NMC
Committed to Nuclear Excellence

Nuclear Management Company, LLC

L-HU-06-029
10 CFR 50.90

July 13, 2006

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555-0001

Point Beach Nuclear Plant Units 1 and 2
Dockets 50-266 and 50-301
Renewed License Nos. DPR-24 and DPR-27

Supplement to Application For Technical Specification Improvement Regarding Steam Generator Tube Integrity

Reference 1) License Amendment Request (LAR) titled, "Application For Technical Specification Improvement Regarding Steam Generator Tube Integrity", dated February 16, 2006

By letter dated February 16, 2006, Nuclear Management Company (NMC) submitted the referenced LAR to adopt Technical Specification (TS) improvements regarding steam generator tube integrity provided in Technical Specification Task Force (TSTF) Standard Technical Specification Change Traveler TSTF-449, "Steam Generator Tube Integrity", Revision 4. This letter supplements the referenced LAR with the revised TS Table of Contents page showing the addition of new Specification 3.4.17 which Reference 1 proposed to include in the TS. NMC is submitting this supplement in accordance with the provisions of 10 CFR 50.90.

Enclosure 1 provides revised Table of Contents page ii.

The additional information provided in this supplement does not impact the conclusions of the Determination of No Significant Hazards Consideration and Environmental Assessment presented in the Reference February 16, 2006 submittal as supplemented May 11, 2006.

In accordance with 10 CFR 50.91, NMC is providing a copy of this letter and enclosure to the designated State Official.

Summary of Commitments

This letter contains no new commitments and no revisions to existing commitments.

700 First Street Hudson, Wisconsin 54016
Telephone: 715-377-3300

I declare under penalty of perjury that the foregoing is true and correct.
Executed on 13 July 2006.

Edward J. Weinkam
Director, Nuclear Licensing and Regulatory Services
Nuclear Management Company, LLC

Enclosures (1)

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Senior Resident Inspector, Point Beach Nuclear Plant, USNRC
State Official, Ms. Ave M. Bie – Public Service Commission of WI

ENCLOSURE 1

Proposed Technical Specification Table of Contents (final)

Point Beach Nuclear Plant Units 1 and 2

Technical Specification Table of Contents Page ii

3 page follows

TABLE OF CONTENTS

1.0 USE AND APPLICATION 1.1-1
1.1 Definitions 1.1-1
1.2 Logical Connectors 1.2-1
1.3 Completion Times 1.3-1
1.4 Frequency 1.4-1

2.0 SAFETY LIMITS (SLs) 2.0-1
2.1 SLs 2.0-1
2.2 SL Violations 2.0-1

3.0 LIMITING CONDITION FOR OPERATION (LCO) APPLICABILITY 3.0-1
3.0 SURVEILLANCE REQUIREMENT (SR) APPLICABILITY 3.0-4

3.1 REACTIVITY CONTROL SYSTEMS 3.1.1-1
3.1.1 SHUTDOWN MARGIN (SDM) 3.1.1-1
3.1.2 Core Reactivity 3.1.2-1
3.1.3 Moderator Temperature Coefficient (MTC) 3.1.3-1
3.1.4 Rod Group Alignment Limits 3.1.4-1
3.1.5 Shutdown Bank Insertion Limits 3.1.5-1
3.1.6 Control Bank Insertion Limits 3.1.6-1
3.1.7 Rod Position Indication 3.1.7-1
3.1.8 PHYSICS TESTS Exceptions-MODE 2 ..... 3.1.8-1

3.2 POWER DISTRIBUTION LIMITS 3.2.1-1
3.2.1 Heat Flux Hot Channel Factor ($F_Q(Z)$) (F_Q Methodology) 3.2.1-1
3.2.2 Nuclear Enthalpy Rise Hot Channel Factor ($F^N_{\Delta H}$) 3.2.2-1
3.2.3 AXIAL FLUX DIFFERENCE (AFD) 3.2.3-1
3.2.4 QUADRANT POWER TILT RATIO (QPTR) 3.2.4-1

3.3 INSTRUMENTATION 3.3.1-1
3.3.1 Reactor Protection System (RPS) Instrumentation 3.3.1-1
3.3.2 Engineered Safety Feature Actuation System (ESFAS) Instrumentation 3.3.2-1
3.3.3 Post Accident Monitoring (PAM) Instrumentation 3.3.3-1
3.3.4 Loss of Power (LOP) Diesel Generator (DG) Start and Load Sequence Instrumentation 3.3.4-1
3.3.5 Control Room Emergency Filtration System (CREFS) Actuation Instrumentation 3.3.5-1
3.3.6 Boron Dilution Alarm 3.3.6-1

3.4 REACTOR COOLANT SYSTEM (RCS) 3.4.1-1
3.4.1 RCS Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits 3.4.1-1
3.4.2 RCS Minimum Temperature for Criticality 3.4.2-1
3.4.3 RCS Pressure and Temperature (P/T) Limits 3.4.3-1
3.4.4 RCS Loops - MODES 1 and 2 ..... 3.4.4-1

TABLE OF CONTENTS

3.4 REACTOR COOLANT SYSTEM (continued)

3.4.5 RCS Loops - MODE 3 ..... 3.4.5-1
3.4.6 RCS Loops - MODE 4 ..... 3.4.6-1
3.4.7 RCS Loops - MODE 5, Loops Filled 3.4.7-1
3.4.8 RCS Loops - MODE 5, Loops Not Filled 3.4.8-1
3.4.9 Pressurizer 3.4.9-1
3.4.10 Pressurizer Safety Valves 3.4.10-1
3.4.11 Pressurizer Power Operated Relief Valves (PORVs) 3.4.11-1
3.4.12 Low Temperature Overpressure Protection (LTOP) 3.4.12-1
3.4.13 RCS Operational LEAKAGE 3.4.13-1
3.4.14 RCS Pressure Isolation Valve (PIV) Leakage 3.4.14-1
3.4.15 RCS Leakage Detection Instrumentation 3.4.15-1
3.4.16 RCS Specific Activity 3.4.16-1
3.4.17 Steam Generator (SG) Tube Integrity 3.4.17-1

3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS) 3.5.1-1
3.5.1 Accumulators 3.5.1-1
3.5.2 ECCS-Operating 3.5.2-1
3.5.3 ECCS-Shutdown 3.5.3-1
3.5.4 Refueling Water Storage Tank (RWST) 3.5.4-1

3.6 CONTAINMENT SYSTEMS 3.6.1-1
3.6.1 Containment 3.6.1-1
3.6.2 Containment Air Locks 3.6.2-1
3.6.3 Containment Isolation Valves 3.6.3-1
3.6.4 Containment Pressure 3.6.4-1
3.6.5 Containment Air Temperature 3.6.5-1
3.6.6 Containment Spray and Cooling Systems 3.6.6-1
3.6.7 Spray Additive System 3.6.7-1

3.7 PLANT SYSTEMS 3.7.1-1
3.7.1 Main Steam Safety Valves (MSSVs) 3.7.1-1
3.7.2 Main Steam Isolation Valves (MSIVs) and Non-Return Check Valves 3.7.2-1
3.7.3 Main Feedwater Isolation 3.7.3-1
3.7.4 Atmospheric Dump Valve (ADV) Flowpaths 3.7.4-1
3.7.5 Auxiliary Feedwater (AFW) System 3.7.5-1
3.7.6 Condensate Storage Tank (CST) 3.7.6-1
3.7.7 Component Cooling Water (CC) System 3.7.7-1
3.7.8 Service Water (SW) System 3.7.8-1
3.7.9 Control Room Emergency Filtration System (CREFS) 3.7.9-1
3.7.10 Fuel Storage Pool Water Level 3.7.10-1
3.7.11 Fuel Storage Pool Boron Concentration 3.7.11-1
3.7.12 Spent Fuel Pool Storage 3.7.12-1
3.7.13 Secondary Specific Activity 3.7.13-1

TABLE OF CONTENTS

3.8 ELECTRICAL POWER SYSTEMS 3.8.1-1
3.8.1 AC Sources-Operating 3.8.1-1
3.8.2 AC Sources-Shutdown 3.8.2-1
3.8.3 Diesel Fuel Oil and Starting Air 3.8.3-1
3.8.4 DC Sources-Operating 3.8.4-1
3.8.5 DC Sources-Shutdown 3.8.5-1
3.8.6 Battery Cell Parameters 3.8.6-1
3.8.7 Inverters-Operating 3.8.7-1
3.8.8 Inverters-Shutdown 3.8.8-1
3.8.9 Distribution Systems-Operating 3.8.9-1
3.8.10 Distribution Systems-Shutdown 3.8.10-1

3.9 REFUELING OPERATIONS 3.9.1-1
3.9.1 Boron Concentration 3.9.1-1
3.9.2 Nuclear Instrumentation 3.9.2-1
3.9.3 Containment Penetrations 3.9.3-1
3.9.4 Residual Heat Removal (RHR) and Coolant Circulation-High Water Level 3.9.4-1
3.9.5 Residual Heat Removal (RHR) and Coolant Circulation-Low Water Level 3.9.5-1
3.9.6 Refueling Cavity Water Level 3.9.6-1

4.0 DESIGN FEATURES 4.0-1
4.1 Site Location 4.0-1
4.2 Reactor Core 4.0-1
4.3 Fuel Storage 4.0-2

5.0 ADMINISTRATIVE CONTROLS 5.0-1
5.1 Responsibility 5.1-1
5.2 Organization 5.2-1
5.3 Unit Staff Qualifications 5.3-1
5.4 Procedures 5.4-1
5.5 Programs and Manuals 5.5-1
5.6 Reporting Requirements 5.6-1
5.7 High Radiation Area 5.7-1