003-01, "Requested Information," Item 2: Provide information on any compensatory measures in use to demonstrate control room habitability (CRH), and plans to retire them.
- (5) GL 2003-01, "Requested Information," Item 1c: Assessment of the Technical Specifications (TSs) to determine if they require verification of the integrity of the CR Envelope (CRE), including ongoing verification of the inleakage assumed in the design basis analysis for CRH, in light of the demonstrated unreliability of the "ΔP surveillance" (a test to ensure positive differential pressure between CR and adjacent areas) alone to provide such verification.

As requested by GL 2003-01, you provided responses by letters referenced above. They are summarized below:

- a. River Bend Station's (RBS's) CR is a positive-pressure-design structure.
- b. The results of American Society of Testing Materials (ASTM) E741 tracer gas tests (ASTM, Standard Test Method, E2029-99, for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution) for RBS's CR indicate that the maximum tested unfiltered inleakage into the CRE, was 18 cubic feet per minute (cfm) which is less than the value of 300 cfm, the assumed value in the design-basis radiological analyses for CRH.
- c. The maximum tested unfiltered inleakage for toxic chemical operation mode was 18 cfm which is less than the value of 300 cfm, the assumed value in your design-basis toxic chemical analysis.
- d. The reactor control capability is maintained from either the CR or the alternate shutdown panel in the event of smoke.
- e. RBS's TSs presently contain a surveillance requirement (SR), SR 3.7.2.4, to demonstrate that the ventilation system can maintain the CRE at a positive pressure relative to outside atmosphere.
- f. RBS would submit a license amendment request (LAR) to include a new TS SR referencing an acceptable surveillance methodology to determine inleakage using ASTM E741 tracer gas or other suitable inleakage testing, within 6 months following the NRC approval of the traveler for TS Task Force (TSTF)-448, "Control Room Habitability," or the publication of the notice of availability of Consolidated Line Item Improvement for TSTF-448 in the *Federal Register*.

The LAR will include the addition of a new TS SR to determine inleakage in accordance with the Control Room Integrity Program (CRIP), and the addition of a new TS Section 5.5, "Programs and Manuals," that will specify the scope of the CRIP. The CRIP will rely on the use of ASTM E741 tracer gas or other suitable inleakage testing. RBS does not anticipate that modifications to the CRE will be required to demonstrate compliance with new SRs.

- g. There are no compensatory measures in place at RBS, to demonstrate CRH.
- h. RBS's Updated Safety Analysis Report requires compliance with the GDCs mentioned above, for CRH, except that TS Amendment No. 132 revised the dose limit requirements for the main CR following a design-basis accident from GDC 19 to those in Title 10 of the *Code of Federal Regulations*, Section 50.67.

P. Hinnenkamp

-3-

Based on the information provided, the regulatory commitment made in your response letters referenced above, and the above discussion, the NRC staff finds the RBS responses to GL 2003-01 to be complete.

If you have any questions regarding this correspondence, please contact me.

Sincerely,

/RA/

Bhalchandra Vaidya, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-458

cc: See next page

P. Hinnenkamp

-3-

Based on the information provided, the regulatory commitment made in your response letters referenced above, and the above discussion, the NRC staff finds the RBS responses to GL 2003-01 to be complete.

If you have any questions regarding this correspondence, please contact me.

Sincerely,

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| OFFICE | NRR/LPLIV/PM | NRR/LPLIV/LA | NRR/SCVB/BC | NRR/PGCB/BC | NRR/LPLIV/BC |
| NAME | BVaidya | LFeizollahi | RDennig | CJackson | DTerao |
| DATE | 9/11/06 | 9/11/06 | 9/18/06 | 9/18/06 | 9/18/06 |

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River Bend Station

cc:

Winston & Strawn LLP
1700 K Street, N.W.
Washington, DC 20006

Manager - Licensing
Entergy Operations, Inc.
River Bend Station
5485 US Highway 61N
St. Francisville, LA 70775

Senior Resident Inspector
P. O. Box 1050
St. Francisville, LA 70775

President of West Feliciana
Police Jury
P. O. Box 1921
St. Francisville, LA 70775

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

Ms. H. Anne Plettinger
3456 Villa Rose Drive
Baton Rouge, LA 70806

Wise, Carter, Child & Caraway
P. O. Box 651
Jackson, MS 39205

Executive Vice President and
Chief Operating Officer
Entergy Operations, Inc.
P. O. Box 31995
Jackson, MS 39286-1995

General Manager - Plant Operations
Entergy Operations, Inc.
River Bend Station
5485 US Highway 61N
St. Francisville, LA 70775

Director - Nuclear Safety
Entergy Operations, Inc.
River Bend Station
5485 US Highway 61N
St. Francisville, LA 70775

Vice President - Operations Support
Entergy Operations, Inc.
P. O. Box 31995
Jackson, MS 39286-1995

Attorney General
State of Louisiana
P. O. Box 94095
Baton Rouge, LA 70804-9095

Brian Almon
Public Utility Commission
William B. Travis Building
P. O. Box 13326
1701 North Congress Avenue
Austin, Texas 78701-3326

Richard Penrod, Senior Environmental Scientist
State Liaison Officer
Office of Environmental Services
Northwestern State University
Russell Hall, Room 201
Natchitoches, LA 71497

May 2006