

November 7, 2006

Mr. David A. Christian
Sr. Vice President and Chief Nuclear Officer
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SUBJECT: MILLSTONE POWER STATION, UNIT NO. 2 - ISSUANCE OF AMENDMENT
RE: REVISION TO TECHNICAL SPECIFICATIONS TO MODIFY THE
DESCRIPTION OF PRESSURIZER POWER OPERATED RELIEF VALVES
AND PRESSURIZER SAFETY VALVES POSITION INDICATORS (TAC NO.
MC9463)

Dear Mr. Christian:

The Commission has issued the enclosed Amendment No. 294 to Facility Operating License No. DPR-65 for Millstone Power Station, Unit No. 2, in response to your application dated January 4, 2006.

The amendment revises Technical Specification 3/4.3.3.8, "Instrumentation, Accident Monitoring." The revisions modify the description of pressurizer power operated relief valves and pressurizer safety valves position indicators

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Victor Nerses, Senior Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosures:

1. Amendment No. 294 to DPR-65
2. Safety Evaluation

cc w/encls: See next page

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Millstone Power Station, Unit No. 2

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DOMINION NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-336

MILLSTONE POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 294
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Dominion Nuclear Connecticut, Inc., the licensee, dated January 4, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 294, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Harold Chernoff, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: November 7, 2006

ATTACHMENT TO LICENSE AMENDMENT NO. 294

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

3/4 3-32

3/4 3-35

Insert

3/4 3-32

3/4 3-35

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 294

TO FACILITY OPERATING LICENSE NO. DPR-65

DOMINION NUCLEAR CONNECTICUT, INC.

MILLSTONE POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

1.0 INTRODUCTION

By application dated January 4, 2006, Dominion Nuclear Connecticut, Inc. (DNC or the licensee) requested Nuclear Regulatory Commission (NRC or the Commission) approval of changes to the Millstone Power Station, Unit No. 2 (MPS2) Technical Specifications (TSs). The changes would modify the description of pressurizer power-operated relief valve (PORV) and pressurizer safety/relief valve (SRV) position indicators by eliminating the phrase "Acoustic Monitor".

2.0 REGULATORY EVALUATION

The NRC staff's evaluation of the acceptability of the proposed TS changes is based upon the following regulations and regulatory guides:

General Design Criterion (GDC) 13, "Instrumentation and Control", which states, in part: "Instrumentation shall be provided to monitor variables and systems over their anticipated ranges for normal operation, for anticipated operational occurrences, and for accident conditions as appropriate to assure adequate safety, including those variables and systems that can affect the fission process, the integrity of the reactor core, the reactor coolant pressure boundary, and the containment and its associated systems."

Regulatory Guide (RG) 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident," Revision 2 (December 1980).

3.0 TECHNICAL EVALUATION

In its application, the licensee stated that the phrase "Acoustic Monitor" is contained in MPS2's current TS Limiting Condition of Operation (LCO) items 4 and 6 of TS 3/4.3.3.8, Tables 3.3-11 and 4.3-7. The licensee is requesting approval to delete the phrase "Acoustic Monitor" from the TSs because it is information that is not necessary to meet the TS requirements.

Without the phrase, the TS requirement for the total and minimum number of channels required to be operable and to verify channel operability at the designated frequencies remains the same. The licensee stated that this would also be the case should the design of the position indication system be changed. Furthermore, the licensee stated that to delete a functional unit from the TS LCO, it must be demonstrated that the four criteria set forth in Section 50.36(c)(2)(ii) of Title 10 of the *Code of Federal Regulations* (10 CFR) no longer apply to the functional unit to be deleted.

The NRC staff notes that using 10 CFR 50.36 as a basis would be appropriate if DNC were justifying deletion of the instruments identified in items 4 and 6 in Table 3.3-11 and Table 4.3-7. This basis, however, is not appropriate for only a change in the names ("Acoustic Monitor") of the instruments.

In the MPS2 Updated Final Safety Analysis Report (UFSAR), the licensee stated that the MPS2 valve monitoring system conforms with NUREG-0578, "TMI-2 [Three Mile Island Nuclear Station, Unit 2] Lessons Learned Task Force Status Report and Short-Term Recommendations" July 1979. In particular, in NUREG-0578, Section 2.1.3, "Information to Aid Operations in Accident Diagnosis and Control", a recommendation was made to have direct indication of power-operated relief valve and safety valve position for pressurized water reactors. This recommendation was incorporated (as TMI Related Action Plan Task II.D.3) in an Action Plan issued by the NRC, NUREG-0660, "NRC Action Plan Developed as a Result of the TMI-2 Accident", Vol. I, May 1980. A clarification of TMI-related items approved for implementation by the Commission at that time was issued in NUREG-0737, "Clarification of TMI Action Requirements", November 1980.

NUREG-0660 also specified the issuance of RG 1.97 and that the NRC will issue requirements for licensees to meet appropriated portions of the RG and will review the licensee's designs for conformance to the RG. In the MPS2 UFSAR, the licensee committed to conforming to RG 1.97 (as noted in docketed letters as described below) and the items the licensee committed to were identified in Table 7.5-3 in the UFSAR. Table 7.5-3 summarizes the RG 1.91 information on file that documents which instruments are credited with monitoring the associated variables.

RG 1.97, Revision 2, recommends that Category 2 primary system SRV (including PORV and code valves) position instrumentation or flow-through or pressure-measuring (in relief valve lines) instrumentation be provided to monitor the operation status and to monitor for loss of coolant in the primary coolant system. RG 1.97 also recommends that Category 2 SRV position or main steam flow instrumentation be provided to monitor the operation of the secondary system (steam generator).

In a letter dated February 29, 1984, the licensee committed to provide Category 2 instrumentation for variable D-10, primary system SRV, to monitor the primary coolant system with a note that stated: "The Acoustic Valve Monitoring System (AVMS) has been upgraded during the 1983 refueling outage." In the same letter, the licensee also committed to provide Category 2 instrumentation for variable D-18, SRV position, to monitor the secondary system (steam generator). The information provided on variable D-18 did not mention the type of technology that would be used to monitor SRV position. Variables D-10 and D-18 (as identified in the February 29, 1984 letter) are included in TS 3/4.3.3.8, Tables 3.3-11 and 4.3-7, as Instruments 4 and 6, respectively.

In letters dated April 9, 1984, and August 7, 1986, the licensee repeated the information that was provided in the February 29, 1984, letter concerning variables D-10 and D-18. In letters dated November 22, 1989, and February 5, 1991, the NRC staff issued safety evaluations on the licensee's conformance to RG 1.97, Revision 2 for MPS2.

In a letter dated March 2, 1992, the licensee provided updated information concerning the MPS2 RG 1.97 instrumentation. The updated information did not include the earlier note concerning the use of the AVMS for variable D-10, thereby removing the commitment to use acoustic monitoring technology to monitor PORV position.

Since the licensee revised the commitment on variable D-10 to not state the technology that would be used to monitor PORV position, and has not made a commitment in this application on the technology that would be used to monitor SRV position, the NRC staff considers that no commitment exists on what technology would be used to monitor PORV or SRV position. Therefore, the licensee established that the phrase "Acoustic Monitor" can be deleted from the names of Instrument 4, "PORV Position Indicator Acoustic Monitor," and Instrument 6, "Safety Valve Position Indicator Acoustic Monitor," in TS 3/4.3.3.8, Tables 3.3-11 and 4.3-7. Furthermore, the NRC staff considers that the identification of the type of technology (such as Acoustic Monitor) to use for monitoring is not safety significant.

From the information provided above, the NRC staff determined that even with the phrase "Acoustic Monitor" deleted, GDC 13 continues to be met. Further, the NRC staff notes that the licensee is committed to providing Category 2 primary system SRV position instrumentation or flow-through or pressure-measuring instrumentation to monitor the operation status and to monitor for loss of coolant in the primary coolant system. Therefore, the NRC staff finds that deleting the phrase "Acoustic Monitor" is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The Connecticut State official agreed with the NRC staff's conclusion as stated in Section 6 of this Safety Evaluation.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area, as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (71 FR 10073). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Barry Marcus
Victor Nerses

Date: November 7, 2006