



ŕ

COLUMBIANA HI TECH LLC

Nuclear Manufacturing Excellence

July 25, 2006

DWO-06-029

Bill Brach, Director Spent Fuel Project Office Office of Nuclear Material Safety and Safeguards United States Nuclear Regulatory Commission 11545 Rockville Pike Rockville, MD 20852

Cc: Director, Document Control Desk, SFPO (without attachments)

Re: Certificate of Compliance No. 9291 <u>Rev 5</u> for the Model No. Liqui-Rad (LR) Transport Unit Package. (copy attached)

Mr. Brach,

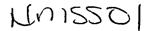
Columbiana Hi Tech, LLC would like to request a renewal of the above noted Certificate of Compliance.

A consolidated SAR was submitted as a previous licensing action to upgrade the approval to a -96 status. This approval was received on March 3, 2006 (C of C Rev 5). No changes have been made to the package since this submittal and approval.

Should you have any questions concerning this request, please feel free to contact me at your convenience.

Sincerely

Donald W. Olson President Columbiana Hi Tech





UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

March 3, 2006

Mr. Donald W. Olson President Columbiana Hi Tech, LLC 1802 Fairfax Road Greensboro, N.C. 27407

SUBJECT: CERTIFICATE OF COMPLIANCE NO. 9291 FOR THE MODEL NO. LIQUI-RAD (LR) TRANSPORT UNIT PACKAGE

Dear Mr. Olson:

As requested by your application dated August 5, 2005, as supplemented February 17, 2006, enclosed is Certificate of Compliance No. 9291, Revision No. 5, for the Model No. Liqui-Rad (LR) Transport Unit Package. Changes made to the enclosed certificate are indicated by vertical lines in the margin. The staff's Safety Evaluation Report is also enclosed.

The approval constitutes authority to use the package for shipment of radioactive material and for the package to be shipped in accordance with the provisions of 49 CFR 173.471. Those on the attached list have been registered as users of the package under the general license provisions of 10 CFR 71.17 or 49 CFR 173.471. Registered Users may request by letter to remove their names from the Registered Users List.

If you have any questions regarding this certificate, please contact me or Shawn Williams of my staff at (301) 415-8500.

Sincerely,

Robert A. Nelson, Chief Licensing Section Spent Fuel Project Office Office of Nuclear Material Safety and Safeguards

Docket No. 71-9291 TAC No. L23883

- Enclosures: 1. Certificate of Compliance
 - No. 9291, Rev. No. 5
 - 2. Safety Evaluation Report
 - 3. Registered Users List

cc w/encls 1 & 2: R. Boyle, Department of Transportation J. Shuler, Department of Energy RAMCERTS Registered Users

••••••••••••••••••••••••••••••••••••••				<u></u>		
NRC FORM 61 (8-2000) 10 CFR 71	18		TE OF COMPLI		JLATORY COMM	ISSION
1. a. CERTIF	CATE NUMBER	b. REVISION NUMBER	C. DOCKET NUMBER	d. PACKAGE IDENTIFICATION NUMBER	PAGE	PAGES
L	9291	5	71-9291	USA/9291/B(U)F-96	1 OF	3
2. PREAMBL	.E					
	ertificate is issued to certify tha rth in Title 10, Code of Federal I			ed in Item 5 below meets the applica ation of Radioactive Material."	ble safety standar	ds
				of the regulations of the U.S. Depart ugh or into which the package will be		ation
3. THIS CER	TIFICATE IS ISSUED ON THE	BASIS OF A SAFETY	ANALYSIS REPORT OF	THE PACKAGE DESIGN OR APPL		
a. ISSUI	ED TO (Name and Address)			ENTIFICATION OF REPORT OR AP	PLICATION	
	Imbiana Hi Tech, LLC	SCHERF	Columbian	a Hi Tech, LLC, consolidate	ed application	l
	2 Fairfax Road	rs Al	Edated Feb	ruary 17, 2006.		
Gree	ensboro, NC 27407	Carlin bree	5.5°	6.6		
	4	AN THE CAR			•	
	**** {****	े के बिट के ब बिट के बिट के		Ċ.		
4. CONDITIO	.45			the second ball		
	cate is conditional upon tuitiing	the requirements of 10	UFR Part / I, as applica	ble, and the conditions specified bel	JW.	
5.	al de la companya de	A CONTRACTOR OF THE STATE				
(a) Pa	ickaging					
14) Model No: _Liqui-l					1
(1)		Had (Lh) i ranspo	on-onit-Package			ł
(2)	Description	San an	HH			
	The LR Package i	s designed to trai	nsport Type B qua	antities of fissile uranyl nitra	ate solutions.	
				ns to protect the containme		
				primary structural compone		
				essel, a carbon steel outer		
				el is built in accordance wit loes not require an ASME s		1
				nary and secondary lids pro		I
	tight seal which is	leak testable. A	closed-cell pheno	lic foam or polyurethane fo	am	
				inment vessel and ceramic		•

surrounds the top and bottom head area of the containment vessel and ceramic fiber blanket and board insulation are used in the sidewalls and outer lid for thermal insulation and impact absorption. The maximum volume of the contents is limited to 230 gallons which maintains a minimum ullage of 33 gallons.

The LR is a cylindrical package set in a rectangular angle frame. The dimensions of the package are approximately $56"(I) \times 56"(w) \times 73"(h)$. The maximum weight of the package is 5692 pounds. The outer vessel is constructed of 10 gauge carbon steel. The containment vessel is constructed of 1/4 inch stainless steel with 1/4 inch thick flanged and dished heads. The containment vessel is rated at 50 psig pressure. Closed-cell phenolic or polyurethane foam and ceramic fiber insulation are sandwiched between the containment vessel and the package's outer shell.

The package is designed to be leak-tight (maximum allowable leakrate of 1×10^{-7} ref-cm³/sec). The containment vessel is closed using a double O-ring and is secured by sixteen 5/8 inch stainless steel studs. The outer lid is closed with four 5/8 inch

 9291 <u>5</u> 71-9291 USA/9291/B(U)F-96 <u>2</u> OF 5.(a)(2) Continued stainless steel bolts and nuts. The package is also equipped with plastic plugs to vent any gases that may be generated by the insulation during a fire event. All valves and fittings are provided within sealed enclosures to contain any leakage during valve failure. (3) Drawings The packaging is constructed and assembled in accordance with Columbiana Hi Tech Drawing Nos. LR-SAR, Sheets 1 through 4-Rev. 7. 	IRC FOI 3-2000) D CFR 71	RM 618			TE OF COMPL		GULATORY COMM	AISS
stainless steel bolts and nuts. The package is also equipped with plastic plugs to vent any gases that may be generated by the insulation during a fire event. All valves and fittings are provided within sealed enclosures to contain any leakage during valve failure. (3) Drawings The packaging is constructed and assembled in accordance with Columbiana Hi Tech Drawing Nos. LR-SAR, Sheets 1 through 4, Rev. 7. 5.(b) Contents (1) Type and form of material Low enriched Uranyl Nitrate solutions with the specifications shown in Table 1 below. The uranium concentration must be less than or equal to 125 GU/liter with an enrichment less than or equal for 5.0 with 20235. Non-fissile chemical impurities may be present up to the chemical impurity specification in Table 1. (2) Maximum quantity of material be package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material be package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material be package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material be package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material be package (3) Drawing (4) Diable 1. </th <th>. a. C</th> <th>ERTIFICATE</th> <th></th> <th></th> <th></th> <th></th> <th></th> <th>P</th>	. a. C	ERTIFICATE						P
gases that may be generated by the insulation during a fire event. All valves and fittings are provided within sealed enclosures to contain any leakage during valve failure. (3) Drawings The packaging is constructed and assembled in accordance with Columbiana Hi Tech Drawing Nos. LR-SAR, Sheets 1 through 4, Rev. 7. 5.(b) Contents (1) Type and form of material Low enriched Uranyi Nitrate solutions with the specifications shown in Table 1 below. The uranium concentration must be less than or equal to 125 gU/liter with an enrichment less than or equal to 5.0 wi% U 235. Non-fissile chemical impurities may be present up to the chemical impurity specification in Table 1. Additionally, fissile isotopes are also limited to the quantities in Table 1. Additionally, fissile isotopes are also limited to the quantities of Uranyi Nitrate solution with limits as shown in table 1. (2) Maximum Guantity, offmaterial ber package 230 gallons of Uranyi Nitrate solution with limits as shown in table 1. (2) Maximum Guantity offmaterial ber package 230 gallons of Uranyi Nitrate solution with limits as shown in table 1. (2) Maximum Guantity offmaterial ber package 230 gallons of Uranyi Nitrate solution with limits as shown in table 1. (2) Maximum Guantity offmaterial ber package 230 gallons of Uranyi Nitrate solution with limits as shown in table 1. Uranyi Nitrate solution with limits as shown in table 1. Uranyi Nitrate solution with limits as shown in table 1. Uranyi Mitrate solution with limits as shown	5.(a)(2)	Continued					
The packaging is constructed and assembled in accordance with Columbiana Hi Tech Drawing Nos. LR-SAR, Sheets 1 through 4. Rev. 7. 5.(b) Contents (1) Type and form of material Low enriched Uranyl Nitrate solutions with the specifications shown in Table 1 below. The uranium concentration must be less than or equal to 125 gU/literwith an enrichment less than or equal to 5.0 wto U235. Non-fissile chemical impurity specification in Table 1. Additionally, fissile isotopes are also limited to the quantities in Table 1. (2) Maximum quantity of material per package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material per package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material per package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material per package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material per package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material per package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. (2) Maximum quantity of Uranyl Nitrate solution with limits as shown in table 1. (2) Maximum quantity of Uranyl Nitrate solution with limits as shown in table 1. (1) Uranyl Nitrate so			gases that may be	generated by th	ne insulation durir	ng a fire event. All valves a		Э
Drawing Nos. LR-SAR, Sheets 1 through 4-Rev. 7. 5.(b) Contents (1) Type and form of material Low enriched Uranyl-Nitrate solutions with the specifications shown in Table 1 below. The uranium concentration must be less than or equal to 125-0U/lite with an enrichment less than or equal to 5.0 wt% U235. Non-fissile chemical impurities may be present up to the chemical impurity specification in Table 1- Additionally, fissile isotopes are also limited to the quantities in Table 1. (2) Maximum quantity of material per package 230 gallons of Urani Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material per package 230 gallons of Urani Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material per package 230 gallons of Urani Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material per package 230 gallons of Urani Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material per package 230 gallons of Urani Nitrate solution with limits as shown in table 1. (2) Maximum quantity of material per package 230 gallons of Urani Nitrate solution with limits as shown in table 1. (2) Maximum quantity of limits as shown in table 1. (2) Maximum quantity 0.11-0.7, Uranium Concentration solution solu		(3)	Drawings					
Low enriched Uranyl-Nitrate solutions with the specifications shown in Table 1 below. The uranium concentration must be less than or equal to 125 gU/liter with an enrichment less than or equal to 5.0 wt% U235. Non-fissile chemical impurities may be present up to the chemical impurity specification in Table 1. Additionally, fissile isotopes are also limited to the quantities in Table 1. (2) Maximum quantity of material per package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. ITEM Numerical impurities SPECIFICATION Solution Density SPECIFICATION Solution Density 0.14 0.74 Uranium Concentration ≤125 gU/l Uranium Concentration ≤125 gU/l U-232 ≤2.0E+03 µg/gU U-234 ≤2.0E+03 µg/gU U-235 ≤0.05 g/gU (12 pounds maximum quantity of U-235 per LR) U-236 ≤2.5E+04 µg/gU U-238 remainder of uranium				AR, Sheets 1 th			Hi Tech	
Low enriched Uranyl-Nitrate solutions with the specifications shown in Table 1 below. The uranium concentration must be less than or equal to 125 gU/liter with an enrichment less than or equal to 5.0 wt% U235. Non-fissile chemical impurities may be present up to the chemical impurity specification in Table 1. Additionally, fissile isotopes are also limited to the quantities in Table 1. (2) Maximum quantity of material per package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. ITEM Numerical impurities SPECIFICATION Solution Density SPECIFICATION Solution Density 0.14 0.74 Uranium Concentration ≤125 gU/l Uranium Concentration ≤125 gU/l U-232 ≤2.0E+03 µg/gU U-234 ≤2.0E+03 µg/gU U-235 ≤0.05 g/gU (12 pounds maximum quantity of U-235 per LR) U-236 ≤2.5E+04 µg/gU U-238 remainder of uranium	5.(b)	Conte	ents	of the		Lo.		
Low enriched Uranyl-Nitrate solutions with the specifications shown in Table 1 below. The uranium concentration must be less than or equal to 125 gU/liter with an enrichment less than or equal to 5.0 wt% U235. Non-fissile chemical impurities may be present up to the chemical impurity specification in Table 1. Additionally, fissile isotopes are also limited to the quantities in Table 1. (2) Maximum quantity of material per package 230 gallons of Uranyl Nitrate solution with limits as shown in table 1. ITEM Numerical impurities SPECIFICATION Solution Density SPECIFICATION Solution Density 0.14 0.74 Uranium Concentration ≤125 gU/l Uranium Concentration ≤125 gU/l U-232 ≤2.0E+03 µg/gU U-234 ≤2.0E+03 µg/gU U-235 ≤0.05 g/gU (12 pounds maximum quantity of U-235 per LR) U-236 ≤2.5E+04 µg/gU U-238 remainder of uranium		(1)	Type and form of m	aterial				
ITEM SPECIFICATION Solution Density Solution Density Chemical Impurities ≤1500 µg/gU Nitric Acid Normality 0.14-0.7 Uranium Concentration ≤125 gU/l U-232 ≤2.0E-03 µg/gU U-234 ≤2.0E+03 µg/gU U-235 ≤0.05 g/gU (12 pounds maximum quantity of U-235 per LR) U-236 ≤2.5E+04 µg/gU U-238 remainder of uranium Pu/Np Alpha Activity ≤93 Bq/gU		(2)	uranium concentrati than or equal to 5.0 chemical impurity sp quantities in Table 1 Maximum quantity of	on must be les wt% U-235. N pecification in T fimaterial per p V. Nitrate solution	s than or equal to on-fissile chemic able 1. Additiona backage	125 gU/liter with an enric al impurities may be prese ally, fissile isotopes are als	hment less nt up to the	e
Chemical Impurities ≤1500 µg/gU Nitric Acid Normality 0.1-0.7 Uranium Concentration ≤125 gU/l U-232 ≤2.0E-03 µg/gU U-234 ≤2.0E+03 µg/gU U-235 ≤0.05 g/gU (12 pounds maximum quantity of U-235 per LR) U-236 ≤2.5E+04 µg/gU U-238 remainder of uranium Pu/Np Alpha Activity ≤93 Bq/gU			ITEM	HOLDE]
Nitric Acid Normality 4^{-} $0.14 \cdot 0.7$ Uranium Concentration $\leq 125 \text{ gU/l}$ U-232 $\leq 2.0E-03 \mu g/g U$ U-234 $\leq 2.0E+03 \mu g/g U$ U-235 $\leq 0.05 g/g U (12 \text{ pounds maximum quantity of U-235 per LR)}$ U-236 $\leq 2.5E+04 \mu g/g U$ U-238remainder of uraniumPu/Np Alpha Activity $\leq 93 \text{ Bq/g U}$	Solut	tion De	nsity			2 00		
Uranium Concentration $\leq 125 \text{ gU/l}$ U-232 $\leq 2.0\text{E-03 }\mu\text{g/gU}$ U-234 $\leq 2.0\text{E+03 }\mu\text{g/gU}$ U-235 $\leq 0.05 \text{ g/gU} (12 \text{ pounds maximum quantity of U-235 per LR)}$ U-236 $\leq 2.5\text{E+04 }\mu\text{g/gU}$ U-238remainder of uraniumPu/Np Alpha Activity $\leq 93 \text{ Bq/gU}$	Cher	nical In	npurities	ez				
U-232 ≤2.0E-03 µg/gU U-234 ≤2.0E+03 µg/gU U-235 ≤0.05 g/gU (12 pounds maximum quantity of U-235 per LR) U-236 ≤2.5E+04 µg/gU U-238 remainder of uranium Pu/Np Alpha Activity ≤93 Bq/gU	Nitric	Acid N	lormality		. 0.1-0.7			
U-232 ≤2.0E-03 µg/gU U-234 ≤2.0E+03 µg/gU U-235 ≤0.05 g/gU (12 pounds maximum quantity of U-235 per LR) U-236 ≤2.5E+04 µg/gU U-238 remainder of uranium Pu/Np Alpha Activity ≤93 Bq/gU	Uran	ium Co	ncentration	···	S125 gU/I			
U-235≤0.05 g/gU (12 pounds maximum quantity of U-235 per LR)U-236≤2.5E+04 μg/gUU-238remainder of uraniumPu/Np Alpha Activity≤93 Bq/gU	U-:	232			≤2.0E-03 µថ្	g/gU	•	
U-235 per LR) U-236 ≤2.5E+04 µg/gU U-238 remainder of uranium Pu/Np Alpha Activity ≤93 Bq/gU	U-:	234	·· ·		≤2.0E+03 µ	g/gU		
U-238remainder of uraniumPu/Np Alpha Activity≤93 Bq/gU	U-:	235			≤0.05 g/gU U-235 per L	(12 pounds maximum qua R)	ntity of	
Pu/Np Alpha Activity ≤93 Bq/gU	U-:	236			≤2.5E+04 μ	g/gU	<u> </u>	
	U-2	238			remainder o	furanium		
Gamma Emitters 0.515E-01 Ci	Pu/N	p Alpha	a Activity		≤93 Bq/gU		······	
	Gam	ma Emi	itters		0.515E-01 C)i		

•

-2000)) CFR 71	M 618		TE OF COMPLI		ULATORY CO	MMISSIC
a. CE	RTIFICATE NUMBER	FOR RADIOACT	IVE MATERIAL PA c. DOCKET NUMBER	d. PACKAGES	PAGE	PAG
	9291	5	71-9291	USA/9291/B(U)F-96		DF 3
			·····			_
5. (c)	Criticality Safety Index	0.0				
6.	In addition to the requirer	ments of Subpart	G of 10 CFR Par	: 71:		
	(a) The package mus Procedures in Cha			erated in accordance with	the Operat	ling
	Acceptance Tests	and Maintenance	e Program in Cha	ntained in accordance wit pter 8 of the application.		
7.	The package authorized provisions of 10 CFR 71.	by this certificate	ls hereby approve	ed for use under the gener	al license	
8.				nber USA/9291/B(U)F-85 lumber USA/9291/B(U)F-		
9.	Transport by air of fissile	material is not au	thorized.			
9. 10.	Transport by air of fissile Revision No. 4 of this cert	See As	ed until March 31	2007.		
10.	ALC: NO PARTY	tificate may be us	AR (A	2007.	·	
10. 11.	Revision No. 4 of this cert Expiration date: October	ifficate may be us 31,2006 BEF	ed until March 31			
10. 11.	Revision No. 4 of this cert Expiration date: October	ificate may be us 31, 2006: <u>REF</u> lidated application	ed until March 31 ERENCES n dated February	17, 2006	MISSION	
10. 11.	Revision No. 4 of this cert Expiration date: October	ificate may be us 31, 2006: <u>REF</u> lidated application	ed until March 31 ERENCES n dated February	17, 2006.	MISSION	
10. 11.	Revision No. 4 of this cert Expiration date: October	ificate may be us 31,2006: REF lidated application	ed until March 31 ERENCES n dated February THE U.S. NUCLE	17, 2006 AR REGULATORY COM	MISSION	
10. 11.	Revision No. 4 of this cert Expiration date: October	ifficate may be us 31,2006 BEF Ilidated application FOR T Rober Licens	ed until March 31	AR REGULATORY COM	MISSION	

•

.



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION REPORT

Docket No. 71-9291 Model No. Liqui-Rad (LR) Transport Unit Package Certificate of Compliance No. 9291 Revision No. 5

SUMMARY

By letter dated August 5, 2005, Columbiana Hi Tech, LLC, (CHT) requested a minor change to the drawings and "-96" designation to Certificate of Compliance (CoC) No. 9291 for the Model No. Liqui-Rad (LR) Transport Unit Package. Staff requested additional information (RAI) dated November 7, 2005. CHT submitted a response to the RAI on February 17, 2006.

EVALUATION

By letter dated August 5, 2005, CHT requested "-96" designation to CoC 9291 for the Model No. Liqui-Rad (LR) Transport Unit Package. This application included revised pages to the Safety Analysis Report (SAR) that included minor editorial changes and revisions to meet the current regulation requirements that became effective on October 1, 2004 (69 FR 3698). A minor drawing revision was requested to make the Manual Vent Enclosure (MVE) optional. The MVE does not provide containment and the option of installing a MVE does not affect the package's ability to meet the requirements of 10 CFR Part 71.

In response to the staff's RAI, the applicant submitted a consolidated application on February 17, 2006. Staff reviewed the documents submitted in the consolidated application and concluded that the application incorporated the changes to the SAR that were previously referenced in the CoC.

Containment

The final rule, Issue No. 3, adopted changes to the A_1 and A_2 values, with the exception of two radionuclides. CHT updated the containment analysis in Chapter 4 of the SAR to incorporate the revised A_2 values. In general, the A_2 values increased. This caused the calculated allowable leakage rate to increase. The staff reviewed the revised mixture A_2 calculations presented in Table 4-2 and concluded that they were performed correctly. The staff then verified that the previously approved methodology of Section 4.2.1 and Section 4.3.2 was utilized to determine the packages allowable leakage rate for normal conditions of transport and hypothetical accident conditions, respectively. The staff found that the allowable leakage rate was conservative because it used the unrevised information presented in Tables 4-3 and 4-4 and was therefore acceptable.

The post fabrication leakage rate remained unchanged at leaktight conditions, but the periodic and pre-shipment leakage tests were increased to 1×10^{-3} ref-cc/sec, which is justified by the information presented in Chapter 4.

The applicant requested that the MVE be optional. The MVE includes a valve on the outer lid to vent the annulus before removing the outer lid. This valve, the annulus, and outer lid are not part of the containment boundary and thus have no affect upon the containment evaluation.

The staff finds that the package, as revised, meets the containment requirements of 10 CFR Part 71 and the containment criteria of ANSI N 14.5-1997.

Evaluation for a "-96" Designation

The amendment also requested a "-96" designation in the package identification number, as specified in 10 CFR 71.19(e). In the February 17, 2006, application, CHT addressed the 19 issues considered in the rulemaking process that resulted in the revised rule (69 FR 3698). The applicant discussed the impacts and effects that each issue of the revised regulations would have on the transportation package and related impacts on the Certificate of Compliance. Based on the statements and representations in the application, the staff concluded that the design has been adequately described and meets the requirements of 10 CFR Part 71 for a "-96" approval.

The following changes were made to the Certificate:

Condition No. 1.d. was revised to reflect the new "-96" package identification number.

Condition No. 3.b. has been revised to reflect the consolidated application submittal.

Condition No. 5(a)(1) has been revised to reflect the new name of the package. The Model No. Eco-Pak Liqui-Rad (LR) Transport Unit Package was requested to be changed to Model No. Liqui-Rad (LR) Transport Unit Package.

Condition No. 5(a)(2) was revised to correct the reference to ASME Pressure Vessel Code as Section VIII, Division 1. In the previous CoC, it was incorrectly referenced as Section VII. The words "or polyurethane foam" was also added in the description consistent with the materials authorized in the SAR.

Condition No. 5.(a)(3) was revised to correct the revision number of Drawing No. LR-SAR, Sheets 1, 2, and 3 to Revision No. 7. This editorial change was made so that all four sheets of Drawing No. LR-SAR, have the same revision number.

Condition No. 5(c) was revised to delete the wording "Minimum transport index to be shown on label for nuclear criticality control."

Condition No. 8 of the certificate allows a package to be marked with the previous package identification number, USA/9291/B(U)F-85, until March 31, 2007. This is to allow time to replace the packaging nameplate that shows the revised package identification number, USA/9291/B(U)F-96.

The applicant indicated in the February 17, 2006, response to RAI No. 1-2, that 10 CFR 71.55(f), which addresses packaging design requirements for packages transporting fissile material by air, are not applicable to this package. Therefore, for clarity, Condition No. 9 of the CoC was added to specify that transport by air of fissile material is not authorized.

Condition No. 10 was added to authorize use of the previous revision of the certificate for a period of approximately 1 year.

The Reference Section has been revised to include the consolidated application. All previous supplements have been removed.

As a consequence of the inclusion of the new Condition Nos. 8, 9, and 10, the previous Condition No. 8 was renumbered to Condition No. 11.

CONCLUSION

For the reasons stated in this Safety Evaluation Report, the staff concludes that the proposed changes do not affect the ability of the package to meet the requirements of 10 CFR Part 71.

:

.

.

.

Issued with Certificate of Compliance No. 9291, Revision No. 5 on March 3, 2006.

.

REGISTERED USERS OF CERTIFICATE OF COMPLIANCE NO. 9291

Mr. John Bossone Columbiana HI TECH, LLC 1802 Fairfax Road Greensboro, NC 27407

.

1-

Mr. Robert S. Freeman Framatome ANP, Inc. 1724 Mount Athos Road PO Box 11646 Lynchburg, VA 24506-1646

Ms. B. Marie Moore Nuclear Fuel Services, Inc. PO Box 337, MS 123 Erwin, TN 37650

Mr. James M. Shuler U. S. Department of Energy EM-24/CLV-1081 1000 Independence Ave., S. W. Washington, DC 20585-2040

Don Olson

From:"MONTGOMERY, Richard" <Richard.Montgomery@areva.com>To:"Don Olson" <dolson@chtnuclear.com>; "Rod Felts" <rfelts@chtnuclear.com>Sent:Friday, July 14, 2006 3:58 PMSubject:Liqui-Rad

Don, Rod,

Note that the LR SAR expires on October 31, 2006. You will need to request a renewal and indicate that a consolidated SAR was submitted as a previous licensing action to upgrade the approval to a -96 status. I like to give the NRC at least a 6-month window.

Richard D. Montgomery, Advisory Engineer **Nuclear Criticality Safety & Shipping Containers AREVA NP Inc.** P.O. Box 11646 Lynchburg, VA 24506-1646 *Office: 434-832-5172 Fax: 434-832-5060 Mobile: 434-221-8340* **Note: Effective March 15, 2006, my email address will change to:** <u>richard.montgomery@areva.com</u>

5HAWN WILLIAMS 301-415-8500