UNITED STATES



NUCLEAR REGULATORY COMMISSION

REGION II SAM NUNN ATLANTA FEDERAL CENTER 61 FORSYTH STREET, SW, SUITE 23T85 ATLANTA, GEORGIA 30303-8931

July 24,2004

Areva NP - Lynchburg ATTN: Mr. Dominique Grandemange Site Manager Mount Athos Road Facility P. O. Box 11646 Lynchburg, Virginia 24506-1646

SUBJECT: NRC INSPECTION REPORT NO. 70-1201/2006-004

Dear Mr. Grandemange:

This refers to the results of the above-referenced Nuclear Regulatory Commission (NRC) inspection conducted at your Lynchburg, Virginia facility on July 10 - 14, 2006. The purpose of the inspection was to determine whether activities authorized by the license were conducted safely and in accordance with NRC requirements. At the conclusion of the inspection on July 14, 2006, the NRC inspector discussed the findings with you and members of your staff.

As a result of the inspection, the enclosed NRC Forms 591FF, SAFETY INSPECTION REPORT, Parts 1 and 3, are being issued. The enclosed forms indicate that no violations were identified during the inspection of your licensed activities. Please retain the form in your files. No acknowledgment of this letter is required.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be made available electronically for public inspection in the NRC Public Document Room or from the NRC's document system (ADAMS), accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html.

Should you have any questions concerning this letter, please contact us.

Sincerely,

/**RA**/

David A. Ayres, Chief Fuel Facility Inspection Branch 1 Division of Fuel Facility Inspection

Docket No. 70-1201 License No. SNM-1168

Enclosure: NRC Form 591FF Parts 1 and 3

D. Grandemange

cc w/encl: Charlie Holman, Manager Environmental, Health, Safety and Licensing Areva NP, Inc. Lynchburg Manufacturing Facility P. O. Box 11646 Lynchburg, VA 24506-1646

Leslie P. Foldesi, CHP, Director Bureau of Radiological Health Division of Health Hazards Control Department of Health Main Street Station 1500 East Main, Room 240 Richmond, VA 23219

D. Grandemange

Distribution w/encl: D. Ayres, RII A. Gooden, RII N. Baker, NMSS J. Henson, RII G. Janosko, NMSS D. Morey, NMSS B. Reilly, NMSS PUBLIC

X PUBLICLY AVAILABLE

□ NON-PUBLICLY AVAILABLE

X NON-SENSITIVE

ADAMS:
Yes ACCESSION NUMBER:

OFFICE	RII:DFFI	RII:DFFI										
SIGNATURE	/RA/	RGbson for										
NAME	R. Gibson	A. Gooden										
DATE	07/24/2006	07/24/2006	07/	/2006	07/	/2006					1	
E-MAIL COPY?	YES	YES	YES	NO	YES	NO	YES	NO	YES	NO	YES	NO

OFFICIAL RECORD COPY DOCUMENT NAME: E:\Filenet\ML062060190.wpd

NRC FORM 591FF PA	RT 1			U.S. NUCLEAR REGULAT	ORY COMMISSION		
10 CFR 2.201	SAFETY INS	SPECTION REPORT	AND COMPLIANCE I	NSPECTION			
1. LICENSEE/LOCATIO Areva NP, Inc. P. O. Box 11646 Lynchburg, VA 24		2	 NRC/REGIONAL OFFICE U.S. Nuclear Regulatory Commission Region II, Division of Fuel Facilities Inspection 61 Forsyth Street, Suite 23T85 Atlanta, GA 30303 				
REPORT 3. DOCKET NUMBER(<u></u>				NODEOTION		
	-1201	4. LICENSE NUMBE SNM	I-1168	5. DATE(S) OF I July 10 - 1			
LICENSEE:							
Regulatory Commission	(NRC) rules and regulatio	ns and the conditions of your	license. The inspection con	afety and to compliance with sisted of selective examination spection findings are as follo	ins of		
		ings, no violations we	re identified.				
2. Previous	s violation(s) closed.						
3. The violation(s), specifically described to you by the inspector as non-cited violations, are not being cited because they were self-identified, non-repetitive, and corrective action was or is being taken, and the remaining criteria in the NRC Enforcement Policy, NUREG-1600, to exercise discretion, were satisfied.							
	Non-Cited Violation(s) wa	as/were discussed involving th	e following requirement(s) a	nd Corrective Action(s):			
cited. This form		activities, as described below TION, which may be subject		ation of NRC requirements ar 1 10 CFR 19.11.	nd are being		
	1	icensee's Statement of Corre	ctive Actions for Item 4, abo	10			
Licensee's Statement of Corrective Actions for Item 4, above. I hereby state that, within 30 days, the actions described by me to the inspector will be taken to correct the violations identified. This statement of corrective actions is made in accordance with the requirements of 10 CFR 2.201 (corrective steps already taken, corrective steps which will be taken, date when full compliance will be achieved). I understand that no further written response to NRC will be required, unless specifically requested. Title Printed Name Signature Date LICENSEE'S							
REPRESENTATIVE NRC INSPECTOR	Richard	l Gibson, Jr.			7/20/06		

U.S. NUCLEAR REGULATORY COMMISSION

SAFETY INSPECTION REPORT AND COMPLIANCE INSPECTION

1. LICENSEE

Areva NP. Inc.

P. O. Box 11646

2. NRC/REGIONAL OFFICE

U.S. Nuclear Regulatory Commission Region II, Division of Fuel Facilities Inspection 61 Forsyth Street, Suite 23T85 Atlanta, GA 30303

REPORT NUMBER(S): 2006-004

Lynchburg, VA 24506-1646

3. DOCKET NUMBER(S):	4. LICENSE NUMBER(S):	5. DATE(S) OF INSPECTION:		
70-1201	SNM-1168	July 10 - 14, 2006		

6. INSPECTOR(S): R. Gibson, Jr.

7. INSPECTION PROCEDURES USED: 83822, 88045, 88035 and 84900

SUPPLEMENTAL INSPECTION INFORMATION

Executive Summary

The Areva Lynchburg Facility fabricates UO_2 pellets into fuel assembly for use in nuclear power reactors. In addition, the licensee conducts activities related to work on equipment potentially contaminated with by-product material. During the period of the inspection, there were no plant upsets or unusual operational occurrences.

This routine, announced inspection included observations and evaluation of environmental protection, waste management, low level radioactive waste, and radiological controls specific to the Service Equipment Refurbishment Facility (SERF) 5. The inspection involved observations of work activities, reviews of selected procedures and records, and interviews with plant personnel. The inspection results were as outlined below:

Radiation Protection (83822)

SERF-5

- Radiation protection program self-assessments and procedure changes in the SERF 5 building were implemented in accordance with the license requirements.
- The external and internal exposure monitoring program was implemented to facilitate as low as reasonably achievable (ALARA) goals. Exposures were less than the limits in 10 CFR 20.1201.
- Radiological safety postings, labeling and radiation work permits were properly used to communicate potential hazards and protective equipment requirements to workers.
- The radiation and contamination survey programs were appropriately implemented to protect workers and identify potential radiation hazard areas. The licensee's staff was cognizant of the active radiation work permits and current radiation and contamination survey maps were available.
- The ALARA programs were adequately implemented to protect workers.

Executive Summary (continued)

Environmental Protection (88045)

- The licensee's environmental monitoring program was implemented in accordance with the license requirements. There were no major changes to the procedures since the last inspection.
- The environmental audit program was consistent with the requirements specified in the license application. The environmental program audits were thorough and corrective actions were tracked to resolution.
- An acceptable quality control program was maintained for collecting and analyzing measurements from environmental samples.
- Since the last inspection, environmental sampling results for soil, vegetation, river sediments and ambient air showed uranium activities near background levels.
- The licensee adequately implemented the environmental monitoring requirements as set forth in the license application.

Waste Management (88035)

- The licensee does not have a liquid effluent stream from the facility. Liquid waste generated from the process was filtered through resins and evaporated through evaporators.
- The gaseous effluent monitoring program was effective in controlling and measuring effluents, and compliant with the requirements of the license. The effluent air sampling equipment, including the sample delivery lines, was properly maintained. Calculated offsite doses were below regulatory limits.

Low-level Radioactive Waste Storage

• The waste storage management program was adequately implemented and provided the information needed to ensure proper storage, safe shipment, and disposal of waste. Low-level radioactive waste was stored in accordance with regulatory requirements.

Items Opened, Closed, And Discussed

<u>Item Number</u>	<u>Status</u>	Description
70-1201/2005-001-02	Closed	IFI - Review Radioactive Materials Calculation for Transport. Based on a review of revised Exhibit A to Addendum 6 to Procedure SL-1600, and a discussion with the licensee, the procedure was revised to include in the calculations the documentation regarding formula units; the use of attenuation factors for the gamma constant; and the average of six dose rate measurements taken from the shipping package.