

#### C.IV.8 Generic Issues

The requirements of *proposed* 10 CFR 52.79(a)(20) specify that the contents of a combined license (COL) application must include the proposed technical resolutions of those unresolved safety issues (USIs) and medium- and high- priority generic safety issues (GSIs) that are identified in the version of NUREG-0933 current on the date 6 months before the docket date of the application and that are technically relevant to the design. Applicants for design certification have a similar requirement in *proposed* 10 CFR 52.47(a)(18) for addressing USIs and GSIs. This requirement specifies that an applicant for design certification must provide the information necessary to demonstrate technical resolution of those USIs and medium- and high-priority GSIs identified in the NUREG-0933 version that is current on the date 6 months before the docket date of the application. In addition, the USIs and GSIs must be technically relevant to the standard plant design.

Since the inception of the generic issues program in 1976, the NRC has identified and categorized reactor safety issues. These safety issues were grouped into TMI Action Plan Items, Task Action Plan Items, New Generic Items, Human Factors Issues, and Chernobyl Issues and are collectively called Generic Safety Issues (GSIs). A listing of generic issues that are applicable to future reactor plants is provided Appendix B to NUREG-0933. Applicants for design certification and combined licenses should refer to this appendix for the list of generic issues that need to be addressed in their applications. In addition, applicants should address those generic issues for which there is either no entry or “TBD” in the “Future Plants Effective Date” column.

At the time DG-1145 was developed, the benchmark for Appendix B was Supplement 29 to NUREG-0933. Applicants for design certification and combined licenses should review the version of NUREG-0933 and its supplements in effect 6 months prior to the docket date of the application for the generic issues that should be addressed for their specific application. NUREG-0933 and its Supplements may be accessed through the NRC’s web page ([www.nrc.gov](http://www.nrc.gov)).

##### C.IV.8.2 *Operational Experience (Generic Letters and Bulletins)*

The requirements of *proposed* 10 CFR 52.79(a)(37) specify that the contents of COL application must include information which demonstrates how operating experience insights from generic letters and bulletins issued up to 6 months before the docket date of the application, or comparable international operating experience, has been incorporated into the plant design. Applicants for design certification have a similar requirement in *proposed* 10 CFR 52.47(a)(19).

The staff has reviewed the generic letters and bulletins dating back to 1980 and made a preliminary assessment of each document to determine whether it needs to be addressed by COL applicants and/or design certification applicants. The attached table provides the results of the staffs preliminary review. This review will be finalized for the final issuance of this guide. The screening criteria used by the staff for this review is provided as follows:

Screening Criteria:

<u>Exclusion Code</u>	<u>Interpretation</u>
N	N/A to commercial nuclear power plants
P	Procurement issue
A	Administrative issue
M	Maintenance/Installation/Surveillance issue
PR	Procedural/Verification
S	Superseded
X	Not applicable

Generic Letter	Title	Exclusion Code	Notes
80-001	NUREG-0630, "Cladding, Swelling and Rupture Models..."		
80-002	Quality Assurance Requirements Regarding DG Fuel Oil	A	
80-003	BWR Control Rod Failures		BWR only
80-004	IEB 80-01 Operability of ADS Valve Pneumatic Supply		see BL 80-01
80-005	IEB 79-01B Environmental Qualification of Class 1E Equipment		
80-006	Issuance of NUREG-0313, Rev 1, "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure		BWR only
80-007	Information Regarding the Program for Environmental Qualification of Safety-Related Electrical Equipment (Not Issued)	X	Not issued
80-008	IEB 80-02 Inadequate Quality Assurance for Nuclear Supplied Equipment	A	BWR only
80-009	Low Level Radioactive Waste Disposal		
80-010	Issuance of NUREG-0588, "Interim Staff Position on Equipment Qualifications of Safety-Related Electrical Equipment		
80-011	IEB 80-03 Loss of Charcoal from Standard Type II, 2 Inch, Tray Absorber Cells	P	See BL 80-03
80-012	IEB 80-04 Analysis of A PWR Main Steam Line Break With Continued Feedwater Addition		See BL 80-04
80-013	Qualification of Safety-Related Equipment		Note 6
80-014	LWR Primary Coolant System Pressure Isolation Valves		USI B-63
80-015	Request for Additional Management and Technical Resources Information	A	
80-016	IEB 79-01b, Environmental Qualification of Class 1E Equipment	X	Note 3 (A-24)
80-017	Modifications to Boiling Water Reactor Control Rod Drive Systems		BWR only
80-018	Crystal River 3 Reactor Trip from Approximately 100% Full Power		B&W facilities only
80-019	Resolution of Enhanced Fission Gas Release Concerns		
80-020	Actions Req. from OL Applicants of NSSS Designs By W and CE Resulting from NRC B&O Task force Review of TMI-2 Accident		
80-021	IEB 80-05 Vacuum Condition Resulting in Damage to CVCS Holdup Tanks		See BL 80-05
80-022	Transmittal of NUREG-0654 "Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Prepared	A	
80-023	Change of Submittal Date for Evaluation Time Estimates	A	
80-024	Transmittal of Information on NRC "Nuclear Data Link Specifications"	A	
80-025	IEB 80-06, Engineering Safety Features Reset Controls		See BL 80-06; Note 2
80-026	Qualifications of Reactor Operators		
80-027	IEB 80-07 BWR Jet Pump Assembly Failure		BWR only

Generic Letter	Title	Exclusion Code	Notes
80-028	IEB 80-08 Examination of Containment Liner Penetration Welds	M	See BL 80-08; AP1000 has no containment liner
80-029	Modifications to Boiling Water Reactor Control Rod Drive Systems		BWR only
80-030	Clarification of the Term "Operable" as it applies to Single-Failure Criterion for Safety Systems Required by the TSs		Note 5
80-031	IEB 80-09 Hydramotor Actuator Deficiencies	P	See BL 80-09
80-032	Information Request on Category I Masonry Walls Employed By Plants Under CP and OL Review		
80-033	Actions Req. from OL Applicants of B&W Designed NSSS Resulting from NRC B&O Task force Review of TMI-2 Accident	X	B&W facilities only
80-034	Clarification of NRC Requirements for Emergency REsponse Facilities	X	Note 3 (A-24)
80-035	Effect of a DC Power Supply Failure on ECCS Performance		
80-036	IEB 80-10 Contamination of Non-Rad. System and Resulting Potential for Unmonitored, Uncontrolled Release to Environment	A, M	See BL 80-10
80-037	Five Additional TMI-2 Related Requirements to Operating Reactors		
80-038	Summary of Certain Non-Power Reactor Physical Protection Requirements	N	
80-039	IEB 80-11 Masonry Wall Design		See BL 80-11
80-040	Transmit NUREG-0654, Report of the B&O Task force; and Appropriate NUREG-0626, Generic Eval. of FW Transient and SBLOCA...		GE facilities only
80-041	Summary of Meetings Held on April 22 & 23, 1980 With Representatives of the Mark I Owners Group	A	Mark I containment facilities only
80-042	IEB 80-12, Decay Heat Removal System Operability		See BL 80-12; Note 2
80-043	IEB 80-13 Cracking in Core Spray Spargers		BWR only
80-044	Reorganization of Functions and Assignments Within ONRR/SSPB	N	
80-045	Fire Protection Rule		Notes 6 and 7
80-046	Generic Technical Activity A-12 Fracture toughness		USI A-12
80-047	Additional Guidance on "Potential for Low Fracture toughness and Lamellar Tearing on PWR SG and Reactor Coolant Pump Supports"		USI A-12
80-048	Revision to May 19, 1980, Letter on Fire Protection (80-045)		See GL 80-45; Note 7
80-049	Nuclear Safeguards Problems	A	
80-050	Generic Activity A-10 BWR Cracks (USI A-10)		BWR only
80-051	On-Site Storage of Low-Level Waste	A	
80-052	Five Additional TMI-2 Related Requirements - Errata Sheets to 5/7/80 Letter	A	

Generic Letter	Title	Exclusion Code	Notes
80-053	Decay Heat Removal Capability		
80-054	IEB 80-14 Degradation of Scram Discharge Volume Capability		BWR only
80-055	IEB 80-15 Possible Loss of Hotline With Loss of off-Site Power	A	Cover letter for BL 80-15
80-056	Commission Memo and Order on Equipment Qualification	X	Note 4
80-057	Further Commission Guidance for Power Reactor Operating Licenses NUREG-0660 and NUREG-0694		
80-058	IEB 80-16 Potential Misapplication of Rosemount Inc. Models 1151/1152 Pressure Transmitters With "A" Or "D" Output Codes	P; M	See BL 80-16
80-059	Transmittal of Federal Register Notice RE Regional Meetings to Discuss Environmental Qualification of Elec. Equipment	N	
80-060	Request for Information Regarding Evacuation Times	A	
80-061	TMI-2 Lessons Learned		
80-062	TMI-2 Lessons Learned		BWR only
80-063	IEB 80-17 Failure of Control Rods to Insert During A Scram at A BWR	A	BWR only; cover letter for BL 80-17
80-064	Scram Discharge Volume Designs		BWR only
80-065	Request for Estimated Construction Completion and Fuel Load Schedules	A	
80-066	IEB 80-17 Supplement 1 Failure of Control Rods to Insert During A Scram at A BWR		BWR only; see BL 80-17
80-067	Scram Discharge Volume		BWR only
80-068	IEB 80-17 Supplement 2 Failures Revealed By Testing Subsequent to Failure of Control Rods to Insert During A Scram at A	A	BWR only; cover letter for BL 80-17
80-069	IEB 80-18 Maintenance of Adequate Minimum Flow Through Centrifugal Charging Pumps Following Secondary Side HELB	A	Cover letter for BL 80-18
80-070	IEB 80-19 Failures of Mercury-Wetted Matrix Relays in RPS of Operating Nuclear Power Plants Designed By GE	A	Cover letter for BL 80-19
80-071	IEB 80-20 Failures of Westinghouse Type W-2 Spring Return to Neutral Control Switches	A	Westinghouse only; cover letter for BL 80-20
80-072	Interim Criteria for Shift Staffing		
80-073	Functional Criteria for Emergency Response Facilities, NUREG-0696	A	
80-074	Notice of forthcoming Meeting With Representatives of EPRI to Discuss Program for Resolution of USI A-12, Fracture toughness	A	

Generic Letter	Title	Exclusion Code	Notes
80-075	Lessons Learned Tech Specs	A	
80-076	Notice of forthcoming Meeting With GE to Discussed Proposed BWR Feedwater Nozzle Leakage Detection System	A	BWR only; see A-10
80-077	Refueling Water Level		
80-078	Mark I Containment Long-Term Program		BWR Mark I containment only
80-079	IEB 80-17 Supplement 3 Failures Revealed By Testing Subsequent to Failure of Control Rods to Insert During a Scram at a BWR		BWR only
80-080	Preliminary Clarification of TMI Action Plan Requirements		
80-081	Preliminary Clarification of TMI Action Plan Requirements - Addendum to 9/5/80 Letter		
80-082	IEB 90-01b, Supplement 2, EQ of Class 1E Equipment	X	Note 3 (A-24)
80-083	Environmental Qualification of Safety-Related Equipment		
80-084	BWR Scram System		BWR only
80-085	Implementation of Guidance from USI A-12, Potential for Low Fracture toughness and Lamellar Tearing on Component Supports		USI A-12
80-086	Notice of Meeting to Discuss Final Resolution of USI A-12	A	
80-087	Notice of Meeting to Discuss Status of EPRI-Proposed Resolution of the USI A-12 Fracture toughness Issue	A	
80-088	Seismic Qualification of AFW Systems		
80-089	IEB 79-01B Supplement 3 Environmental Qualification of Class 1E Equipment		
80-090	Post TMI Requirements, NUREG-0737		
80-091	ODYN Code Calculation		BWR only
80-092	IEB 80-21 Valve Yokes Supplied By Malcolm Foundry Company, Inc.	P	
80-093	Emergency Preparedness		
80-094	Emergency Plan	A	
80-095	Generic Activity A-10		BWR only
80-096	Fire Protection		
80-097	IEB 80-23 Failures of Solenoid Valves Manufactured By Valcor Engineering Corporation	A	Cover letter for BL 80-23
80-098	IEB 80-24, Prevention of Damage Due to Water Leakage Inside Containment	A	Cover letter for BL 80-24; Note 2
80-099	TS Revision for Snubber Surveillance		
80-100	Appendix R to 10 CFR Part 50 Regarding Fire Protection		Note 6
80-101	Inservice Inspection Programs	A	

Generic Letter	Title	Exclusion Code	Notes
80-102	Commission Memo and Order of May 23, 1980 (Referencing IEB 79-01b, Supp. 2)	X	Note 3 (A-24)
80-103	Fire Protection - Revised Federal Register Notice	A	
80-104	Orders on Environmental Qualification of Safety Related Electrical Equipment	A	
80-105	Implementation of Guidance for USI A-12, Potential for Low Fracture toughness and Lamellar Tearing on Component Supports		USI A-12
80-106	Report on ECCS Cladding Models, NUREG-0630	A	Note 9
80-107	BWR Scram Discharge System		BWR only
80-108	Emergency Planning	A	
80-109	Guidelines for SEP and Soil Structure Interaction Reviews		
80-110	Periodic Updating of FSARs	A	
80-111	IEB 80-17 Supplement 4, Failure of Control Rods to Insert During A Scram at a BWR	A	Cover letter for BL 80-17; BWR only
80-112	IEB 80-25 Operating Problems With Target Rock Safety Relief Valves	A	Cover letter for BL 80-25
80-113	Control of Heavy Loads		USI A-36
81-001	Qualification of Inspection, Examination, Testing and Audit Personnel (Reg. Guide 1.58)		RG 1.58 was withdrawn
81-002	Analysis, Conclusions and Recommendations Concerning Operator Licensing	A	
81-003	Implementation of NUREG-0313, Rev. 1, Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping		BWR only
81-004	Emergency Procedures and Training for Station Blackout Events		USI A-44
81-005	Information Regarding the Program for Environmental Qualification of Safety-Related Electrical Equipment		USI A-24
81-006	Periodic Updating of Final Safety Analysis Reports (FSARs)	A	
81-007	Control of Heavy Loads		USI A-36
81-008	ODYN Code		GE BWR only
81-009	BWR Scram Discharge System		BWR only
81-010	Post-TMI Requirements for the Emergency Operations Facility	A	
81-011	BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking (NUREG-0619)		BWR only
81-012	Fire Protection Rule (45 FR 76602, 11/19/80)		
81-013	SER for GEXL Correlation for 8X8R Fuel Reload Applications for Appendix D Submittals of the GE Topical Report		BWR only

Generic Letter	Title	Exclusion Code	Notes
81-014	Seismic Qualification of AFW Systems		
81-015	Environmental Qualification of Class 1E Electrical Equipment-Clarification of Staff's Handling of Proprietary Information	A	
81-016	NUREG-0737 Item I.C.1 SER on Abnormal Transient Operating Guidelines (ATOG)		B&W facilities only
81-017	Functional Criteria for Emergency Response Facilities	X	Note 3
81-018	BWR Scram Discharge System-Clarification of Diverse Instrumentation Requirements		BWR only
81-019	Thermal Shock to Reactor Pressure Vessels		USI A-49
81-020	Safety Concerns Associated with Pipe Breaks in the BWR Scram System		BWR only
81-021	Natural Circulation Cooldown	A	
81-022	Engineering Evaluation of the H.B. Robinson Reactor Coolant System Leak on 1/29/81	A	No action defined
81-023	INPO Plant Specific Evaluation Reports	A	
81-024	Multi-Plant Issue B-56 Control Rods Fail to Fully Insert		BWR only
81-025	Change in Implementing Schedule for Submission and Evaluation of Upgraded Emergency Plans	A	
81-026	Licensing Requirements for Pending Construction Permit and Manufacturing License Applications		
81-027	Privacy and Proprietary Material in Emergency Plans	A	
81-028	Steam Generator Overfill		USI A-47; PWR only
81-029	Simulator Examinations	A	
81-030	Safety Concerns Associated With Pipe Breaks in the BWR Scram System		BWR only
81-031	Small Break LOCA Confirmatory Integral Systems Experiments for B&W Designed Plants (Never issued)	X	Not issued
81-032	NUREG-0737, Item II.K.3.44-Evaluation of Anticipated Transients Combined With Single Failure		BWR only
81-033	Technical Specification for Station Batteries Multiplant Action (Never issued)	X	Not issued
81-034	Safety Concerns Associated With Pipe Breaks in the BWR Scram System		BWR only
81-035	Safety Concerns Associated With Pipe Breaks in the BWR Scram System		BWR only
81-036	Revised Schedule for Completion of TMI Action Plan Item II.D.1, Relief and Safety Valve Testing	A	
81-037	ODYN Code Reanalysis Requirements		BWR only
81-038	Storage of Low-Level Radioactive Wastes at Power Reactor Sites	A	
81-039	NRC Volume Reduction Policy		
81-040	Qualifications of Reactor Operators	A	

Generic Letter	Title	Exclusion Code	Notes
82-001	New Applications Survey	A	
82-002	Commission Policy on Overtime	A	
82-003	High Burnup MAPLHGR Limits		GE BWR only
82-004	Use of INPO SEE-IN Program	A	See TMI action plan item I.C.5
82-005	Post-TMI Requirements	A	
82-006	RTD Response Time Determination [never issued]	X	Not issued
82-007	Transmittal of NUREG-0909 Relative to the Ginna Tube Rupture	A	BWR only
82-008	Transmittal of NUREG-0909 Relative to the Ginna Tube Rupture	A	Note 1
82-009	EQ of Safety-Related Electrical Equipment	X	Note 3 (A-24)
82-010	Post-TMI Requirements	A	
82-011	Transmittal of NUREG-0916 Relative to the Restart of R.E. Ginna Nuclear Power Plant	A	
82-012	Nuclear Power Plant Staff Working Hours	A	
82-013	Reactor Operator and Senior Reactor Operator Examinations	A	
82-014	Submittal of Documents to the NRC	A	
82-015	Nuclear Plant Staff Working Hours (Not issued)	X	Not issued
82-016	NUREG-0737 Technical Specifications		
82-017	Inconsistency of Requirments Between 50.54(T) and 50.15	X	Note 4
82-018	Reactor Operator and Senior Reactor Operator Regualification Examinations	A	
82-019	Submittal of Copies of Documentation to NRC-Copy Requirements for Emergency Plans and Physical Security Plans	N	
82-020	Guidance for Implementation of the SRP	X	Note 4
82-021	Technical Specifications for Fire Protection Audits		
82-022	Congressional Request for Information Concerning Steam Generator Tube Integrity		USI A-3
82-023	Inconsistency Between Requirements of 10 CFR 73.40(D) and Standard TSs		
82-024	Safety Relief Valve Quencher Loads: BWR MARK II and III Containments		BWR only
82-025	Integrated IAEA Exercise for Physical Inventory at LWRs	A	
82-026	NUREG-0744, Rev. 1 - Pressure Vessel Material Fracture Toughness		USI A-11
82-027	Transmittal of NUREG-0763, "Guidelines for Confirmatory In-Plant Tests of Safety-Relief Valve Discharges for BWR Plants," and NUREG-0783		BWR only
82-028	Inadequate Core Colling Instrumentation System	X	Note 3
82-029	NUREG/CR-2980	X	Not issued
82-030	Filings Related to 10 CFR 50 Production and Utilization Facilities	A	
82-031	NUREG-0737 Technical Specifications	X	Not issued

Generic Letter	Title	Exclusion Code	Notes
82-032	Draft Steam Generator Report [SAIC]		USI A-3
82-033	Supplement 1 to NUREG-0737-Emergency Response Capabilities	X	Note 3
82-034	Secy 82-111 Modification of Vacuum Breakers & Mark I Containments	X	Not issued
82-035	Never issued	X	Not issued
82-036	Assignment of Authority to Regions for Nonpower Reactors Amendments and Licensing Activities (Not issued)	X	Not issued
82-037	NUREG-0737 Tech Specs	X	Not issued
82-038	Meeting to Discuss Developments for Operator Licensing Examinations	A	
82-039	Problems with Submittals of 10 CFR 73.21 Safeguards Info	A	
83-001	Operator Licensing Examination Site Visit	A	
83-002	NUREG-0737 Technical Specifications		BWR only
83-003	Regulatory Guide 1.150, "Ultrasonic Testing of Reactor Vessel Welds During Pre-Service and Inservice Examinations" (Not issued)	X	Not issued
83-004	Regional Workshops Regarding Supplement 1 to NUREG-0737, Requirements for Emergency Response Capability	A	
83-005	Safety Evaluation of "Emergency Procedure Guidelines, Revision 2," NEDO-24934, June 1982		BWR only
83-006	Certificates and Revised Format for Reactor Operator and Senior Reactor Operator Licenses	A	
83-007	The Nuclear Waste Policy Act of 1982	A	Note 6
83-008	Modification of Vacuum Breakers on Mark I Containments		GE BWR only
83-009	Review of Combustion Engineering Owners' Group Emergency Procedures Guideline Program		CE facilities only
83-010	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"		CE facilities only
83-011	Licensee Qualification for Performing Safety Analyses in Support of Licensing Actions		
83-012	Issuance of NRC Form 398 - Personal Qualifications Statement - Licensee	A	
83-013	Clarification of SRs for HEPA filters and Charcoal Adsorber Units in STS and ESF Cleanup Systems		
83-014	Definition of "Key Maintenance Personnel" (Clarification of GL 82-12)	A	See GL 82-12
83-015	Implementation of RG 1.150		
83-016	Transmittal of NUREG-0977 Relative to the ATWS Events at Salem Generating Station, Unit No. 1 (GL 83-028)	A	See GL 83-28

Generic Letter	Title	Exclusion Code	Notes
83-017	Integrity of Requalification Examinations for Renewal of Reactor Operator and Senior Reactor Operator Licenses	A	
83-018	NRC Staff Review of the BWR Owners' Group (BWROG) Control Room Survey Program		BWR only
83-019	New Procedures for Providing Public Notice Concerning Issuance of Amendments to Operating Licenses	A	
83-020	Integrated Scheduling for Implementation of Plant Modifications	A	
83-021	Clarification of Access Control Procedures for Law Enforcement Visits	A	
83-022	Safety Evaluation of "Emergency Response Guidelines"	A	
83-023	Safety Evaluation of "Emergency Procedure Guidelines"		CE facilities only
83-024	TMI Task Action Plan Item I.G.1, "Special Low Power Testing and Training," Recommendations for BWRs		BWR only; see TMI Action Plan Item I.G.1
83-025	Issuance of NRC form 398 Personal Qualifications Statement - Licensee (not issued)	X	Not issued
83-026	Clarification of SRs for Diesel Fuel Impurity Level Tests		RG 1.137
83-027	Surveillance Intervals in Standard TSs		
83-028	Required Actions Based on Generic Implications of Salem ATWS Event	A, M	Note 3
83-029	This Generic Communication was not issued	X	Not issued
83-030	Deletion of STS SR 4.8.1.1.2.d.6 for DG testing		RG 1.108
83-031	Safety Evaluation of "Abnormal Transient Operating Guidelines"		B&W facilities only
83-032	NRC Staff Recommendations Regarding Operator Actions for Reactor Trip and ATWS	A	
83-033	NRC Positions of Certain Requirements of Appendix R		
83-034	Important to Safety (not issued)	X	Not issued
83-035	Clarification of TMI Action Plan Item II.K.3.31		
83-036	NUREG-0737 Technical Specifications		BWR only
83-037	NUREG-0737 Technical Specifications		
83-038	NUREG-0965, "NRC Inventory of Dams"	A	
83-039	Voluntary Survey of Licensed Operators	A	
83-040	Operator Licensing Examination	A	
83-041	Fast Cold Start of Diesel Generator		
83-042	Clarification to GL 81-07 Regarding Response to NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants"		USI A-36
83-043	Reporting Requirements of 10 CFR 50, Sections 50.72 and 50.73, and Standard Technical Specifications	A	
83-044	Availability of NUREG-1021, "Operator Licensing Examiner Standards"	A	

Generic Letter	Title	Exclusion Code	Notes
84-001	NRC Use of the Terms "Important to Safety" and "Safety-Related"	A	Note 4
84-002	Notice of Meeting Regarding Facility Staffing	A	
84-003	Availability of NUREG-0933, "A Prioritization of Generic Safety Issues"	A	
84-004	Safety Evaluation of W Topical Reports on Elimination of Postulated Pipe Breaks in PWRs		Task Action Plan Item A-2 (USI A-2)
84-005	Copy of change to NUREG-1021, "Operator Licensing Examiner Standards"	A	
84-006	Operator and Senior Operator License Examination Criteria for Passing Grade	A	
84-007	Procedural Guidance for Pipe Replacement at BWRs		BWR only
84-008	Interim Procedures for NRC Management of Plant-Specific Backfitting	A	
84-009	Recombiner Capability Requirements of 10CFR50.44(c)(3)(ii)	X	Note 6
84-010	Administration of Operating Tests Prior to Initial Criticality		
84-011	Inspections of BWR Stainless Steel Piping		BWR only
84-012	Compliance with 10CFR Part 61 and Implementation of the RETS and Attendant PCP		
84-013	TSs for Snubbers		
84-014	Replacement and Requalification Training Program	A	
84-015	Proposed Staff Actions to Improve and Maintain DG Reliability		
84-016	Adequacy of On-Shift Operating Experience for Near Term Operating License Applicants	A	Note 7
84-017	Annual Meeting to Discuss Recent Developments Regarding Operator Training, Qualifications, and Examinations	A	
84-018	Filing of Applications for Licenses and Amendments	A	
84-019	Availability of Supplement 1 to NUREG-0933, "A Prioritization of Generic Safety Issues"	A	
84-020	Scheduling Guidance for Licensee Submittals of Reloads That Involve Unreviewed Safety Questions		
84-021	Long Term Low Power Operation in PWRs		PWR only
84-022	10 CFR 20.408 Termination Reports (not issued)	X	Not issued
84-023	Reactor Vessel Water Level Instrumentation in BWRs		BWR only
84-024	Certificate of Compliance to 10CFR50.49, EQ of Electrical Equipment Important to Safety	X	Note 8
85-001	Fire Protection Policy Steering Committee Report	A	
85-002	Staff Recommended Actions ... Regarding Steam Generator Tube Integrity		USI A-3

Generic Letter	Title	Exclusion Code	Notes
85-003	Clarification of Equivalent Control Capacity for Standby Liquid Control Systems		BWR only
85-004	Operating Licensing Examinations	A	
85-005	Inadvertent Boron Dilution Event		
85-006	Quality Assurance Guidance for ATWS Equipment That is Not Safety-Related		USI A-9
85-007	Implementation of Integrated Schedules for Plant Modifications	A	
85-008	10 CFR 20.408 Termination Reports - format	A	
85-009	Technical Specifications for Generic Letter 83-28, Item 4.3		
85-010	Technical Specification for Generic Letter 83-28, Items 4.3 and 4.4		B&W facilities only
85-011	Completion of Phase II of "Control of Heavy Loads at Nuclear Power Plants" NUREG-0612		USI A-36
85-012	Implementation of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps		
85-013	Transmittal of NUREG-1154 Regarding the David-Besse Loss of Main and Aux. Feedwater Event	PR	
85-014	Commercial Storage at Power Reactor Sites of Low Level Radioactive Waste Not Generated By the Utility		
85-015	Information on Deadlines for 10 CFR 50.49, "EQ of Electric Equipment Important to Safety at NPPs"	A	
85-016	High Boron Concentrations		
85-017	Availability of Supplements 2 and 3 to NUREG-0933, "A Prioritization of Generic Safety Issues"	A	
85-018	Operator Licensing Examinations	A	
85-019	Reporting Requirements on Primary Coolant Iodine Spikes	A	
85-020	Resolution of Generic Issue 69: High Pressure Injection/Make-up Nozzle Cracking in Babcock and Wilcox Plants		B&W facilities only
85-021	Not Issued	X	Not issued
85-022	Potential for Loss of Post-LOCA Recirculation Capability Due to Insulation Debris Blockage		USI A-43
86-001	Safety Concerns Associated With Pipe Breaks in the BWR Scram System		BWR only
86-002	Technical Resolution of Generic Issue B-19 - Thermal Hydraulic Stability		BWR only
86-003	Applications for License Amendments	A	
86-004	Policy Statement on Engineering Expertise on Shift		

Generic Letter	Title	Exclusion Code	Notes
86-005	Implementation of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps"	X	B&W facilities only; Note 3 (see TMI Action Item II.K.3.5)
86-006	Implementation of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps"	X	CE facilities only; Note 3 (see TMI Action Item II.K.3.5)
86-007	Transmittal of NUREG-1190 Regarding the San Onofre Unit 1 Loss of Power and Water Hammer Event		USI A-1
86-008	Availability of Supplement 4 to NUREG-0933, A Prioritization of Generic Safety Issues"	A	
86-009	Technical Resolution of Generic Issue B-59, (N-1) Loop Operation in BWRs and PWRs		
86-010	Implementation of Fire Protection Requirements		
86-011	Distribution of Products Irradiated in Research Reactors	N	
86-012	Criteria for Unique Purpose Exemption from Conversion from the use of HEU Fuel	N	
86-013	Potential Inconsistency Between Plan Safety Analyses and TSs		
86-014	Operator Licensing Examinations	A	
86-015	Information relating to Compliance with 10CFR50.49	X	Note 8
86-016	Westinghouse ECCS Evaluation Models		
86-017	Availability of NUREG-1169, "Technical Findings Related to Generic Issue C-8, BWR Misc. Leakage and Treatment Methods"		BWR only; see Task Action Item C-8
87-001	Public Availability of the NRC Operator Licensing Examination Question Bank	A	
87-002	Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operating Reactors (USI A-46)		USI A-46
87-003	Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operating Reactors (USI A-46)		USI A-46
87-004	Temporary Exemption from Provisions of the FBI Criminal History Rule for Temporary Workers	A	
87-005	Request for Additional Info (RFI) on Assessment ... Degradation of Mark I Drywells		BWR only
87-006	Periodic Verification of Leak Tight Integrity of Pressure Isolation Valves		
87-007	Information Transmittal of Final Rulemaking for Revisions to Operator Licensing - 10 CFR 55 and Confirming Amendments	A	
87-008	Implementation of 10 CFR 73.55 Miscellaneous Amendments and Search Requirements	A	
87-009	Section 3.0 and 4/0 of STS on LCOs and SRs		

Generic Letter	Title	Exclusion Code	Notes
87-010	Implementation of 10 CFR 73.57, Requirements for FBI Criminal History Checks	A	
87-011	Relaxation in Arbitrary Intermediate Pipe Rupture Requirements		
87-012	Loss of Residual Heat Removal While the RCS is Partially Filled		
87-013	Integrity of Requalification Examinations at Non-Power Reactors	N	
87-014	Operator Licensing Examinations	A	
87-015	Policy Statement on Deferred Plants	X	Deferred facilities only
87-016	Transmittal of NUREG-1262, "Answers to Questions on Implementation of 10 CFR 55 on Operators' Licenses"	A	
88-001	NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping		BWR only
88-002	Integrated Safety Assessment Program II	A	
88-003	Resolution of Generic Safety Issue 93, "Steam Binding of Auxiliary Feedwater Pumps"		GSI 93
88-004	Distribution of Gems Irradiated in Research Reactors	N	
88-005	Boric Acid Corrosion of Carbon Steel Reactor Pressure Boundary Components in PWRs		
88-006	Removal of Organization Charts from Technical Specification Administrative Control Requirements	A	
88-007	Modified Enforcement Policy...10CFR50.49	X	Note 8
88-008	Mail Sent or Delivered to the office of Nuclear Reactor Regulation	A	
88-009	Pilot Testing of Fundamentals Examination	A	BWR only
88-010	Purchase of GSA Approved Security Containers	P	
88-011	NRC Position on Radiation Embrittlement of RV Materials and its Impacts on Operations		USI A-47; RG 1.99
88-012	Removal of Fire Protection Requirements for TSs	A	
88-013	Operator Licensing Examinations	A	
88-014	Instrument Air Supply System Problems Affecting Safety-Related Equipment		
88-015	Electric Power Systems - Inadequate Control Over Design Process		
88-016	Removal of Cycle-Specific Parameter Limits from TSs	A	
88-017	Loss of Decay Heat Removal - 10 CFR 50.54(f)		
88-018	Plant Record Storage on Optical Disks	X	Note 4
88-019	Use of Deadly Force by Licensee Guards to Prevent Theft of Special Nuclear Material	A; PR	
88-020	Individual Plant Examination for Severe Addicent Vulnerabilities		
89-001	Implementation of Programmatic Control for RETS...	A	

Generic Letter	Title	Exclusion Code	Notes
89-002	Actions to Improve the Detection of Counterfeit and Fraudulently Marketed Products	P	
89-003	Operator Licensing National Examination Schedule	A	
89-004	Guidance on Developing Acceptable IST Programs		
89-005	Pilot Testing of the Fundamentals Examination	A	
89-006	Task Action Plan Item I.D.2 - Safety Parameter Display System - 10 CFR 50.54(f)		
89-007	Power Reactors Safetguards Contingency Planning for Surface Vehicle Bombs	A; PR	
89-008	Erosion/Corrosion Induced Pipe Wall Thinning	M	
89-009	ASME Section III Component Replacements	P	
89-010	Safety-Related MOV Testing and Surveillance	M	
89-011	Resolution of Generic Issue 101 "Boiling Water Reactor Water Level Redundancy"		BWR only
89-012	Operator Licensing Examination	A	
89-013	Service Water System Problems Affecting Safety-Related Equipment		
89-014	Line-Item Improvements in TSs...	A	
89-015	Emergency Response Data System	A	
89-016	Installation of a Hardened Wetwell Vent		BWR only
89-017	Planned Administrative Changes to the NRC Operator Licensing Written Examination Process	A	
89-018	Resolution of Unresolved Safety Issue A-17, "Systems Interactions in Nuclear Power Plants"		USI A-47; USI A-17
89-019	Request for Action Related to Resolution of Unresolved Safety Issue A-47, "Safety Implication of Control Systems in LWR Nuclear Power Plants," Pursuant to 10 CFR 50.54(f)		USI A-47, A-27
89-020	Protected Area Long-Term Housekeeping	N	
89-021	Request for Information Concerning Status of Implementation of Unresolved Safety Issue (USI) Requirements	A	
89-022	Potential for Increased Roof Loads and Plant Area Flood Runoff Depth at Licensed Nuclear Power Plants Due to Recent Change in Probable Maximum Precipitation Criteria Developed By the National Weather Service		
89-023	NRC Staff Responses to Questions Pertaining to Implementation of 10 CFR Part 26	A	
90-001	Request for Voluntary Participation in NRC Regulatory Impact Survey	A	
90-002	Alternative Requirements for Fuel Assemblies in the Design Features of TSs		
90-003	Relaxation of Staff Position in Generic Letter 83-28, Item 2.2 Part 2 "Vendor Interface for Safety-Related Components"	P	

Generic Letter	Title	Exclusion Code	Notes
90-004	Request for Information on the Status of Licensee Implementation of GSIs Resolved with Imposition of Requirements or CAs	A	
90-005	Guidance for Performing Temporary Non-Code Repair of ASME Code Class 1, 2, and 3 Piping	M	
90-006	Resolution of Generic Issues 70, "PORV and Block Valve Reliability," and 94, "Additional LTOP Protection for PWRs"		USI A-26
90-007	Operator Licensing National Examination Schedule	A	
90-008	Simulation Facility Exemptions	A	
90-009	Alternative Requirements for Snubber Visual Inspection Intervals and Corrective Actions		
91-001	Removal of the Schedule for the Withdrawal of RV Material Surveillance Specimens from the TSs	A	
91-002	Reporting Mishaps Involving LLW Forms Prepared for Disposal	A	
91-003	Reporting of Safeguards Events	A	
91-004	Changes in TS Surveillance Intervals to Accomodate a 24-month Fuel Cycle	A	
91-005	Licensee Commercial-Grade Procurement and Dedication Programs	P	
91-006	Resolution of Generic Issue A-30, "Adequacy of Safety-Related DC Power Supplies," Pursuant to 10 CFR 50.54(f)		USI A-30
91-007	GI-23, RCP Seal Failure and Its Possible Effect on SBO		USI 23
91-008	Removal of Component Lists from the TSs	A	
91-009	Modification of Surveillance Interval, Electrical Protection Assemblies in Power Supplies, RPS		BWR only
91-010	Explosives Searches at Protected Area Portals	N	
91-011	Resolution of Generic Issues 48, "LCOs for Class 1E Vital Instrument Buses," and 49, "Interlocks and LCOs for Class 1E Tie Breakers" Pursuant to 10 CFR 50.54(f)		USI 48 & 49
91-012	Operator Licensing National Examination Schedule	A	
91-013	Request for Info Related to the Resolution of GI 130, "Essential Service Water System Failures at Multi-Unit Sites"		USI 130
91-014	Emergency Telecommunications	A	
91-015	Operating Experience Feedback Report, Solenoid-Operated Valve Problems	P, M	
91-016	Licensed Operators' and Other Nuclear Facility Personnel Fitness for Duty	A	
91-017	Generic Safety Issue 29, "Bolting Degradation or Failure in Nuclear Power Plants"	P; M	

Generic Letter	Title	Exclusion Code	Notes
91-018	Information ... Regarding Two NRC ... [IPs] on Resolution of Degraded and Nonconforming Conditions and on Operability	A	
91-019	Information to Addressees Regarding New Telephone Numbers for NRC offices Located in one White Flint North	A	
92-001	Reactor Vessel Structural Integrity		
92-002	Resolution of Generic Issue 79, "Unanalyzed Reactor Vessel (PWR) Thermal Stress During Natural Convection Cooldown"	PR	USI 79
92-003	Compilation of the Current Licensing Basis: Request for Voluntary Participation in Pilot Program	A	
92-004	Resolution of the Issues Related to Reactor Vessel Water Level Instrumentation in BWRs Pursuant to 10 CFR 50.54(f)		BWR only
92-005	NRC Workshop on the Systematic Assessment of Licensee Performance (SALP) Program	A	
92-006	Operator Licensing National Examination Schedule	A	
92-007	Office of Nuclear Reactor Regulation Reorganization	A	
92-008	Thermo-Lag 330-1 Fire Barriers (BL-92-001)	P	Note 2
92-009	Limited Participation By NRC in the IAEA International Nuclear Event Scale	A	
93-001	Emergency Response Data System Test Program	A	
93-002	NRC Public Workshop on Commercial Grade Procurement and Dedication	A	
93-003	Verification of Plant Records	A	
93-004	Rod Control System Failure and Withdrawal of RCCAs		
93-005	Line-Item TS Improvements to Reduce SRs for Testing During Power Ops	A	
93-006	Research Results on Generic Safety Issue 106, "Piping and the Use of Highly Combustible Gases in Vital Areas"		GSI 106
93-007	Modification of the TS Admin Control Requirements for Emergency and Security Plans	A	
93-008	Relocation of TS Tables of Instrument Response Time Tables	A	
94-001	Removal of Accelerated Testing and Special Reporting Requirements for Emergency DGs	M	GSI B-56
94-002	Long-Term Solutions and Upgrade of Interim Operating Recommendations for Thermal-Hydraulic Instabilities in BWRs		BWR only
94-003	Intergranular Stress Corrosion Cracking of Core Shrouds in BWRs		BWR only

Generic Letter	Title	Exclusion Code	Notes
94-004	Voluntary Reporting of Additional Occupational Radiation Exposure Data	A	
95-001	NRC Staff Technical Position on Fire Protection for Fuel Cycle Facilities	N	
95-002	Use of NUMARC/EPRI Report TR-102348, "Guideline on Licensing Digital Upgrades," in Determining the Acceptability of Performing Analog-to-Digital Replacements Under 10 CFR 50.59	A	
95-003	Circumferential Cracking of SG Tubes		USI A-3
95-004	Final Disposition of the Systematic Evaluation Program Lessons-Learned Issues	A	
95-005	Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking		
95-006	Changes in the Operator Licensing Program	A	
95-007	Pressure Locking and Thermal Binding of Safety-Related Power-Operated Gate Valves	P	
95-008	10 CFR 50.54(p) Process for Changes to Security Plans Without Prior NRC Approval	PR	
95-009	Monitoring and Training of Shippers and Carriers of Radioactive Materials	PR	
95-010	Relocation of Selected TS Requirements Related to Instrumentation	A	
96-001	Testing of Safety-Related Logic Circuits		
96-002	Reconsideration of Nuclear Power Plant Security Requirements Associated with an Internal Threat	A	
96-003	Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits	A	
96-004	Boraflex Degradation in Spent Fuel Pool Storage Racks	P	
96-005	Periodic Verification of Design-Basis Capability of Safety-Related MOVs		
96-006	Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions		
96-007	Interim Guidance on Transportation of Steam Generators	PR	PWR only
97-001	Degradation of CRDM Nozzle and Other VHPs		See BL 2001-01 and 2002-02
97-002	Revised Contents of the Monthly Operating Report, May 15, 1997	A	
97-003	Annual Financial Surety Update Requirements for Uranium Recovery Licensees, July 9, 1997	N	
97-004	Assurance of Sufficient NPSH for ECCS and Containment Heat Removal Pumps		
97-005	SG Tube Inspection Techniques	M; PR	PWR only

Generic Letter	Title	Exclusion Code	Notes
97-006	Degradation of SG Internals	PR	PWR only
98-001	Year 2000 Readiness of Computer Systems at Nuclear Power Plants, Issued May 11, 1998.	A	
98-002	Loss of Reactor Coolant Inventory and Associated Potential Loss of Emergency Mitigation Functions While in a Shutdown Condition		
98-003	NMSS Licensees' and Certificate Holders' Year 2000 Readiness Programs, 6/22/98	N	
98-004	Potential for Degradation of the ECCS and CSS after a LOCA Because of Construction and Protective Coating Deficiencies and Foreign Material		
98-005	Boiling Water Reactor Licensees Use of the BWRVIP-05 Report to Request Relief from Augmented Examination Requirements on Reactor Pressure Vessel Circumferential Shell Welds, November 10, 1998		BWR only
99-001	Recent Nuclear Material Safety and Safeguards Decision on Bundling Exempt Quantities, 5/3/1999 [Materials]	N	
99-002	Laboratory Testing of Nuclear-Grade Activated Charcoal	PR	
03-001	Control Room Habitability		
04-001	Requirements for steam generator tube inspections		PWR only
04-002	Potential impact of debris blockage on Emergency Recirculation during design basis accidents at PWRs		PWR only
06-001	Steam generator tube integrity and associated technical specifications		PWR only
06-002	Grid reliability and the impact on plant risk and the operability of offsite power		
06-003	Potentially nonconforming Hemyc and MT fire barrier configurations		

**NOTES**

- 1 The issues involved in this GL were incorporated into Issues A-3 through A-5
- 2 The issues involved in this GL are covered in the review of the associated BL
- 3 The issues involved in this GL are covered in the review of a USI or GSI issue, for example a TMI Action Plan issue

Generic Letter	Title	Exclusion Code	Notes
4	The GL is not involved with operational experience		
5	Not used		
6	The GL involved with requirements stated in the CFR or in an SRP Section		
7	The GL concerns were addressed in another GL; therefore does not need to be reviewed again (e.g., 86-004)		
8	The GL involved enforcement of 10CFR 50.49 (EQ) reg; therefore does not need to be reviewed		
9	The GL does not involve requirements: therefore does not need to be reviewed		

Bulletin	Title	Exclusion Code	Notes
80-01	Operability of ADS Valve Pneumatic Supply		BWR only
80-02	Inadequate Quality Assurance for Nuclear Supplied Equipment	P	BWR only
80-03	Loss of Charcoal from Standard Type II, 2 Inch, Tray Adsorber Cells	P	
80-04	Analysis of a PWR Main Steam Line Break with Continued FW Addition		
80-05	Vacuum Condition Resulting in Damage to the CVCS Holdup Tanks		
80-06	Engineered Safety Feature Reset Controls	PR	
80-07	BWR Jet Pump Assembly Failure		BWR only
80-08	Examination of Containment Liner Penetration Welds		
80-09	Hydramotor Actuator Deficiencies	P	
80-10	Contamination of Nonradioactive System and Resulting Potential for..	A; M	
80-11	Masonry Wall Design		
80-12	Decay Heat Removal System Operability		
80-13	Cracking in Core Spray Spargers		BWR only
80-14	Degradation of Scram Discharge Volume Capability		BWR only
80-15	Possible Loss of Emergency Notification System with LOOP	A	
80-16	Potential Misapplication of Rosemount...Models 1151 and 1152 Pressure Transmitters with Either "A" or "D" Output Codes	P; M	
80-17	Failure of 76 of 185 Control Rods to Fully Insert During a Scram at a BWR		BWR only
80-18	Maintenance of Adequate Min Flow Centrifugal Charging Pump....		
80-19	Failures of Mercury-Wetted Matrix Relays in Reactor Protective Systems of...Plants Designed by Combustion Engineering	P	
80-20	Failure of Westinghouse Type W-2 Spring Return to Neutral Control Switches	P	
80-21	Valve Yokes Supplied by Malcolm Foundry Company, Inc.	P	
80-22	Automation Industries, Model 200-520-008 Sealed-Source Connectors	N	
80-23	Failures of Solenoid Valves Manufactured by Valcor Engineering Corporation	P	
80-24	Prevention of Damage Due to Water Leakage Inside Containment		
80-25	Operating Problems with Target Rock Safety-Relief Valves at BWRs		BWR only
81-01	Failure and Surveillance of Mechanical Snubbers		
81-02	Failure of Gate Type Valves to Close Against Differential Pressure	P; A; S	
81-03	Flow Blockage of Cooling Water to Safety System Components...	M	
82-01	Alteration of Radiographs of Welds in Piping Subassemblies	A	
82-02	Degradation of Threaded Fasteners in the RC Pressure Boundary of PWR Plants	P; M	
82-03	Stress Corrosion Cracking in Thick-Wall, Large Diameter, Stainless Steel, Recirculation System Piping at BWR Plants		BWR only
82-04	Deficiencies in Primary Containment Electrical Penetration Assemblies	P	

Bulletin	Title	Exclusion Code	Notes
83-01	Failure of Trip Breakers (Westinghouse DB-50) to Open on Automatic Trip Signal	M	
83-02	Stress Corrosion Cracking in Large-Diameter Stainless Steel Recirculation System Piping at BWR Plants		BWR only
83-03	Check Valve Failures in Raw Water Cooling Systems of DGs		
83-04	Failure of the Undervoltage Trip Function of Reactor Trip Breakers	P	
83-05	ASME Nuclear Code Pumps and Spare Parts Manufactured by the Hayward Tyler Pump Company	P	
83-06	Nonconforming Materials Supplied by Tube-Line Facilities at Long Island City, NY, Houston,TX, & Carol Stream,IL	P	
83-07	Apparently Fraudulent Products Sold by Ray Miller, Inc.	P	
83-08	Elect. Circuit Breakers With Undervoltage Trip...in Safety-Related Applications other than the Reactor Trip System	M	
84-01	Cracks in BWR Mark I Containment Vent Headers		BWR only
84-02	Failure of GE Type HFA Relays In Use In Class 1E Safety Systems	P	
84-03	Refueling Water Cavity Water Seal		
85-01	Steam Binding of Auxiliary Feedwater Pumps		GSI 93
85-02	Undervoltage Trip Attachments of Westinghouse DB-50 Reactor Trip Bkrs.	P; M	
85-03	MOV Common Mode Failures During Plant Transients...	A; M; P	See GL 89-10
86-01	Min. Flow Logic Problems that Could Disable RHR Pumps		
86-02	Static "O" Ring Differential Pressure Switches	P	
86-03	Potential Failure of Multiple ECCS Pumps Due to Single Failure...		
86-04	Defective Teletherapy Timer That May Not Terminate Treatment Dose	N	
87-01	Thinning of Pipe Walls	A; S	
87-02	Fastener Testing to Determine Conformance w Applicable Material Specs.	P	
88-01	Defects in Westinghouse Circuit Breakers	P	
88-02	Rapidly Propagating Fatigue Cracks in SG Tubes		
88-03	Inadequate Latch Engagement in HFA Type Latching Relays Manufactured by General Electric (GE) Company	P	
88-04	Potential Safety-related Pump Loss		
88-05	Nonconforming Materials Supplied by Piping Supplies, Inc. ... and West Jersey Manufacturing Company ...	P	
88-06	Actions To Be Taken for the Transportation of Model No. SPEC 2-T Radiographic Exposure Device	N	
88-07	Power Oscillations in BWRs		BWR only
88-08	Thermal Stress in Piping Connected to RCS		

Bulletin	Title	Exclusion Code	Notes
88-09	Thimble Tube Thinning in Westinghouse Reactors		
88-10	Nonconforming Molded-Case Circuit Breakers	P	
88-11	Pressurizer Surge Line Thermal Stratification		
89-01	Failure of Westinghouse SG Tube Mechanical Plugs	P	PWR only
89-02	Stress Corrosion Cracking of High-Hardness Type 410 .. Bolting .. Anchor Darling S350W .. Swing Check Valves .. Similar	P	
89-03	Potential Loss of Required Shutdown Margin During Refueling Ops	PR	
90-01	Loss of Fill-Oil in Transmitters Manufactured by Rosemount	P	
90-02	Loss of Thermal Margin Caused by Channel Box Bow		BWR only
91-01	Reporting Loss of Criticality Safety Controls	N	
92-01	Failure of Thermo-Lag 330 Fire Barrier System...	P	
92-02	Safety Concerns Related to "End of Life" of Aging Theratronics Teletherapy Units	N	
92-03	Release of Patients after Brachytherapy	N	
93-01	Release of Patients After Brachytherapy Treatment With Remote Afterloading Devices	N	
93-02	Debris Plugging of ECCS Suction Strainers	PR	USI A-43.
93-03	Resolution of Issues Related to Reactor Vessel Water Level Instrumentation in BWRs		BWR only
94-01	Potential Fuel Pool Draindown Caused by Inadequate Maintenance Practices at Dresden Unit 1		Plant specific
94-02	Corrosion Problems in Certain Stainless Steel Packagings Used to Transport Uranium Hexafluoride	P	
95-01	Quality Assurance Program for Transportation of Radioactive Material	P	
95-02	Unexpected Clogging of a RHR Pump Strainer...	PR	
96-01	Control Rod Insertion Problem	P	
96-02	Movement of Heavy Loads over Spent Fuel in the Reactor Core...	PR	USI A-36
96-03	Potential Plugging of ECCS Suction Strainers by Debris	PR	See GSI 191; USI A-43
96-04	Chemical, Galvanic, or Other Reactions in Spent Fuel Storage and Transportation Casks	PR	
97-01	Potential for Erroneous Calibration, Dose Rate, or Radiation Exposure Measurements with Certain Victoreen Model 530 and 530SI Electrometer/Dosemeters	P; A	
97-02	Puncture Testing of Shipping Packages under 10 CFR Part 71	A	
	<i>***no BLs between 1998 and 2000***</i>		
01-01	Circumferential Cracking of RPV Head Penetration Nozzles		

Bulletin	Title	Exclusion Code	Notes
02-01	Reactor Pressure Vessel Head Degradation...		
02-02	Reactor Pressure Vessel Head and Vessel Head Penetration Nozzle Inspection Programs		
03-01	Potential Impact of Debris Blockage on Emergency Sump Recirculation at Pressurized-Water Reactors		USI A-43; See GSI-191
03-02	Leakage from Reactor Pressure Vessel Lower Head Penetrations and Reactor Coolant Pressure Boundary Integrity		
03-03	Potentially Defective 1-inch Valves for Uranium Hexafluoride Cylinders	N	
03-04	Rebaselining of Data in the Nuclear Materials Management and Safeguards System	PR	
04-01	Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Piping Connections at Pressurized-Water Reactors		PWR only
05-01	Material Control and Accounting at Reactors and Wet Spent Fuel Storage Facilities		
05-02	Emergency Preparedness and Response Actions for Security-Based Events		