

August 17, 2006

Mrs. Mary G. Korsnick  
Vice President R.E. Ginna Nuclear Power Plant  
R.E. Ginna Nuclear Power Plant, LLC  
1503 Lake Road  
Ontario, NY 14519

SUBJECT: R.E. GINNA NUCLEAR POWER PLANT - REVIEW OF STEAM GENERATOR  
TUBE INSPECTION SUMMARY REPORT FOR THE SPRING 2005 OUTAGE  
(TAC NO. MD1207)

Dear Mrs. Korsnick:

By letter dated July 1, 2005, as supplemented on July 13, 2006, R.E. Ginna Nuclear Power Plant, LLC (the licensee), submitted its steam generator (SG) tube inspection summary report for the spring 2005 outage at R.E. Ginna Nuclear Power Plant (Ginna).

The Nuclear Regulatory Commission staff has completed its review of the SG tube inspection report. The staff concluded that the licensee provided the information required by the Ginna Technical Specifications and that no additional follow-up is required at this time. The staff's review of the report is enclosed.

If you have any questions, please contact me at 301-415-1457.

Sincerely,

*/RA/*

Patrick D. Milano, Senior Project Manager  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure:  
As stated

cc w/encl: See next page

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STAFF REVIEW OF THE 2005 STEAM GENERATOR TUBE INSPECTION REPORT

R. E. GINNA NUCLEAR POWER PLANT, LLC

R. E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

By letters dated July 1, 2005, and July 13, 2006 (Agencywide Documents Access and Management System Accession Nos. ML051930115 and ML062010230, respectively), R.E. Ginna Nuclear Power Plant, LLC (the licensee), submitted information related to the steam generator (SG) tube inspections during the spring 2005 outage at R.E. Ginna Nuclear Power Plant (Ginna).

The two SGs at Ginna are designated A and B. In spring 1996, the original SGs were replaced with Babcock and Wilcox International replacement SGs. The replacement SGs have 4765 total tubes that have an outside diameter of 0.749 inch and a wall thickness of 0.044 inch. The tubes are thermally-treated Inconel 690 material and have been hydraulically expanded into the tubesheet. The stainless steel Type 410 tube support plates (TSPs) are lattice grid type. The replacement SGs were placed in operation in June 1996.

The licensee provided the scope, extent, methods, and results of its SG tube inspections in the documents referenced above. In addition, the licensee described corrective actions (i.e., tube plugging) taken in response to the inspection findings.

As a result of the review of these reports, the NRC staff has the following observations:

- a. There is one tube in SG B in which the tubesheet hole on the hot leg side had several scratches on the inside diameter. The hole was buffed during fabrication to remove the sharp edges and it was tubed normally. The tube was inspected with a rotating probe during the outage and no degradation was identified. A non-conformance report was written on this tube during manufacture.
- b. The U-bend area of the tube in row 9 column 121 of SG A does not permit the passage of a 0.620-inch diameter bobbin probe. This condition has been identified in every inservice inspection performed since 1997. The licensee stated that the restriction appears to be the result of a ding. The ding was inspected with a rotating probe and no active degradation was found. The licensee does not suspect the restriction to be service-induced due to the location of the ding. The restriction was not noticed during the preservice inspection since a 0.610-inch diameter probe was used to inspect the tube. A 0.610-inch diameter bobbin probe has been able to traverse the ding in all inservice inspections.
- c. The licensee visually identified a metallic loose part on the periphery of SG B (the part was also detected during the eddy current examination). The licensee tried to remove the loose part but was unsuccessful because the loose part has become firmly wedged between the tubes. The licensee observed that deposits had started to buildup around this location, indicating that the loose part has been there for some time. The licensee

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stated that no degradation has been found in the surrounding tubes, but four tubes were stabilized and plugged.

- d. There are several tubes in which the U-bend areas are in close proximity. The extent of the tube to tube proximity has not changed (based on eddy current examination). The licensee, however, did not screen the tubes to identify whether additional tubes may be identified as being in close proximity since it was not expected that the extent of condition would change with time. The NRC staff notes that industry experience (ADAMS No. ML042020262) has shown that new cases of tube to tube proximity could develop as the operational time increases.

Based on a review of the information provided, the NRC staff concludes that the licensee provided the information required by its Technical Specifications. In addition, the staff concludes that there are no technical issues that warrant follow-up action at this time since the inspections appear to be consistent with the objective of detecting potential tube degradation and the inspection results appear to be consistent with industry operating experience at similarly designed and operated units.