

September 20, 2006

Mr. Donald K. Cobb
Assistant Vice President - Nuclear Generation
Detroit Edison Company
6400 North Dixie Highway
Newport, MI 48166

SUBJECT: FERMI 2 - RESPONSE TO GENERIC LETTER 2003-01, "CONTROL ROOM HABITABILITY" (TAC NO. MB9804)

Dear Mr. Cobb:

The U.S. Nuclear Regulatory Commission (NRC) acknowledges the receipt of your responses to Generic Letter (GL) 2003-01, "Control Room Habitability," dated August 11, 2003 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML032310500); December 8, 2003 (ADAMS Accession No. ML033510077); and May 25, 2005 (ADAMS Accession No. ML051530174) for the Fermi 2 facility. This letter provides the status of your response.

The GL requested that you confirm that the Fermi 2 control room meets applicable habitability regulatory requirements (e.g., General Design Criterion (GDC) 1, 3, 4, 5, and 19), with special attention to: (1) determination of the most limiting unfiltered and/or filtered inleakage into the control room envelope (CRE) and comparison to values used in your design bases for meeting control room operator dose limits from accidents (GL 2003-01, Item 1a); (2) determination that the most limiting unfiltered inleakage into the CRE is incorporated into your hazardous chemical assessments (GL 2003-01, Item 1b); and (3) determination that reactor control capability is maintained in the control room or at the alternate shutdown location in the event of smoke (GL 2003-01, Item 1b). The GL further requested information on any compensatory measures in use to demonstrate control room habitability, and the plans to retire them (GL 2003-01, Item 1a).

In your letter dated May 25, 2005, you reported the results of the American Society for Testing Material Standard E741, "Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution," tracer gas tests for the Fermi 2 control room which is pressurized for accident mitigation. You determined that the maximum tested inleakage into the CRE was 59 cubic feet per minute (cfm) which is less than the value of 600 cfm assumed in the design-basis radiological analyses for control room habitability.

You indicated that based on your assessments of hazardous chemicals for both offsite stationary and mobile sources and for onsite hazardous chemical sources, no credible scenarios for toxic chemical release to impact control room habitability exists. Therefore, unfiltered inleakage is not incorporated in your design-basis hazardous chemical assessments. You also indicated that reactor control capability is maintained from either the control room or the alternate shutdown panel in the event of smoke.

The GL further requested that you assess your technical specifications (TS) to determine if they verify the integrity of the CRE, including ongoing verification of the inleakage assumed in the

D. Cobb

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design-basis analysis for control room habitability, in light of the demonstrated inadequacy of a delta (Δ) P measurement to alone provide such verification (GL 2003-01, Item 1.c).

In your May 25, 2005, response you stated that License Amendment No. 162 revised your TSs to include adequate requirements for using acceptable methodology to verify the integrity of the CRE and the assumed unfiltered inleakage in the design-basis analysis. You also indicated that you intend to submit a license amendment to remove a license condition contained in License Amendment No. 162 following NRC approval of Technical Specification Task Force Traveler No. 448, "Control Room Habitability."

The information you provided also supported the fact that there are no compensatory measures in place to demonstrate control room habitability. Furthermore, the information you provided also supported the conclusion that you are committed to meet the intent of the GDC regarding control room habitability.

Based on your responses, no additional actions are required and your response to GL 2003-01 is considered complete.

If you have any questions regarding this correspondence, please contact me at 301-415-1439.

Sincerely,

/RA/

David H. Jaffe, Senior Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-341

cc: See next page

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David H. Jaffe, Senior Project Manager
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